NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
EDWIN I. HATCH, UNIT I	0 5 0 0 0 3 2 1	1 OF 0 2

TITLE (4)
RCIC Turbine Electrical and Mechanical Overspeed Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 1	0 9	8 5	8 5	— 0 0 8	— 0 0	0 2	0 7	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
2	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 0 1	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
T. L. Elton, Acting Superintendent of Regulatory Compliance	AREA CODE 9 1 2 — 3 6 7 — 7 8 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	B N	S C	A 1 2 3	Y		X	B N			Y	
X	B N	S C	T 1 4 7	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 01/09/85, with the reactor mode switch in the startup and hot standby position and reactor power at less than 1%, during performance of the "RCIC TURBINE OVERSPEED INSTRUMENT CALIBRATION" procedure (HNP-1-3403), and "RCIC TURBINE MECH OVERSPEED TRIP" procedure (HNP-1-5286), plant personnel noted that the electrical overspeed trip did not actuate within the specified limits of Tech. Specs. table 3.2-3, item 2 (i.e., less than or equal to 110% of rated speed), and the mechanical overspeed trip did not actuate within the specified limits of Tech. Specs. table 3.2-3, item 2 (i.e., less than or equal to 125% of rated speed) respectively.

On 01/11/85, with the reactor mode switch in the run position at 275 MWt (approximately 11% power), during performance of the "RCIC PUMP OPERABILITY" procedure (HNP-1-3405), in order to prove RCIC operable following an overspeed test, plant personnel noted that RCIC isolated. An investigation revealed that RCIC isolated due to an apparent high exhaust diaphragm pressure.

No actual or potential safety consequences or implications resulted from these events. These events had no impact on any other Unit 1 system or on Unit 2. The health and safety of the public were not affected by these events. These are non-repetitive events.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
EDWIN I. HATCH, UNIT I	0 5 0 0 0 3 2 1	8 5	— 0 0 8	— 0 0	0 2 OF 0 2

On 01/09/85, with the reactor mode switch in the startup and hot standby position and reactor power at less than 1%, during performance of the "RCIC TURBINE OVERSPEED INSTRUMENT CALIBRATION" procedure (HNP-1-3403), and "RCIC TURBINE MECH OVERSPEED TRIP" procedure (HNP-1-5286), plant personnel noted that the electrical overspeed trip did not actuate within the specified limits of Tech. Specs. table 3.2-3, item 2 (i.e., less than or equal to 110% of rated speed), and the mechanical overspeed trip did not actuate within the specified limits of Tech. Specs. table 3.2-3, item 2 (i.e., less than or equal to 125% of rated speed) respectively.

The electrical overspeed trip was repaired by replacing the electrical overspeed trip device's adjustment potentiometers. The electrical overspeed trip was then re-calibrated and functionally tested satisfactorily per HNP-1-3403 on 01/10/85. The mechanical overspeed trip was adjusted and functionally tested satisfactorily per HNP-1-5286 on 01/10/85.

On 01/11/85, with the reactor mode switch in the run position at 275 MWt (approximately 11% power), during performance of the "RCIC PUMP OPERABILITY" procedure (HNP-1-3405), in order to prove RCIC operable following an overspeed test, plant personnel noted that RCIC isolated. An investigation revealed that RCIC isolated due to an apparent high exhaust diaphragm pressure.

When performing HNP-1-3405, plant personnel made a increment change from approximately 300 GPM to approximately 400 GPM by switching the RCIC Pump Control M/A station from AUTO to MANUAL. However, this increment resulted in an instantaneous change in both RCIC discharge flow and pressure (i.e., discharge flow went from approximately 300 GPM to approximately 400 GPM and discharge pressure went from approximately 1000 PSIG to approximately 1200 PSIG). RCIC then isolated on an apparent high exhaust diaphragm pressure.

Inspection of the RCIC exhaust diaphragm revealed that it had been stressed (i.e., crushed in on one side). Based on engineering judgement, plant personnel postulated that the above mentioned flow and pressure transient resulted in the damage to the exhaust diaphragm and the resulting isolation.

The damaged exhaust diaphragm was replaced. RCIC was then functionally tested satisfactorily per HNP-1-3405 and returned to service on 1/12/85.

HNP-1-3405 will be reviewed to see if a revision is required to caution personnel in how to prevent an instantaneous change in pressure and flow when taking the RCIC Pump Control M/A Station from AUTO to MANUAL.

HPCI remained operable during these events.

No actual or potential safety consequences or implications resulted from these events. These events had no impact on any other Unit 1 system or on Unit 2. The health and safety of the public were not affected by these events. These are non-repetitive events.

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Edwin I. Hatch Nuclear Plant

Georgia Power

February 7, 1985
GM-85-100

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-321

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-321/1985-008. This report is required by 10CFR 50.73(a)(2)(i).

H. C. Nix
General Manager

HCN/TLE/vlz

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