AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Stati
DATE October 8, 1984
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

MONTH September, 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	472.8	17	393.4
2	473.3	18	481.5
3	474.3	19	482.0
4	474.7	20	481.1
5	474.8	21	481.2
6	476.2	22	482.2
7	476.9	23	482.5
8	476.8	24	482.6
9	477.7	25	483.2
10	479.3	26	485.0
11	479.4	27	483.8
12	479.2	28	484.1
13	478.3	29	483.2
14	451.8	30	483.7
15	129.3	31	
16	281.6		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

IE24
1/1

OPERATING DATA REPORT

DOCKET NO. 50-285
DATE October 8, 1984
COMPLETED BY T. P. Matthews
TELEPHONE (402)536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: September, 1984
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	6,575.0	96,577.0
12. Number Of Hours Reactor Was Critical	720.0	3,498.1	73,392.0
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.0
14. Hours Generator On-Line	715.8	3,391.5	72,794.0
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,029,861.8	4,743,069.8	91,502,783.5
17. Gross Electrical Energy Generated (MWH)	346,072.0	1,542,392.0	29,859,961.0
18. Net Electrical Energy Generated (MWH)	329,417.6	1,464,826.6	29,544,695.3
19. Unit Service Factor	99.4	51.6	75.4
20. Unit Availability Factor	99.4	51.6	75.4
21. Unit Capacity Factor (Using MDC Net)	95.7	46.6	64.4
22. Unit Capacity Factor (Using DER Net)	95.7	46.6	62.1
23. Unit Forced Outage Rate	0.6	0.5	3.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): N/A

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH September, 1984

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE September 8, 1984
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
84-03	840915	S	4.2	B	4	N/A	XX	XXXXXX	Unit taken off line to perform turbine overspeed tests September 15, 1984 at 0558. The unit was placed back on line the same day at 1010.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending September 1984 .

1. Scheduled date for next refueling shutdown. November 1985

2. Scheduled date for restart following refueling. January 1986

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Maybe

   a. If answer is yes, what, in general, will these be?

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

   c. If no such review has taken place, when is it scheduled?

4. Scheduled date(s) for submitting proposed licensing action and support information. October 1985

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

6. The number of fuel assemblies:
   a) in the core 133 assemblies
   b) in the spent fuel pool 305 "
   c) spent fuel pool storage capacity 729 "
   d) planned spent fuel pool storage capacity May be increased via fuel pin consolidation "

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1996

Prepared by J K Gasper Date October 2, 1984

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

September, 1984
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at 100% power throughout the month of September except for the 15th when the turbine/generator was taken off line for approximately four hours for overspeed testing.

An "URGE" test was completed on September 26, 1984, fulfilling our commitment to MAPP demonstrating maximum net output.

Modification work to the chemical waste lagoons was started in September.

Annual licensed operator simulator training at Combustion Engineering in Windsor, Connecticut, began in September.

An Auxiliary Operator Nuclear was hired during September.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

LER Number	Deficiency
84-016	Exceeding High Alarm Setpoint on Radiation Monitor (RM-062).
84-017	VIAS Actuation (RM-050).
84-018	VIAS Actuation (RM-060).

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure	Description
SP-FAUD-1	Fuel Assembly Uplift Condition Detection.
	This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.
SP-CSF-1	Carbon Steel Fasteners Inservice Testing.
	This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only provided the inspection of components. No systems were modified or removed from service.
SP-SST-1	Site Security Test.
	This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it did not involve safety related equipment and was conducted as designed.
SP-PRCPT-1	Post Refueling Core Physics Testing and Power Ascension.
	This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because this procedure provides for testing required by Station Technical Specifications.
SP-CTPC-1	Core Thermal Power Calculation--NSSS Calorimetric.
	This procedure did not constitute an unreviewed safety question because these tests only involve hand calculation of the NSSS calorimetric power.

System Acceptance Committee Packages for September, 1984:

Package	Description/Analysis
EEAR FC-80-25	Spent Fuel Pool Reracking.
	This modification replaced the Exxon storage racks with PaR storage racks. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (continued)

System Acceptance Committee Packages for September, 1984: (continued)

Package	Description/Analysis
EEAR FC-79-171B	Safety/Relief Valves Piping Analysis and Restraints. This modification provided for the addition of nine pipe restraints and for the reconfiguration of the loop seals of the safety valves shortening the length of piping upstream of the pressurizer safety valves using the same quality materials as the original piping. This modification has no adverse effect on the safety analysis.
EEAR FC-81-23	Bottled Gas Storage Dock on West Side of Auxiliary Building. This modification did not involve safety related equipment, therefore, it has no adverse effect on the safety analysis.
EEAR FC-80-08	Prevention of Unplanned Release of Radioactivity. This modification provided for the installation of concrete curbs and has no adverse effect on the safety analysis.
EEAR FC-84-114	Additional Pipe Restraint on SI-192A Subsystem at Detail E. This modification provided for additional pipe restraint on SI-192A subsystem at detail E and has no adverse effect on the safety analysis.
EEAR FC-83-68	Heater Drain Level Piping Restraints. This modification provided for the addition of six supports on the feedwater heater drain level control valve piping and has no adverse effect on the safety analysis.
EEAR FC-83-15	Heater Drain Pump Recirculation Piping. This modification provided for the relocation of piping and valves and for the installation of supports on lines. This modification has no adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

None

F. CHANGES IN PLANT OPERATING STAFF

Mr. Jeffrey Yeager started in September as Auxiliary Operator Nuclear.

G. TRAINING

Training for the month of September included Emergency Plan training for plant personnel and annual simulator training for licensed personnel. NRC license candidates received oral/written testing as well as walkthroughs and classroom training in preparation for the NRC administered exam. General employee, non-licensed operator, fire brigade and other training was conducted per the annual schedule.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Package	Description/Analysis
Amendment No. 83	This amendment updates the surveillance capsule removal schedule (Table 3-7). The amendment also corrects a typographical error that was made on TS page 2-98 which was issued on August 2, 1984 (Amendment 82). The containment wide range radiation monitors are RM-091 A and B and not RM-019 A and B.
Amendment No. 84	This amendment changes the administrative controls section of the technical specifications to reflect changes to the plant support and plant organizations.

II. MAINTENANCE (Significant Safety Related)

None

W. Gary Gates

W. Gary Gates
Manager
Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

October 12, 1984
LIC-84-345

Mr. Richard C. Deyoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Wasington, DC 20555

Reference: Docket No. 50-285

Dear Mr. DeYoung:

September Monthly Operating Report

Please find enclosed ten (10) copies of the September Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,

R. L. Andrews
Division Manager
Nuclear Production

RLA/TPM/dao

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
NRC File

IE24
1/1