

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 7 1	PAGE (3) OF 0 2
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TITLE (4)
Auxiliary Building Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 9	2 1	8 4	8 4	0 6 0	0 0 1	0 1	0 1	8 4			0 5 0 0 0
											0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) 1	20.402(b)	20.406(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 1 0 0	20.405(a)(1)(i)	50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Heyward R. Rogers, Compliance Section Engineer	TELEPHONE NUMBER 6 1 5 8 7 0 - 6 1 4 6
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Seven auxiliary building isolations occurred between the dates of 09/21/84 and 09/30/84. The inadvertent isolations were caused by spikes on the spent fuel pool radiation monitors. In all events the area was surveyed and no high radiation found and all systems returned to normal. There was no effect on public health or safety.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 7 8 4	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		— 0 6 0	— 0 0 0		2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This report describes seven inadvertent auxiliary building isolations caused by spikes on the spent fuel pool radiation monitors. For all incidents, the spent fuel pool area was surveyed by Health Physics and no indication of abnormal radiation levels were found. Operations personnel reset the signal, realigned all affected dampers to their normal position, and returned the ventilation systems to normal operation. The event dates included in this report are (1) 1906 CST on 09/21/84 from RM-90-102, (2) 1115 CST on 09/23/84 from RM-90-102, (3) 1730 CST on 09/24/84 from RM-90-102, (4) 0039 CST on 09/28/84 from RM-90-102, (5) 0228 CST on 09/28/84 from RM-90-102, (6) 2205 CST on 09/29/84 from RM-90-103, and (7) 0114 CST on 09/30/84 from RM-90-103.

The two spent fuel pool radiation monitors are Geiger-Mueller tubes manufactured by General Atomic (Models RD-1 and RP-1) mounted above the fuel pool and are designed to monitor the air spaces above the pool area. A high radiation signal initiates auxiliary building isolation and is designed to limit releases to the environment in the event of a fuel handling accident. For all events described, the auxiliary building isolations were caused from noise and/or background spikes reaching the monitor setpoint resulting in the automatic isolation. The range of these monitors is from 10^{-1} to 10^{+4} mr/hr and the technical specification allowable setpoint is 15 mr/hr. The spikes causing the isolations peaked at approximately the setpoint value for the events described. The noise was from normal EMF on the monitor signal cable and the background radiation was from contaminated waste material passing near the monitors or fuel pool water radiation levels.

The following corrective actions have been taken to reduce the number of inadvertent auxiliary building isolations:

- 1) A one-second time delay was added to the high radiation isolation circuit to help discriminate between an inadvertent spike and an actual high radiation condition.
- 2) The spent fuel pool water was cleaned through its own filtering system to reduce the background levels to the monitors.
- 3) Contaminated waste material near the spent fuel pool monitors has been moved to reduce background levels and efforts are being made to maintain this area clear of contaminated waste.
- 4) A technical specification change is being pursued to increase the allowable isolation actuation setpoint from the present 15 mr/hr to a higher value.

It is expected that corrective actions one through three will limit the number of ABIs; however, until the technical specification setpoint change is obtained, inadvertent isolations from these monitors are expected to continue. For the events in this report, there was no effect upon public health and safety.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

October 18, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

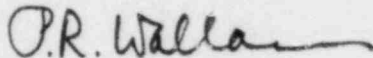
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO.
50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT
SQRO-50-327/84C60

The enclosed licensee event report provides details concerning seven incidents of inadvertent auxiliary building isolation from spent fuel pool radiation monitors. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



P. R. Wallace
Plant Manager

Enclosure
cc (Enclosure):

James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30323

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, NUC PR, Sequoyah

