

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT II	DOCKET NUMBER (2) 0 5 0 0 0 3 6 6 1	PAGE (3) OF 0 2
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TITLE (4)
Unplanned Reactor Scram

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 9	2 1	8 4	8 4	0 2 1	0 0	1 0	1 2	8 4			0 5 0 0 0
DOCKET NUMBER(S) 0 5 0 0 0											

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)								73.71(b)
POWER LEVEL (10) 1 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(c)					
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)					
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)						
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)						
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)

NAME T. L. Elton, Acting Superintendent of Regulatory Compliance	TELEPHONE NUMBER 9 1 2 3 6 7 + 7 8 5 1
AREA CODE	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
X	S B	F S V A #	9 9	Y					

SUPPLEMENTAL REPORT EXPECTED (14)	EXPECTED SUBMISSION DATE (15)	MONTH DAY YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 09/21/84, at approximately 1701 CDT, with the reactor mode switch in the run position and reactor power at 2440 MWT (approximately 100% power), Unit 2 received a reactor scram on MSIV's not fully open subsequent to the Drywell Pneumatic System's Nitrogen Inlet Valve (2P70-F005) closing on a high system flow isolation signal due to a nitrogen leak on the pneumatic system solenoid valve for the inboard MSIV 2B21-F022D.

No actual or potential safety consequences or implications resulted from this event. This event had no impact on any other Unit 1 system or on Unit 2. The health and safety of the public were not affected by this event. This is a non-repetitive event; however, the last reactor scram is referenced in LER 50-366/1984-020.

This event was the result of a nitrogen leak on the pneumatic system's leak on the pneumatic system's solenoid valve for inboard MSIV 2B21-F022D.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) EDWIN I. HATCH, UNIT II	DOCKET NUMBER (2) 0 5 0 0 0 3 6 6 8 4	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
			0 2 1	0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day LER is required by 10CFR50.73 (a)(2)(iv) because of the reactor scram and Engineered Safety Feature Actuation.

On 09/21/84, at approximately 1701 CDT, with the reactor mode switch in the run position and reactor power at 2440 Mwt (approximately 100% power), Unit 2 received a reactor scram on MSIV's not fully open subsequent to the drywell pneumatic system's nitrogen inlet valve (2P70-F005) isolating on a high system flow isolation signal. An investigation of 2P70-F005's closing revealed that the valve operated correctly to isolate a nitrogen leak on inboard MSIV 2B21-F022D's solenoid valve. The nitrogen leak was determined as being the result of a blown gasket on 2B21-F022D's solenoid valve.

The transient proceeded smoothly. Reactor water level increased to +80" (reference instrument zero) via the reactor feedpumps, which tripped at +58". The HPCI and RCIC turbines which were in their normal standby configuration also received a trip on high water level. Reactor water level remained steady and reactor pressure increased to approximately 1049 psig. At that time safety relief valve (SRV) "M" was opened per the "ANNUNCIATOR RESPONSE PROCEDURES" (HNP-2-2001) and low low set SRV's "B", "D", "F", and "G" lifted and reduced reactor pressure to 840 psig. Reactor water level then decreased to - 20". RCIC and HPCI was manually started; however, only RCIC was used to inject water to the reactor vessel. HPCI was placed in full flow test to the condensate storage tank to control reactor pressure. No other ECCS systems auto started nor was there a need to manually start any other ECCS system for level control.

Reactor water level did reach the level of a group 2 isolation. Also, a group 1 isolation was received on low vacuum.

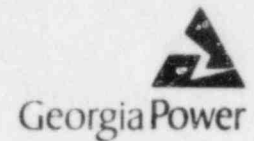
On 09/21/84, between approximately 1701 CDT and 1730 CDT, RCIC tripped 3 times for no known reason. However, RCIC was successfully restarted and was used to control reactor water level until it was manually shutdown at approximately 2020 CDT. Per the control room level indicators, the highest reactor water level was +80" and the lowest reactor water level was -25".

No actual or potential safety consequences or implications resulted from this event. This event had no impact on any other Unit 1 system or on Unit 2. The health and safety of the public were not affected by this event. This event is non-repetitive; however, the last reactor scram is referenced in LER 50-366/1984-020.

The cause of this event is component failure. The pneumatic system solenoid valve for MSIV 2B21-F002D was repaired by replacing a blown gasket. The valve was functionally tested satisfactorily and returned to service on 09/22/84.

RCIC was functionally tested satisfactorily per the "RCIC PUMP OPERABILITY" procedure (HNP-2-3405) and returned to service on 09/23/84.

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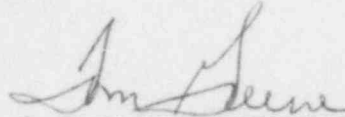
Edwin I. Hatch Nuclear Plant

October 12, 1984
GM-84-902

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1984-021. This report is required by 10CFR 50.73(a)(2)(iv).



for H. C. Nix
General Manager

HCN/TLE/vlz

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