

LICENSEE EVENT REPORT (LER)

APPROVED ONE NO. 3160-0104
EXPIRES - 8/31/85

FACILITY NAME (1) Limerick Generating Station - Unit 1		DOCKET NUMBER (2) 0 1 5 1 0 1 0 1 3 5 2	PAGE (3) 1 OF 0 1 3
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TITLE (4)
Initiation of Shutdown due to HPCI Inoperability

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME		DOCKET NUMBER (2)
0 1	0 6	8 5	8 5	0 0 4		0 1	0 0	2 0 1 5 8 5			0 1 5 1 0 1 0 1 3 5 2

OPERATING MODE (8) 2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § 1.61 (Chart one or more of the following) (11)										
POWER LEVEL (10) 0 0 3	20.402(b)	20.406(a)	20.736(i)(2)(iv)	73.710(i)							
	20.406(b)(1)(i)(B)	20.36(a)(1)	20.736(i)(2)(v)	73.710(j)							
	20.406(b)(1)(i)(C)	20.36(a)(2)	20.736(i)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
	20.406(b)(1)(i)(D)	X 20.736(i)(2)(ii)	20.736(i)(2)(vii)(A)								
	20.406(b)(1)(i)(E)	20.736(i)(2)(iii)	20.736(i)(2)(vii)(B)								
	20.406(b)(1)(i)(F)	20.736(i)(2)(iv)	20.736(i)(2)(viii)								

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME	AREA CODE	NUMBER	
John C. Nagle, Engineer - Special Projects	2 1 5	8 4 1 - 5 1 8 4	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	B J	P T R	3 6 9	Y					

SUPPLEMENTAL REPORT EXPECTED (14)	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO			

ABSTRACT (Limit to 1600 spaces, i.e., approximately fifteen single-space typewritten lines) (16)
Abstract: 85-004

During start-up operations at approximately 1835 hours on January 6, 1985, a transmitter trip unit alarm light indicated GROSS FAIL/INOP for one of the HPCI low steam pressure transmitters (PT-55-IN058F). This instrument provides an input to the Nuclear Steam Supply Shutoff System (NSSS) isolation logic for the HPCI steam supply outboard isolation valve. Although not required by Technical Specifications, due to some questions regarding HPCI operability, the decision was made to initiate a shutdown. Further, a decision was made to close the associated isolation valve thus rendering HPCI inoperable. RCIC was also inoperable for maintenance; therefore, the unit was required to be placed in hot shutdown within six hours in accordance with the requirements of Technical Specification 3.0.3. Steps were immediately taken to return RCIC to operability such that at 2125 hours shutdown operations ceased and power ascension began. The pressure transmitter was repaired and subsequently returned to service.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 5 2 8 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		85	004	00	02	OF	03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description of the Event:

At approximately 1835 hours on January 6, 1985, a transmitter trip unit alarm indicated GROSS FAIL/INOP for PT-55-IN058F (HPCI steam supply pressure transmitter). Because of a question regarding the operability of HPCI, a shutdown was initiated at this time. This pressure transmitter trip unit (a Rosemont GP7A22MBGE3) provides an input for the isolation logic of the HPCI steam supply outboard isolation valve. Technical Specification 3.3.2 (Isolation Actuation Instrumentation) requires that the associated instrument channel be placed in a tripped condition within one hour. However, in order to be conservative, it was decided to place the outboard isolation valve in the closed position at 1854 hours. Closing the isolation valve rendered the HPCI system inoperable. Technical Specification 3.5.1.C requires that HPCI be restored to operable status within 14 days provided that Core Spray System, the Low Pressure Coolant Injection System the Automatic Depressurization System and the Reactor Core Isolation Cooling System are operable. Because the RCIC system was out-of-service for maintenance, the Action Statement for Technical Specification 3.0.3 requiring that the unit be in hot shutdown within six hours was implemented.

Consequences of the Event:

At the time of this event the unit was at approximately 4.5% power undergoing low power testing. Core spray, low pressure coolant injection, and automatic depressurization systems were available and HPCI could have been promptly returned to operable status, therefore, the consequences of this event are minimal.

Cause of the Event:

This event resulted from the licensed operator's electing to close the isolation valve, thus meeting the intent of the Technical Specifications. This was a conservative decision. Because RCIC was out-of-service, it was necessary to initiate a plant shutdown as required by the Technical Specifications.

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			0 0 4	-- 0 0 0	3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Immediate Corrective Actions:

Steps were immediately taken to return RCIC to operability such that at 2125 hours shutdown operations ceased and power ascension began. The pressure transmitter was repaired and subsequently returned to service.

Corrective Actions:

This matter has been discussed with operations personnel and a memo has been sent to the licensed operators which delineates the Technical Specification requirements regarding isolation valve instrumentation and the associated Action Statements. As a long term corrective action, the licensed operator requalification program is being modified to address identified Technical Specification interpretation. The next cycle of requalification is scheduled to begin in mid February.

PHILADELPHIA ELECTRIC COMPANY

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February 5, 1985

Docket No. 50-352

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Washington, DC 20555

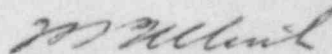
SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER concerns a plant shutdown initiated as a result of the inoperability of the High Pressure Coolant Injection system while the Reactor Core Isolation Cooling system was removed from service for maintenance.

Reference: Docket No. 50-352
Report Number: 85-004
Revision Number: 00
Event Date: January 6, 1985
Report Date: February 5, 1985
Facility: Limerick Generating Station
P.O. Box A, Sanatoga, PA 19464

This LER is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
J. T. Wiggins, Senior Site Inspector
See Service List

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January 16, 1985