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NRC WEBINAR

APRIL 21, 2020

INDIAN POINT
NUCLEAR POWER
PLANT
DECOMMISSIONING



Today's Presenters



Bruce Watson



Anthony Dimitriadis



Ted Carter



Richard Chang

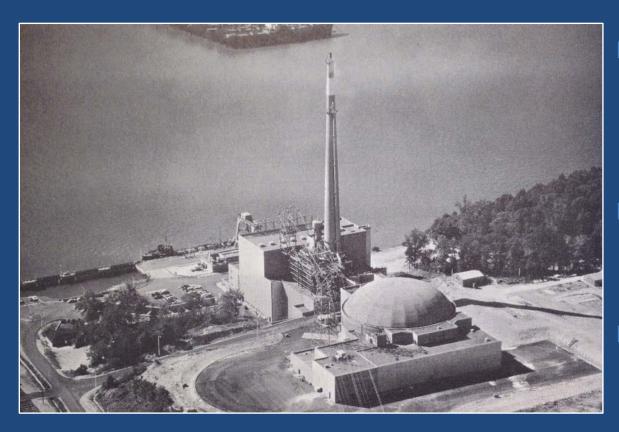


Katherine Warner

NRC Staff Response to COVID-19

- NRC staff have been engaged with the industry (NEI and the Industry Working Group) on decommissioning and ISFSIonly facilities.
 - NRC management has communicated with NEI to ensure the industry maintains safety and security vigilance on decommissioning and ISFSI-only sites during these unusual times.
 - NRC Project Managers and the Regional Inspectors have frequent communications with the licensees on COVID-19 issues and are not aware of any issues that would affect public health and safety at decommissioning reactors.

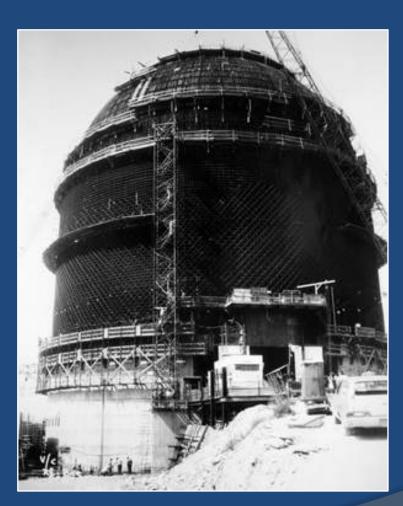
Indian Point History



- March 26, 1962 –
 Operating license issued for Indian Point 1
- Sept. 16, 1962 –Unit 1 begins operations
- Oct. 31, 1974 –Unit 1 ceases operations

Indian Point History (cont'd.)

- Oct. 14, 1966 –Constructionpermit issued forIndian Point 2
- Aug. 1, 1974 –Unit 2commencesoperations
- January 2017 –Shutdownagreementannounced
- Sept. 17, 2018 –Unit 2 license is renewed by NRC



- Aug. 14, 1969 Construction permit issued for Indian Point 3
- Aug. 30, 1976 –Unit 3commencesoperations
- January 2017 Shutdown agreement announced
 - Sept. 17, 2018 Unit 3 license renewed by NRC U.S.NRC

Proposed Change in Ownership



- April 2019 Holtec and Entergy announce deal for Holtec to acquire and decommission Indian Point
- Nov. 21, 2019 –
 NRC receives
 license transfer
 application
- Jan. 3, 2020 NRC accepts application for formal review

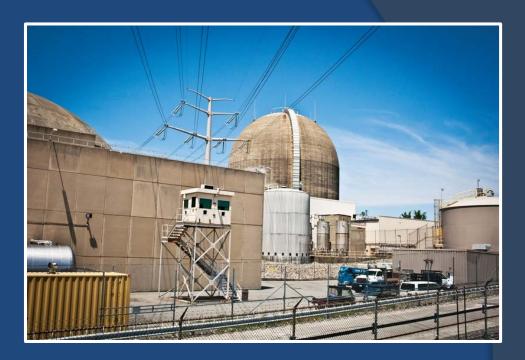
Hearing Requests

Requests for a hearing on license transfer application remain under consideration by the Commission; NRC staff are considering comments received



Near-term Developments

- Certification of permanent cessation of operations
- Certification of permanent removal of fuel from reactor



 Review of Post-Shutdown
 Decommissioning
 Activities Report
 (PSDAR)

Accession No. ML19354A698

Post-Shutdown Decommissioning Activities Report (PSDAR)

It should contain:

- Description of planned decommissioning activities
- High-level schedule of planned decommissioning activities
- Site-specific cost estimate for the decommissioning
- Environmental impacts of decommissioning



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December 19, 2019

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

> Indian Point Nuclear Generating Units 1, 2 and 3 Docket Nos. 50-3, 50-247, 50-286 and 72-051 Provisional Operating License No. DPR-5 Renewed Facility Operating License Nos. DPR-26 and DPR-64

Subject: Post Shutdown Decommissioning Activities Report including Site-Specific Decommissioning Cost Estimate for Indian Point Nuclear Generating Units 1, 2,

and 3

Reference: [1] Letter US NRC to Con Edison, "Order to Authorize Decommissioning and Amendment No. 45 to License No. DR-5 for Indian Point Unit No. 1," January 31, 1996 (ML070310227)

- [2] Letter from ENOI to US NRC, "Notification of Permanent Cessation of Power Operations Indian Point Nuclear Generating Unit Nos. 2 and 3, Docket Nos. 50-247 and 50-286, License Nos. DPR-26 and DPR-64," dated February 8, 2017 (ML17044A004)
- [3] Letter, ENOI to US NRC, Application for Order Consenting to Transfers of Control of Licenses and Approving Conforming License Amendments, Indian Point Nuclear Generating Units 1, 2 and 3, November 21, 2019 (ML19326B953)

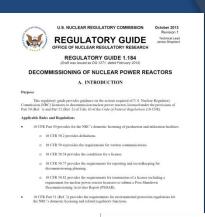
Pursuant to 10 CFR 50.82(a)(4), Holtec Decommissioning International, LLC (HDI) is submitting a Post Stutulowa Decommissioning Activities Report (PSDAR) for the Indian Point Nuclear Generating Units 1, 2 and 3. On January 31, 1996, the US NRC issued an order to authorize decommissioning of IP1 and Amendment No. 45 to License No. DPR-5, which revised the license to possession-only status (Reference 1). By letter dated February 8, 2017, Entergy Nuclear Operations, Inc. (ENOI), Entergy Nuclear Indian Point 2, LLC (ENIP2), and Entergy Nuclear Indian Point 3, LLC (ENIP3) notified the Nuclear Regulatory Commission (NRC) that it would permanently cease power operations at IP2 and IP3 by April 30, 2020 and April 30, 2021, respectively, consistent with the terms of a certain settlement agreement with the State of New York and related parties (Reference 2).

Page 1 of 3



PSDAR Review Process

- Normally, the NRC notices receipt of the PSDAR in the Federal Register and requests public comments
- It also schedules a public meeting to discuss PSDAR & solicit public comments
- In case of Indian Point, Holtec has submitted PSDAR, but the NRC is not actively reviewing it while the license transfer application is under review



U.S. NUCLEAR REGULATORY COMMISSION OFFICE OF NUCLEAR REGULATORY RESEARCH REGULATORY GUIDE

REGULATORY GUIDE 1.185
(Draft was issued as DG-1272, dated December 2012)

STANDARD FORMAT AND CONTENT FOR POST-SHUTDOWN DECOMMISSIONING ACTIVITIES REPORT

A. INTRODUCTION

Purpose

his regulacey guids identifies the type of information that the post-dutafour decommissioning activities report (PSAM) must contain and establishes a standarf format for the PSAM, that the U.S. Nacher Regulatory Commission (NIC) staff considers acceptable. This regulatory guid applies only to holders of the cineses to operate tuncture power reactives mode Parts 60 (Ref. 1) and 5 (Ref. 2) of Trife 10 of the Code of Federal Regulations (10 CFR) and they may use this standard format to prepare PSDARs.

Annileable Rules and Regulations

- 10 CFR Part 50 provides for the NRC's domestic licensing of production and utilization facilitie
 - 0 CFR 50.2 provides definitions.
 - 10 CFR 50.4 provides the requirements for written communications
 - 10 CFR 50 54 movides the conditions for a license
 - 10 CFR 50.75 provides the requirements for reporting and recordiscepts decommissioning planning.
 - 10 CFR 50.82 provides the requirements for termination of a license include

suggestions regarding this guide or development of new guides may be submitted through the NEC's public Web site

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Reactor Decommissioning

The process of removing a reactor facility safely from the operating mode to a permanent shutdown condition and reducing the residual radioactivity to a level that permits the release of the property for unrestricted use and termination of the license

BEFORE



Maine Yankee



AFTER



Decommissioning Options



- DECON Equipment, structures, etc., are removed or decontaminated to a level that permits unrestricted release
- SAFSTOR Plant is placed in a safe, stable condition and maintained in this state until it is subsequently decontaminated to levels that permit unrestricted release

How Long to Decommission?



Under NRC regulations, the process must be completed within 60 years

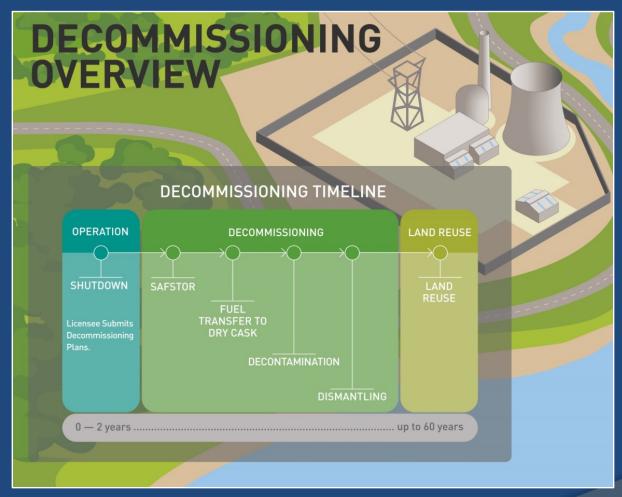
Guiding Principles of Decommissioning



- Protection of plant & workers
- Protection of the public
- Communications & outreach with external stakeholders



Decommissioning Process - Phases



- Before Cleanup
- During Cleanup
- After Cleanup





- Ready the plant for decommissioning
- Move spent nuclear fuel to dry cask storage
- Submit & update PSDAR



- Removal of structures & components
- Soil remediation
- Radioactive waste shipments



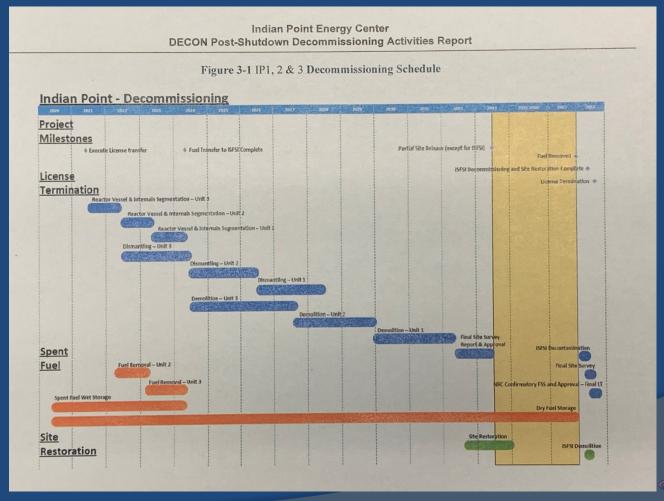
- Site restoration
- NRC license termination
- Spent fuel management-Independent Spent Fuel Storage Installation

Power Reactors in Decommissioning

- 13 units in active decommissioning
- □ 10 units in SAFSTOR
- INDIAN POINT began planning for decommissioning after it notified NRC



Indian Point Decommissioning Schedule & Cost Summary



Oversight Program After Shutdown

- Oversight and monitoring conducted over the entire period of decommissioning process
- Oversight program is described in Inspection Manual Chapter (IMC)
 2561 & 2690

NRC INSPECTION MANUAL

DWM

MANUAL CHAPTER 2561

DECOMMISSIONING POWER REACTOR INSPECTION PROGRAM

2561-01 PURPOSE

To establish the inspection policy and guidance for decommissioning power reactors for the Offices of Nuclear Reactor Regulation (NRR) and Nuclear Material Safety and Safeguards (NMSS).

2561-02 OBJECTIVES

02.01 To obtain information through direct observation and verification of licensee activities to determine whether the power reactor is being decommissioned safely, that spent fuel is safely stored onsite or transferred to another licensed location, and that site operations and license termination activities are in conformance with applicable regulatory requirements, licensee commitments, and management controls.

02.02 To ensure that the licensee's systems and techniques for decommissioning and license termination activities are adequate and in accordance with regulatory requirements. These systems include, in part, management and organization effectiveness; self-assessment, auditing, and corrective actions; design control; maintenance and surveillance; radiation protection; radioactivity measurements; and, effluent controls.

02.03 To identify declining trends in performance and perform inspections to verify that the licensee has resolved the issue(s) before performance declines below an acceptable level.

02.04 To provide for effective allocation of resources for the inspection of Part 50 power reactors following permanent cessation of operation.

2561-03 APPLICABILITY

This program is to be implemented following the certification date for the removal of all nuclear fuel from the reactor vessel (10 CFR 50.82(a)(1)(iii)) and is to continue until license termination.

- 1 -

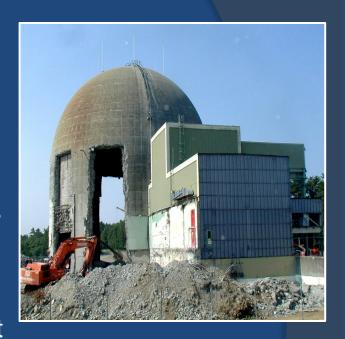
2561-04 DEFINITIONS

Issue Date: 04/14/03

2561

Oversight Program After Shutdown

- Decommissioning inspection program includes both <u>core</u> and <u>discretionary</u> inspections
- Implementation depends on activities being planned or performed.
 - Post-Operation Transition Phase
 - Actively Decommissioning Fuel in Spent Fuel Pool
 - Actively Decommissioning No Fuel in Spent Fuel Pool
 - SAFSTOR Fuel in Spent Fuel Pool
 - > SAFSTOR No Fuel in Spent Fuel Pool
 - Final Surveys under way



How Does Emergency Planning Change?



- Emergencypreparedness remains
- 'All hazards' approach utilized vs. formal preplanned off-site radiological response plans

How will plant security change?



- Security controls remain in place
- Some key features include: intrusion detection and response, assessment of alarms, and off-site assistance, when necessary

What Happens to the Spent Fuel?

- Removed from spent fuel pool
- Stored on-site in dry cask storage systems
- Safety and security programs remain until fuel removed from site



NRC ISFSI Inspection Program



- ☐ Inspectionsperformed according to guidance in InspectionManual Chapter 2690
- Inspections cover all activities related toISFSIs from design to operation

NRC ISFSI Inspection Program (cont'd.)



Is the Spent Fuel Pool Safe?

- Robust structures
- Designed to withstand severe natural events
- Regulated design features & operational practices implemented to maintain fuel in safe condition



How Does the NRC Make
Decisions on Post-Shutdown
Changes?



Public Involvement on Decommissioning

- Public meeting to discuss the decommissioning process and the plant's PSDAR
- NRC staff typically provide briefings at meetings of state/citizen decommissioning advisory panels
- An opportunity for a hearing
- Public meeting on License Termination Plan

NRC References

NRC INSPECTION MANUAL

MANUAL CHAPTER 2690

INSPECTION PROGRAM FOR DRY STORAGE OF SPENT REACTOR FUEL AT PENDENT SPENT FUEL STORAGE INSTALLATIONS AND FOR 10 CFR PART 71 TRANSPORTATION PACKAGINGS



REGULATORY GUIDE

REGULATORY GUIDE 1.185

STANDARD FORMAT AND CONTENT FOR POST-SHUTDOWN DECOMMISSIONING ACTIVITIES

A. INTRODUCTION

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- 10 CFR Part 50 provides for the NRC's domestic licensing of production and utilization facilities

 - o 10 CFR 50.4 provides the requirements for written communications
 - o 10 CFR 50.54 provides the conditions for a license.

 - 10 CFR 50.82 provides the requirements for termination of a license including a requirement for nuclear power reactors licensees to submit a PSDAR.



NRC INSPECTION MANUAL

DECOMMISSIONING POWER REACTOR INSPECTION PROGRAM

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2561-04 DEFINITIONS

*U.S.NRC

Decommissioning Nuclear Power Plants

When a power company decides to close a nuclear power plant permanently, the facility must be decommissioned by safely removing a from service and reducing residual radioactivity to a level that permits release of the operating locuses. The Notalest Regulatory Commission has strate rules governing nuclear power plant docommissioning, nuclear Regulatory Commission has strate rules governing nuclear power plant docommissioning, nuclearing for radioactively continuated plant systems and structures, and nemoval of the radioactive fuel. These requirements protect workers and the public during the entire decommafter the license is terminated.

Licensees may choose from three decommissioning strategies: DECON, SAFSTOR, or ENTOMB.

Under DECON (immediate dismantling), soon after the nuclear facility closes, equipment ctures, and portions of the facility containing radioactive contaminants are removed or ontaminated to a level that permits release of the property and termination of the NRC license

Under SAFSTOR, often considered "deferred dismantling," a nuclear facility is maintained and monitored in a condition that allows the radioactivity to decay; afterwards, the plant is dismantled and the property decontaminated.

Under ENTOMB, radioactive con Under ENTOMB, radioactive contaminants are permanently encased on site in structurally sound material such as concrete. The facility is maintained and monitored until the radioactivity decays to a level permitting restricted release of the property. To date, no NRC-licensed facilities have requested this option.

The licensee may also choose to adopt a combination of the first two choices in which some portions of the facility are dismantled or decontaminated while other parts of the facility are dismantled or decontaminated while other parts of the facility are left in SAFSTOR. The decision may be based on fafetors besides radioactive decay, such as availability of waste disposal sites.



Decommissioning must be completed within 60 years of the plant ceasing operations. A fit ond that would be considered only when necessary to protect public health and safety in accord NRC regulations.



U.S. NUCLEAR REGULATORY COMMISSION

REGULATORY GUIDE

REGULATORY GUIDE 1.184

DECOMMISSIONING OF NUCLEAR POWER REACTORS

A. INTRODUCTION

Commission (NRC) licensees to decommission multar power reactors licensed under the provisions of Part 50 (Ref. 1) and Part 52 (Ref. 2) of Title 10 of the Code of Federal Regulations (10 CFR).

- 10 CFR Part 50 provides for the NRC's domestic licensing of production and utilization facilities

 - requirement for nuclear power reactor licensees to submit a Post-Shutdown Decommissioning Activities Report (PSDAR).
- 10 CFR Part 51 (Ref. 3) provides the requirements for environmental protection regulations for the NRC's domestic licensing and related regulatory functions.



Staff Responses to Frequently Asked **Questions Concerning** Decommissioning of **Nuclear Power Plants**

Final Report



U.S. Nuclear Regulatory Commission Office of Nuclear Reactor Regulation Washington, DC 20555-0001



NUREG-1628



Links for NRC References

- IMC 2561: Decommissioning Power Reactor Inspection Program
- □ RG 1.184: Decommissioning of Nuclear Power Reactors
- NUREG 1628: Staff Responses to FAQs Concerning
 Decommissioning of Nuclear Power Reactors
- □ NRC Backgrounder: Decommissioning of Nuclear Power Plants
- NRC YouTube Video on Decommissioning

Questions

Neil Sheehan@NRC.GOV or 484-471-9233

