



UNITED STATES  
**NUCLEAR REGULATORY COMMISSION**  
 WASHINGTON, D.C. 20555-0001

February 22, 1996

Mr. Roger O. Anderson, Director  
 Licensing and Management Issues  
 Northern States Power Company  
 414 Nicollet Mall  
 Minneapolis, Minnesota 55401

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NO. 2 - EVALUATION OF  
 THE THIRD 10-YEAR INTERVAL INSERVICE INSPECTION EXAMINATION PLAN AND  
 ASSOCIATED REQUESTS FOR RELIEF (TAC NO. M90187)

Dear Mr. Anderson:

Northern States Power Company (NSP) submitted its Third 10-Year Interval Inservice Inspection (ISI) Examination Plan, Revision 0, for the Prairie Island Nuclear Generating Plant, Unit 2, in a letter dated November 15, 1994. Included in the ISI Examination Plan were requests for relief from the American Society of Mechanical Engineers (ASME) Code Section XI requirements that you determined to be impractical. The NRC staff, with the help of its contractor, Idaho National Engineering Laboratory (INEL), reviewed Revision 0, and in response to NRC's May 30, 1995, request for additional information, NSP submitted additional information on July 13, 1995. After further staff review and a telephone conversation with the licensee, NSP submitted additional information for Unit 2 in a letter dated October 5, 1995.

The staff and its contractor have evaluated the information provided in Revision 0 to the Third 10-Year Interval ISI Examination Plan for Unit 2, including associated requests for relief. The staff's conclusions are contained in the enclosed Safety Evaluation. Based on the information submitted, the staff adopts the contractor's conclusions and recommendations presented in the attached Technical Evaluation Report (TER). The staff has concluded that for Requests for Relief Nos. RR-5, Rev. 2 (Part 2) and RR-6, the examinations required by the code are impractical and the proposed alternatives to Code requirements provide reasonable assurance of operational readiness. Therefore, relief is granted for Requests for Relief Nos. RR-5 Rev. 2 (Part 2) and RR-6 pursuant to 10 CFR 50.55a(g)(6)(i) as requested.

The staff has also concluded that the alternative contained in Request for Relief No. RR-5, Rev. 2 (Part 1) provides an acceptable level of quality and is authorized pursuant to 10 CFR 50.55a(a)(3)(i) as requested. Furthermore, the staff concluded that for Request for Relief No. RR-7, compliance with the Code requirements would result in a hardship without a compensating increase in safety, and that the proposed alternate testing provides reasonable assurance of operational readiness. Therefore, the alternative contained in Request for Relief No. RR-7 is authorized pursuant to 10 CFR 50.55a(a)(3)(ii) provided that when using the calibration blocks without the appropriate documentation, a comparison should be made between the acoustical properties of the calibration block and the material being examined. This comparison should be done once prior to the use of the calibration block, to ensure that the sensitivity is sufficient to find existing flaws in corresponding examination volumes.

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In letters dated April 19, and July 13, 1995, you withdrew Requests for Relief Nos. 1, 2, 3, and 4. Therefore, the staff did not review these requests.

If you have any questions regarding the enclosed Safety Evaluation or TER, please contact the NRC Project Manager, Beth Wetzel, at 301-415-1355.

Sincerely,

Original Signed By: T. Colburn for

John N. Hannon, Project Director  
Project Directorate III-1  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-306

Enclosure: Safety Evaluation

cc w/encl: See next page

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*T. Colburn*  
*2/22/96*

In letters dated April 19, and July 13, 1995, you withdrew Requests for Relief Nos. 1, 2, 3, and 4. Therefore, the staff did not review these requests.

If you have any questions regarding the enclosed Safety Evaluation or TER, please contact the NRC Project Manager, Beth Wetzel, at 301-415-1355.

Sincerely,



John N. Hannon, Project Director  
Project Directorate III-1  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-282

Enclosure: Safety Evaluation

cc w/encl: See next page

Mr. Roger O. Anderson, Director  
Northern States Power Company

Prairie Island Nuclear Generating  
Plant

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March 1995

DISTRIBUTION FOR ISI RELIEF LETTER DATED: February 22, 1996

Docket File

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