

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-272
 Unit Name Salem # 1
 Date Oct. 10, 1984
 Telephone 609-935-6000
 Extension 4455

Completed by J. P. Ronafalvy

Month September 1984

Day Average Daily Power Level
(MWe-NET)

Day Average Daily Power Level
(MWe-NET)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

P. 8,1-7 R1

8410270259 840930
 PDR ADOCK 05000272
 R PDR

IE24
 111

OPERATING DATA REPORT

Docket No. 50-272
 Date Oct. 10, 1984
 Telephone 935-6000
 Extension 4455

Completed by J. P. Ronafalvy

Operating Status

1. Unit Name	<u>Salem No. 1</u>	<u>Notes</u>
2. Reporting Period	<u>September 1984</u>	
3. Licensed Thermal Power (MWt)	<u>3338</u>	
4. Nameplate Rating (Gross MWe)	<u>1135</u>	
5. Design Electrical Rating (Net MWe)	<u>1090</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1124</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1079</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason	<u>N/A</u>	

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>720</u>	<u>6575</u>	<u>63600</u>
12. No. of Hrs. Reactor was Critical	<u>0</u>	<u>1237.6</u>	<u>34388.8</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>54.5</u>	<u>3088.4</u>
14. Hours Generator On-Line	<u>0</u>	<u>1197.8</u>	<u>32975.7</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0</u>	<u>3800023</u>	<u>99619394</u>
17. Gross Elec. Energy Generated (MWH)	<u>0</u>	<u>1281380</u>	<u>32896480</u>
18. Net Elec. Energy Generated (MWH)	<u>(10750)</u>	<u>1190286</u>	<u>31161598</u>
19. Unit Service Factor	<u>0</u>	<u>18.2</u>	<u>51.8</u>
20. Unit Availability Factor	<u>0</u>	<u>18.2</u>	<u>51.8</u>
21. Unit Capacity Factor (using MDC Net)	<u>0</u>	<u>16.8</u>	<u>45.4</u>
22. Unit Capacity Factor (using DER Net)	<u>0</u>	<u>16.6</u>	<u>45.0</u>
23. Unit Forced Outage Rate	<u>100</u>	<u>71.6</u>	<u>32.8</u>
24. Shutdowns scheduled over next 6 months (type, date and duration of each)	<u>N/A</u>		

25. If shutdown at end of Report Period, Estimated Date of Startup:
10-13-84

26. Units in Test Status (Prior to Commercial Operation):

	<u>Forecast</u>	<u>Achieved</u>
Initial Criticality	<u>9/30/76</u>	<u>12/11/76</u>
Initial Electricity	<u>11/1/76</u>	<u>12/25/76</u>
Commercial Operation	<u>12/20/76</u>	<u>6/30/77</u>

UNIT SHUTDOWN AND POWER REDUCTIONS
 REPORT MONTH September 1984

Docket No. 50-272
 Unit Name Salem No.1
 Date Oct. 10, 1984
 Telephone 609-935-6000
 Extension 4455

Completed by J.P. Ronafalvy

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
84-176	7-9	F	720	A	4	-	HA	GENERA	Generator Liquid Cooling System
84-178	9-5	F	133.0	A	4	-	WA	VALVEX	Nuclear Service Water Valves
84-180	9-10	F	483.0	A	4	-	RB	CRDRVE	Nuclear Other Control Rod Drive Problems

1
 F: Forced
 S: Scheduled

2 Reason
 A-Equipment Failure-explain
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & Licensing Exam
 F-Administrative
 G-Operational Error-explain
 H-Other-explain

3 Method
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuation of
 Previous Outage
 5-Load Reduction
 9-Other

4 Exhibit G
 Instructions
 for Prepara-
 tion of Data
 Entry Sheets
 for Licensee
 Event Report
 (LER) File
 (NUREG 0161)

5 Exhibit 1
 Salem as
 Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH September 1984

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: October 10, 1984
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1413	Reactor Coolant	Replacement of reactor coolant wide range RTD's with environmentally qualified elements.
1EC-1514	Incore Thermocouples	Change method of connecting field cable to incore thermocouples. Also cable itself will be replaced.
1EC-1622	Station Air	Assign a new number to 1SA323 (A-3) valve. This valve is located on the 4" line to the Condensate Polishing System demineralizer.
1EC-1628	Reactor Protection	Rewire shunt trip coil of "A", (B) reactor trip breakers to open breakers upon a SSPS "A" and/or SSPS "B" train trip signal.
1EC-1652	Safety Injection	Replace the existing 2" accumulator level tap isolation valves with two 1" Yarway valves.
1EC-1762	Miscellaneous Condensate Condenser Sprays	Upgrade the condenser spray headers in the lower exhaust necks of condenser shells #11, 12, 13. Modify hangers to prevent pipe breakage due to high thermal stress loading.
1EC-1860	Charging/Safety Injection Pumps Lube Oil Coolers	Replace the existing lube oil coolers on the Nos. 11 & 12 C/SI pumps with coolers of similar design characteristics but upgraded materials of constructions.

MAJOR PLANT MODIFICATIONS
REPORT MONTH September 1984

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: October 10, 1984
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

<u>*DCR NO.</u>	<u>PRINCIPLE SYSTEM</u>	<u>SUBJECT</u>
1EC-1945	Containment Ventilation	Install spring assembly in the backdraft damper mechanism of the Containment Fan Coil Units 11, 12, 13, 14 and 15.
1SC-0025A	Fuel Transfer System	Modify Fuel Transfer System (WRAPS 13) to improve the operation of original WRAPS 13 modification.
1SC-1043	Circulating Water	Change anchor bolts on the following equipment: 11, 12, 13 and 14 Screen Wash Pumps and their associated automatic strainers. Also change anchor bolts on #1 and #3 Circulator Bearing Lube Pumps and their associated automatic strainers.
1SC-1287A	Containment Sump Pumps	Modify control circuit to prevent both pumps from running prior to reaching Hi-Hi level due to logic error in pump alternating circuit.
1SC-1357	Containment Ventilation	Replace containment fan coil unit suction expansion joints with ones of improved design.
1SC-1449	Reactor Coolant Pump Motor Oil Lift Pump	Provide an alternate oil lift pump for the reactor coolant pump oil lift pump.

MAJOR PLANT MODIFICATIONS
REPORT MONTH SEPTEMBER 1984

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: October 10, 1984
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO. SAFETY EVALUATION 10 CFR 50.59

- 1EC-1413 This change involves a direct replacement of existing equipment. The system will not change functionally. No unreviewed safety or environmental questions are involved.
- 1EC-1514 This modification does not alter reliability of safety related systems. This change introduces a superior method of connecting cables as well as introducing a cable that has been environmentally qualified. The cable function remains unchanged. No unreviewed safety or environmental questions are involved.
- 1EC-1622 This change only specifies the correct identification of a valve. No unreviewed safety or environmental questions are involved.
- 1EC-1628 This change does not alter any plant process or discharge. No unreviewed safety or environmental questions are involved.
- 1EC-1652 This change is a mechanical change that does not alter the installed controls equipment function. No unreviewed safety or environmental questions are involved.
- 1EC-1762 This change in the condenser spray header hangers will not affect station operations or plant effluent. No unreviewed safety or environmental questions are involved.
- 1EC-1860 This change involves the installation of lube oil coolers with upgraded materials but with the same design specifications. These new coolers are designed to the 1980 Edition of the ASME Code - Section III, Class 3 1981 on the Service Water side and designed to Division I of Section VIII of the 1980 Edition of the ASME Code on the lube oil side. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

- 1EC-1945 The new spring assembly does not prevent opening of the damper blades when the fan operates. Failure of the spring does not jeopardize the operation of the backdraft damper. Also, it does not affect any presently performed Safety Analysis nor does it create any new safety hazards. No unreviewed safety or environmental questions are involved.
- 1SC-0025A This change does not affect any performed safety analysis nor does it create any new safety hazards. No unreviewed safety or environmental questions are involved.
- 1SC-1043 This change is a replacement in kind except that the anchor is made of an upgraded material designed to resist chloride corrosion. No unreviewed safety or environmental questions are involved.
- 1SC-1287A This change has been reviewed for all realistic failure modes and it has been determined that no unreviewed safety or environmental questions are involved.
- 1SC-1357 This change involves the replacement of an expansion joint in the CFCU's. The replacement is of an improved design. It does not change the original function of the expansion joint. No unreviewed safety or environmental questions are involved.
- 1SC-1449 This modification is non-safety related. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 1

WO NO DEPT UNIT EQUIPMENT IDENTIFICATION

0099103915

SMD

1

100' ELEVATION AIRLOCK

FAILURE DESCRIPTION: EXTERIOR DOOR WILL NOT OPEN

CORRECTIVE ACTION: READJUSTED INTERLOCK ROD

0099099233

SMD

1

VALVE 12MS167

FAILURE DESCRIPTION: ALL THE OIL IN THE HYDRAULIC SYSTEM OF VALVE
BLEW OUT

CORRECTIVE ACTION: BLED AND REFILLED RESERVOIR

939520-7 OD

1

NO. 13 REACTOR COOLANT PUMP - SEAL LEAKOFF

FAILURE DESCRIPTION: LOW FLOW INDICATED AS COMPARED WITH OTHER PUMPS

CORRECTIVE ACTION: NO. 13 RCP SEALS REPLACED

0099026066

SMD

1

OHA C-28 SPENT FUEL PIT LOW ALARM

FAILURE DESCRIPTION: ALARM DOES NOT COME IN WITH ACTUAL LOW LEVEL

CORRECTIVE ACTION: REPLACED ALARM DEVICE

0099103354

SMD

1

1RC43 - REACTOR HEAD VENT

FAILURE DESCRIPTION: VALVE IS LEAKING THROUGH

CORRECTIVE ACTION: CLEANED INTERNAL PARTS

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
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0099103346

SMD

1

IRC41 - REACTOR HEAD VENT

FAILURE DESCRIPTION: VALVE LEAKS THROUGH

CORRECTIVE ACTION: CLEANED INTERNAL PARTS

94170

PD

1

1PS1

FAILURE DESCRIPTION: VALVE DOES NOT OPEN FROM CONTROL ROOM

CORRECTIVE ACTION: REPLACED DIAPHRAGM ON VALVE; REPLACED MAN/AUTO STATION 1HC455

936925

PD

1

1PS3

FAILURE DESCRIPTION: WITH A 60% DEMAND SIGNAL THE CLOSE LITE DOES NOT EXTINGUISH AND THE VALVE DOES NOT APPEAR TO OPEN.

CORRECTIVE ACTION: REPLACED DIAPHRAGM; REPLACED POSITIONER RELAYS; REPLACED AIR REGULATOR; REPLACED LEAKING CONOFLOW REGULATOR TO TRANSDUCER

943632

SMD

1

13GB3

FAILURE DESCRIPTION: VALVE FAILED LEAK RATE TEST

CORRECTIVE ACTION: CLEANED STEM; REPLACED GASKET.

84-09-24-052-1

SMD

1

1C DIESEL

FAILURE DESCRIPTION: SWITCH ON THE 125V DC CONTROL POWER FOR THE 4KV DIESEL GEN OUTPUT BROKE OFF

CORRECTIVE ACTION: REPLACED SWITCH

WO NO DEPT UNIT EQUIPMENT IDENTIFICATION

84-08-30-001-6
SMD

1 1B DIESEL GENERATOR

FAILURE DESCRIPTION: D/G AIR STARTING VALVES FAILED TO ATTAIN THE
REQUIRED 50 RPM DURING SURVEILLANCE TESTING

CORRECTIVE ACTION: REPLACED STARTING MOTOR AIR SOLENOID VALVE
PILOT AIR START VALVES POSITIONS 1 AND 4

0099128578
SMD

1 VITAL HEAT TRACING

FAILURE DESCRIPTION: FIRE IN BIT HEAT TRACE; DE-ENERGIZED AT OPENING
BREAKER IN HEAT TRACE PANELS 1A2 AND 1B2

CORRECTIVE ACTION: HT622 PRIMARY AND SECONDARY FOUND SHORTED TO
THE PIPE; REPLACED BURNT ENDS

84-07-04-340-1
SMD

1 11CFCU

FAILURE DESCRIPTION: FAN TRIPPED ON OVERLOAD OF LOWER SPEED BREAKER
WHEN ATTEMPTING TO START IN HIGH SPEED

CORRECTIVE ACTION: REPLACED STARTING RELAY FOR HIGH SPEED BREAKER

0099103940
SMD

1 NO. 12 CFCU MOTOR

FAILURE DESCRIPTION: HIGH WINDING AND BEARING TEMPERATURES

CORRECTIVE ACTION: REPLACED MOTOR

0099104768
SMD

1 NO. 12 CFCU

FAILURE DESCRIPTION: SERVICE WATER LEAK DOWNSTREAM OF 12SW71 MOTOR
COOLER SW OUTLET VALVE

CORRECTIVE ACTION: INSTALLED NEW SPOOL PIECE AND FLANGE

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-09-03-033-1	SMD	1	NO. 12 CONTAINMENT SUMP PUMP
			FAILURE DESCRIPTION: PUMP RUNS CONTINUOUSLY WHEN THE BREAKER IS CLOSED
			CORRECTIVE ACTION: NOS. 11 AND 12 SUMP PUMP STRAINERS WERE FOUND CLOGGED AND SUBSEQUENTLY WERE CLEARED; REPAIRED BROKEN LUG ON INDICATION LEAD
84-07-30-042-1	SMD	1	NO. 11 CHILLED WATER PUMP
			FAILURE DESCRIPTION: PUMP TRIPS AFTER ONE HOUR RUN
			CORRECTIVE ACTION: CLEANED PUMP AND INSTALLED NEW SEALS
84-09-08-017-6	SMD	1	14MS178 (CHECK VALVE)
			FAILURE DESCRIPTION: VALVE HAS BONNET LEAK
			CORRECTIVE ACTION: REPLACED BONNET GASKET AND USED FURMANITE COMPOUND
84-09-13-020-3	SMD	1	1SJ79
			FAILURE DESCRIPTION: BODY TO BONNET LEAK
			CORRECTIVE ACTION: REMOVED EXTRA GASKET AND REPLACED CORRODED STUD
951382	OD	1	VALVE 1SJ222 - 1SJ223
			FAILURE DESCRIPTION: VALVE LEAKS BY ITS SEAT
			CORRECTIVE ACTION: CUT OUT VALVES AND WELDED IN NEW VALVES AS PER DCR 1ED-0351

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-08-25-017	SMD	1	1SJ4
			FAILURE DESCRIPTION: NO CLOSED INDICATION IN THE CONTROL ROOM
			CORRECTIVE ACTION: CLEANED CONTACTS
0099026058	SMD	1	NO. 12 BORIC ACID TRANSFER PUMP
			FAILURE DESCRIPTION: SEAL LEAK
			CORRECTIVE ACTION: REPAIRED PUMP
84-09-04-090-5	SMD	1	NO. 16 SERVICE WATER STRAINER
			FAILURE DESCRIPTION: SHEAR PIN ON STRAINER SHEARED
			CORRECTIVE ACTION: REPLACED BOTTOM SHOES AND SHEAR KEY
0099101696	SMD	1	11SW20
			FAILURE DESCRIPTION: VALVE INOPERABLE
			CORRECTIVE ACTION: CHANGED TORQUE SWITCH SETTING
84-09-08-043-5	SMD	1	11SW23
			FAILURE DESCRIPTION: CLOSED INDICATION DOES NOT CHANGE WHEN CYCLING THE VALVE
			CORRECTIVE ACTION: REPLACED AUXILIARY RELAY

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY
September 1984

SALEM UNIT NO. 1

Unit No. 1 remained shutdown as the fifth refueling outage continues. The Unit began the month in Mode 5. Preparations were completed for Unit operational Mode changes from Mode 5 to Mode 4 and from Mode 4 to Mode 3. The Unit entered Mode 4 on 9/02/84 and on 9/07/84 entered Mode 3. Limitorque valve modifications were completed on 9/02/84. A total of thirty-eight (38) valves were inspected of which thirteen (13) required modification. No. 12 CFCU motor has been replaced. No. 13 Service Water Strainer has been repaired. Valve 13SW20 was reinstalled after repairs. During Rod Drop Tests, Control Rod #2B1 could not be raised past indicated position of 187 steps. On 9/12/84 at 2311 hours a Unit cooldown was commenced to investigate and effect repairs for the problem with Control Rod #2B1. The Unit entered Mode 6 on 9/17/84. Investigation of the problem with Control Rod #2B1 revealed that its tophat was rotated 90° out of position. In addition to correcting this problem, the remaining sixty (60) tophats were also checked. Control Rod #2D2's tophat was also found rotated. During the Split Pin Replacement work performed by Westinghouse, the tophats were removed for each control rod guide tube in the upper internals to facilitate replacement of the split pins. Apparently, during the process, the two top hats in question were inadvertently repositioned. Following discovery, the tophats were positioned correctly. In the process of replacing the Reactor Vessel Head, the thermocouple column in core location L-1 was slightly bent. Following repairs to the thermocouple column the Reactor Vessel Head was installed on 9/23/84. On 9/27/84 the Unit entered Mode 5. During the week of 9/24/84, a CVCS outage was initiated and completed. As of 9/30/84, the conoseals were installed and the thermocouple connections were being secured.

REFUELING INFORMATION

COMPLETED BY: J. Ronafalvy DOCKET NO.: 50-272
 UNIT NAME: Salem 1
 DATE: October 10, 1984
 TELEPHONE: 609/935-6000
 EXTENSION: 4455

Month September 1984

1. Refueling information has changed from last month:
 YES _____ NO X
2. Scheduled date for next refueling: February 22, 1986
3. Scheduled date for restart following refueling: May 4, 1986
4. A) Will Technical Specification changes or other license amendments be required?
 YES _____ NO _____
 NOT DETERMINED TO DATE 10/1/84
- B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
 YES _____ NO X
 If no, when is it scheduled? January 1986
5. Scheduled date(s) for submitting proposed licensing action:
January 1986 if required
6. Important licensing considerations associated with refueling:
NONE

7. Number of Fuel Assemblies:
 A) Incore 193
 B) In Spent Fuel Storage 296
8. Present licensed spent fuel storage capacity: 1170
 Future spent fuel storage capacity: 1170
9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: September 2001



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

October 10, 1984

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of September 1984 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

JR:sbh

cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
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