NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION (7.77) LICENSEE EVENT REPORT CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 
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0 CON'T REPORT L 6 0 5 0 0 3 2 1 0 1 1 1 5 8 3 8 1 0 0 5 8 4 9 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80 0 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10 On 11/15/83, plant personnel were notified that during a safety relief 0 2 valve (SRV) bench test at Wyle Laboratory, 8 of 11 SRV's (1B21-F013 A.B. 0 3 D,F,H,J,K & L) did not lift within the specified setpoint plus or minus 0 4 1% as required by Tech. Specs. section 2.2.A.1.b. Plant operation was not affected. The health and safety of the public were not affected by this 0 6 event. This is a repetitive event as last reported on LER 50-321/1982-099 0 7 in respect to valves 1B21-F013 F and H. CAUSE COMP COMPONENT CODE SUBCODE IAILIVIEIX (14 B (13) OCCURRENCE REVISION REPORT NO. EVENT YEAR CODE TYPE LER/AD NO. [8]3 REPORT 1 0 5 0 3 X 1 | NUMBER NPRD-4 FORM SUB EFFECT COMPONENT MANUFACTURER SUBMITTED PRIME COMP FUTURE METHOD HOURS SUPPLIER N (24) T 0 2 0 N (25 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The valves failed due to a stuck pilot disc, and/or lack of proper 1 0 clearance of the pilot rod in the labyrinth seal area. The valves were disassembled and cleaned. The labyrinth seal area was inspected and all discrepancies were corrected. The valves were reassembled and functionally tested satisfactorily. 1 4 90 METHOD OF DISCOVERY FACILITY OTHER STATUS (30) % POWER DISCOVERY DESCRIPTION (32) 010 B (31 Surveillance Test ACTIVITY CONTENT HO. AMOUNT OF ACTIVITY (35 LOCATION OF RELEASE (36) OF RELEASI EASED (33) 2 (34) NA NA XPOSURES DESCRIPTION (39) 37) Z (38 NA 前( PERSONNEL INJURIES DESCRIPTION (41) 0 0 (40) NA JA10260191 841005 DR ADOCK 0500032 OSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION NA Z (42) PUBLICITY DESCRIPTION (45) NRC USE ONLY N 1(44) NA NAME OF PREPARER \_\_\_\_ S. B. Tipps (912)367-7851 PHONE ...

# NARRATIVE REPORT FOR LER 50-321/1983-105, Rev. 1 UPDATE REPORT - PREVIOUS REPORT DATE 12/09/83

LICENSEE : GEORGIA POWER COMPANY FACILITY NAME : EDWIN I. HATCH DOCKET NUMBER : 50-321

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### Tech. Specs. section(s) which requires report:

This 30-day LER is required by Tech. Specs. section 6.9.1.9.b due to the event's showing that the unit was not meeting the requirements of Tech. Specs. section 2.2.A.1.b.

### Plant conditions at the time of the event(s):

On 11/15/83, the plant was in cold shutdown for a fuel reconstitution outage when this event occurred.

# Detailed description of the event(s):

On 11/15/83, plant personnel were notified that during a safety relief valve (SRV) bench test at Wyle Laboratory, 1B21-F013 A, B, D, F, H, J, K, & L lifted at 1110, 1152, 1112, 1062, 1104, 1126, 1102, and 1065 PSIG respectively. Thus the valves did not lift within the specified setpoint plus or minus 1% as required by Tech. Specs. section 2.2.A.l.b.

### Consequences of the event(s):

Plant operation was not affected by this event. The health and safety of the public were not affected by this event.

### Status of redundant or backup subsystems and/or systems:

Safety relief valve's 1B21-F013 C, E, and G remained operable during this event.

#### Justification for continued operation:

The safety relief valves were repaired and functionally tested satisfactorily prior to reactor startup as required by Tech. Specs. section 3.6.H.1.

# If repetitive, number of previous LER:

This is a repetitive event with respect to valves 1B21-F013 F and H as last reported on LER 50-321/1982-099.

Narrative Report for LER 50-321/1983-105, Rev. 1 Page Two

### Impact to other systems and/or Unit:

This event had no effect on any other Unit 1 system. Target Rock Model 7567F safety relief valves are utilized by both Unit 1 and Unit 2 as main steam safety relief valves. However, this event did not affect Unit 2.

# Cause(s) of the event(s):

The valve failures were the result of a stuck pilot disc, and/or lack of proper clearance of the pilot rod in the labyrinth seal area.

The Target Rock model 7567F safety relief values used on Units 1 and 2 are only certified to a  $\pm$  3% tolerance by Target Rock. Thus, a contributing cause of 7 of the 8 values' being outside of the  $\pm$ 1% Tech. Specs. tolerance is a setpoint that is too conservative for these SRV's to consistently meet.

# Immediate Corrective Action:

The valves were disassembled and cleaned. The labyrinth seal area was inspected and discrepancies were corrected. The valves were reassembled and functionally tested satisfactorily.

# Supplemental Corrective Action:

N/A

# Scheduled (future) corrective action:

N/A

# Action to prevent recurrence (if different from corrective actions):

A request to change the Tech. Specs. tolerance to a value that the valves can meet is currently being prepared for sumbittal to the NRC.

Georgia Power Company
 Post Office Box 439
 Baxley, Georgia 31513
 Telephone 912 367-7781
 912 537-9444



Edwin I. Hatch Nuclear Plant

October 5, 1984 GM-84-189

PLANT E. I. HATCH Licensee Event Report Docket No. 50-321

United States Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Attached is Licensee Event Report No. 50-321/1983-105, Rev. 1. This report is required by Hatch Unit 1 Technical Specifications Section 6.9.1.9.b.

Carry 1 H. C. Nix

General Manager

HCN/SBT/djs

xc: R. J. Kelly R. E. Conway J. T. Beckham, Jr. P. D. Rice K. M. Gillespie S. B. Tipps R. D. Baker Control Room Document Control