

NARRATIVE REPORT
FOR LER 50-321/1983-105, Rev. 1
UPDATE REPORT - PREVIOUS REPORT DATE 12/09/83

LICENSEE : GEORGIA POWER COMPANY
FACILITY NAME : EDWIN I. HATCH
DOCKET NUMBER : 50-321

Tech. Specs. section(s) which requires report:

This 30-day LER is required by Tech. Specs. section 6.9.1.9.b due to the event's showing that the unit was not meeting the requirements of Tech. Specs. section 2.2.A.1.b.

Plant conditions at the time of the event(s):

On 11/15/83, the plant was in cold shutdown for a fuel reconstitution outage when this event occurred.

Detailed description of the event(s):

On 11/15/83, plant personnel were notified that during a safety relief valve (SRV) bench test at Wyle Laboratory, 1B21-F013 A, B, D, F, H, J, K, & L lifted at 1110, 1152, 1112, 1062, 1104, 1126, 1102, and 1065 PSIG respectively. Thus the valves did not lift within the specified setpoint plus or minus 1% as required by Tech. Specs. section 2.2.A.1.b.

Consequences of the event(s):

Plant operation was not affected by this event. The health and safety of the public were not affected by this event.

Status of redundant or backup subsystems and/or systems:

Safety relief valve's 1B21-F013 C, E, and G remained operable during this event.

Justification for continued operation:

The safety relief valves were repaired and functionally tested satisfactorily prior to reactor startup as required by Tech. Specs. section 3.6.H.1.

If repetitive, number of previous LER:

This is a repetitive event with respect to valves 1B21-F013 F and H as last reported on LER 50-321/1982-099.

Impact to other systems and/or Unit:

This event had no effect on any other Unit 1 system. Target Rock Model 7567F safety relief valves are utilized by both Unit 1 and Unit 2 as main steam safety relief valves. However, this event did not affect Unit 2.

Cause(s) of the event(s):

The valve failures were the result of a stuck pilot disc, and/or lack of proper clearance of the pilot rod in the labyrinth seal area.

The Target Rock model 7567F safety relief valves used on Units 1 and 2 are only certified to a $\pm 3\%$ tolerance by Target Rock. Thus, a contributing cause of 7 of the 8 valves' being outside of the $\pm 1\%$ Tech. Specs. tolerance is a setpoint that is too conservative for these SRV's to consistently meet.

Immediate Corrective Action:

The valves were disassembled and cleaned. The labyrinth seal area was inspected and discrepancies were corrected. The valves were reassembled and functionally tested satisfactorily.

Supplemental Corrective Action:

N/A

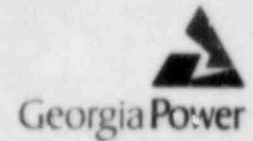
Scheduled (future) corrective action:

N/A

Action to prevent recurrence (if different from corrective actions):

A request to change the Tech. Specs. tolerance to a value that the valves can meet is currently being prepared for submittal to the NRC.

• Georgia Power Company
• Post Office Box 439
Baxley, Georgia 31513
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Edwin I. Hatch Nuclear Plant

October 5, 1984
GM-84-189

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-321

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Attached is Licensee Event Report No. 50-321/1983-105, Rev. 1.
This report is required by Hatch Unit 1 Technical Specifications
Section 6.9.1.9.b.

A handwritten signature in cursive script that reads 'Harry Nix'.

H. C. Nix
General Manager

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HCN/SBT/djs

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