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U S Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

> PRAIRIE ISLAND NUCLEAR GENERATING PLANT Docket Nos. 50-282 License Nos. DPR-42 50-306 DPR-60

Reporting of Steam Generator 'nspection Results

The propose of this letter is to clarify when the detailed results of the No. 12 steam generator eddy current inspection, completed October 2, 1992, will be provided to the NRG. Our letter, dated Oct 1 14, 1992, "Steam Generator Tube Plugging and Sleeving", mistakenly reported the inspection of Unit 1 steam generator No. 12, which was completed on October 2, 1992, as an inservice inspection. This steam generator tube inspection was performed in response to increased Unit 1 primary to secondary leakage. This leakage did not exceed the limits of Technical Specification 3.1.C.2.e before the unit was shutdown, and therefore, the inservice inspection requirements of Technical Specification 4.12 were not implemented.

Prairie Island Technical Specification 4.12.E.2 requires that the results of steam generator tube inservice inspections, not associated with a refueling outage, be submitted within 90 days of the completion of the inspection. In accordance with these requirements, our October 14, 1992 letter stated that expanded information on the No. 12 steam generator inspection would be provided in an inservice inspection report within 90 days of the completion of the inspection. However, because this ispection was not performed in response to Technical Specification 4.12, or as part of the normal scheduled inservice inspection program, it was not considered an inservice inspection. Rather, it was considered an engineering surveillance intended to identify sources of steam generator tube leakage.

Because the requirements of Technical Specification 4.12.E.2 do not apply to this inspection, an expanded inservice inspection report will not be provided within 90 days of the completion of the inspection as stated in our October 14, 1992 letter. Rather, the expanded information on the No. 12 steam generator inspection will be included with the summary reports f ASME Code Section XI inspections to be submitted within 90 days of the completion of the current two unit outage.

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We apologize for any inconvenience that may result from this change. Please contact us if you have any questions with regards to the reporting of the results of the No. 12 steam generator inspection.

Thomas M Parker

Manager

Nuclear Support Services

c: Regional Administrator - Region III, NRC Senior Resident Inspector, NRC NRR Project Manager, NRC J E Silberg