



Northern States Power Company

414 Nicollet Mall  
Minneapolis, Minnesota 55401-1927  
Telephone (612) 330-5500

October 29, 1992

Report required by  
10 CFR Part 50, Section 50.73

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT  
Docket No. 50-263 License No. DPR-22

Automatic Transfer of High Pressure Coolant Injection Suction  
From Condensate Storage Tanks to Suppression Pool  
Caused by Inadvertent Jarring of Level Switch

The voluntary Licensee Event Report for this occurrence is attached. Please contact us if you require further information.

Thomas M Parker  
Manager  
Nuclear Support Services

c: Regional Administrator - III NRC  
Sr Resident Inspector, NRC  
NRR Project Manager, NRC  
State of Minnesota,  
Attn: Kris Sanda

Attachment

030157

9211040070 921029  
PDR ADOCK 05000263  
S PDR

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20545, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

|   |   |                             |
|---|---|-----------------------------|
| FACILITY NAME (1)<br><b>MONTICELLO NUCLEAR GENERATING PLANT</b> | DOCKET NUMBER (2)<br><b>0 5 0 0 0 2 6 3 1</b> | PAGE (3)<br><b>1 OF 0 3</b> |
|---|---|-----------------------------|

TITLE (4) **Automatic Transfer of High Pressure Coolant Injection Suction From Condensate Storage Tanks to Suppression Pool Caused by Inadvertent Jarring of Level Switch**

| EVENT DATE (5) |     |      | LER NUMBER (6) |                   |                 | REPORT DATE (7) |     |      | OTHER FACILITIES INVOLVED (8) |   |                  |   |   |   |   |  |  |           |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|---|------------------|---|---|---|---|--|--|-----------|
| MONTH          | DAY | YEAR | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH           | DAY | YEAR | FACILITY NAMES                |   | DOCKET NUMBER(S) |   |   |   |   |  |  |           |
| 1              | 0   | 3    | 9              | 2                 | 0               | 1               | 4   | 0    | 0                             | 1 | 0                | 2 | 9 | 9 | 2 |  |  | 0 5 0 0 0 |
|                |     |      |                |                   |                 |                 |     |      |                               |   |                  |   |   |   |   |  |  | 0 5 0 0 0 |

OPERATING MODE (9) **N**

POWER LEVEL (10) **1 0 0**

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)

|                   |                          |                  |                          |                      |                                     |  |                          |
|-------------------|--------------------------|------------------|--------------------------|----------------------|-------------------------------------|--|--------------------------|
| 20.402(b)         | <input type="checkbox"/> | 20.405(c)        | <input type="checkbox"/> | 50.73(a)(2)(iv)      | <input checked="" type="checkbox"/> | 73.71(b)   | <input type="checkbox"/> |
| 20.405(a)(1)(i)   | <input type="checkbox"/> | 50.36(e)(1)      | <input type="checkbox"/> | 50.73(a)(2)(v)       | <input type="checkbox"/>            | 73.71(c)   | <input type="checkbox"/> |
| 20.405(a)(1)(ii)  | <input type="checkbox"/> | 50.36(e)(2)      | <input type="checkbox"/> | 50.73(a)(2)(vi)      | <input type="checkbox"/>            | OTHER (Specify in Abstract below and in Text, NRC Form 366A) | <input type="checkbox"/> |
| 20.405(a)(1)(iii) | <input type="checkbox"/> | 50.73(a)(2)(i)   | <input type="checkbox"/> | 50.73(a)(2)(vii)(A)  | <input type="checkbox"/>            |  | <input type="checkbox"/> |
| 20.405(a)(1)(iv)  | <input type="checkbox"/> | 50.73(a)(2)(ii)  | <input type="checkbox"/> | 50.73(a)(2)(viii)(B) | <input type="checkbox"/>            |  | <input type="checkbox"/> |
| 20.405(a)(1)(v)   | <input type="checkbox"/> | 50.73(a)(2)(iii) | <input type="checkbox"/> | 50.73(a)(2)(ix)      | <input type="checkbox"/>            |  | <input type="checkbox"/> |

LICENSEE CONTACT FOR THIS LER (12)

|   |  |
|---|--|
| NAME<br><b>Ray Dennis, Shift Supervisor</b> | TELEPHONE NUMBER   |
|   | AREA CODE: <b>6 1 1 2</b> NUMBER: <b>2 9 5 1 1 1 4 1 0</b> |

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC |
|-------|--------|-----------|--------------|-------------------|-------|--------|-----------|--------------|-------------------|
|       |        |           |              |                   |       |        |           |              |                   |
|       |        |           |              |                   |       |        |           |              |                   |

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

|                               |       |     |      |
|-------------------------------|-------|-----|------|
| EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|                               |       |     |      |

ABSTRACT (Limit to 1400 spaces; i.e., approximately fifteen single-space typewritten lines) (16)

While investigating the cause of a small fluctuation in the Reactor Core Isolation Cooling system suction pressure, an operator inadvertently dropped his hard hat on a Condensate Storage Tank level switch. The switch activated a Condensate Storage Tank low level alarm and a transfer of the Reactor Core Isolation Cooling and High Pressure Coolant Injection suction from the Condensate Storage Tanks to the Suppression Pool. The cause of this event was an inadvertently dropped hard hat on sensitive instrumentation. A contributing cause was the location of the instrumentation. The alarm immediately cleared and the systems were returned to the normal standby line-up. The operations staff has been cautioned concerning work around sensitive instrumentation and the area of the level switches has been posted alerting workers that sensitive instrumentation is located in the area.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

| FACILITY NAME (1)                   | DOCKET NUMBER (2) | LER NUMBER (5) |                   |                 | PAGE (3) |
|-------------------------------------|-------------------|----------------|-------------------|-----------------|----------|
|                                     |                   | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER |          |
| MONTICELLO NUCLEAR GENERATING PLANT | 05000 263         | 92             | 014               | 00              | 2 OF 3   |

TEXT (if more space is required, use additional copies of NRC Form 366A) (17)

DESCRIPTION

At 0230 hours, on October 3, 1992, with the plant operating at 100% of rated thermal power the High Pressure Coolant Injection (EII System: BJ) system and Reactor Core Isolation Cooling (EII System: BN) system suction valves from the Suppression Pool opened and the suction valves from the Condensate Storage Tanks (EII System: KA) closed.

While investigating a small fluctuation in the Reactor Core Isolation Cooling system suction pressure observed during surveillance testing, an operator inadvertently dropped his hard hat on a Condensate Storage Tank level switch (EII Component: LS). The level switch actuated a Condensate Storage Tank "Low Level Alarm" in the Control Room (EII System: NA) and initiated a transfer of the Reactor Core Isolation Cooling and High Pressure Coolant Injection suctions from the Condensate Storage Tank to the Suppression Pool. MO-2100, Reactor Core Isolation Cooling Inboard Torus Suction, MO-2101, Reactor Core Isolation Cooling Outboard Torus Suction, MO-2061, High Pressure Coolant Injection Pump Suction (Inner) Isolation, and MO-2062, High Pressure Coolant Injection Pump Suction (Outer) Isolation, valves (EII Components: 20) opened. MO-2102, Reactor Core Isolation Cooling Condensate Storage Suction, and MO-2061, High Pressure Coolant Injection Suction From Condensate Storage, valves closed.

The control room alarm cleared immediately and the systems were returned to normal standby line-up.

The High Pressure Coolant Injection system is an Engineered Safety Feature (ESF) and actuation is reportable under 10 CFR Part 50 Section 50.73(a)(2)(iv).

CAUSE

The cause of this event was an inadvertent dropping of a hard hat onto a level switch. The unusual location of the Condensate Storage Tank level gauge, which is about two feet off the floor, was a direct contributing factor. The operator needed to bend down to view the reading on the gauge. This caused his hard hat to fall from his head and strike the level switch which is located at floor level.

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TEXT (if more space is required, use additional copies of NRC Form 366A) (17)

ANALYSIS

Realignment of the High Pressure Coolant Injection and Reactor Core Isolation Cooling systems suction to the Suppression Pool does not affect system operability.

Based on the ability of both systems to perform their function there were no consequences to the health and safety of the public.

CORRECTIVE ACTION

1. The High Pressure Coolant Injection and Reactor Core Isolation Cooling systems were returned to normal line up with suction from the Condensate Storage Tanks.
2. A visual inspection of the Condensate Storage Tank level switch was conducted.
3. The operations staff has been cautioned about the sensitivity of instrumentation and the care which must be taken when viewing these devices.
4. A sign has been posted at the location of the level switch alerting individuals that sensitive instrumentation is located in the area.

ADDITIONAL INFORMATION

Failed Component Identification:

None

Previous Similar Event:

There are four identified similar events. Licensee Event Reports 87-004, 87-009, 89-038, and 90-017. These events are all similar in that they all involve inadvertent jarring of sensitive instrumentation. The corrective actions for these events included revision of procedures, labeling of plant instrumentation, and instruction to plant personnel when working near sensitive instrumentation. These corrective actions did not prevent this event because the Condensate Storage Tank level switches had not been identified as sensitive instrumentation.