

NRC Form 365 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION  
APPROVED OMB NO. 3150-0104  
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Pilgrim Nuclear Power Station (PNPS) - Unit 1 DOCKET NUMBER (2) 050002931 PAGE (3) 1 OF 2

TITLE (4) Safety Valve Setpoints Below Requirement of Technical Specifications

Table with columns: EVENT DATE (5), LER NUMBER (6), REPORT DATE (7), OTHER FACILITIES INVOLVED (8). Includes sub-columns for month, day, year, sequential number, revision number, and facility names.

Table for regulatory requirements: 20.402(b), 20.405(a)(i)-(v), 20.406(a)(i)-(v), 20.406(b), 20.406(c)(1)-(2), 50.73(a)(2)(i)-(x), 50.73(b), 50.73(c), 50.73(d)(vii)-(B), 50.73(e)(x).

LICENSEE CONTACT FOR THIS LER (12) NAME: R. Schifone - Plant Engineer TELEPHONE NUMBER: 617 746-7900

Table for component failure: COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13). Columns include CAUSE, SYSTEM, COMPONENT, MANUFACTURER, REPORTABLE TO NRPDS.

SUPPLEMENTAL REPORT EXPECTED (14) YES (if yes, complete EXPECTED SUBMISSION DATE) NO X

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 3/28/84, during a refueling outage, the Maintenance Department was notified by Wyle Laboratories that both of the Main Steam Safety Valves exhibited set pressures more than 1% below the nameplate set pressure. This is contrary to the requirements of PNPS Technical Specification (T.S.) 2.2.C, which requires both valves to lift at 1240 psi ± 13 psi. When tested, one valve lifted at 1209 psi, and the other lifted at 1155 psi.

The cause of this deviation has been determined to be the set pressure calibration method. The procedure allowed for the use of the nitrogen as a substitute test gas, in lieu of steam.

As a result of this determination, future safety valve testing/calibration will be performed using steam as the test medium.

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S PDR

Handwritten signature/initials: TEGG 41

NRC Form 366A  
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104  
EXPIRES: 8/31/85

FACILITY NAME (1) Pilgrim Nuclear Power Station - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 9 3 8 4	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		0	04	01	02	OF 02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 3/28/84, during a refueling outage, the Maintenance Department was notified by Wyle Laboratories that both of the PNPS Dresser Main Steam Safety Valves, Model 3777, exhibited set pressures more than 1% below the nameplate set pressure. This is contrary to the requirements of PNPS Technical Specification (T.S.) 2.2.C, which requires both valves to lift at 1240 psi ± 13 psi. The subject valves were being tested in accordance with the requirements of T.S. 4.6.D.1.

Both valves were tested twice. Valve #203-4A, Serial #BK6262 lifted at 1213 psi during Test 1 and 1209 psi during Test 2.

Valve #203-4B, Serial #BK6309 lifted at 1165 psi during Test 1 and 1155 psi during Test 2.

Subsequent evaluation has indicated that the procedure used for set pressure calibration was inadequate. The procedure allowed for the use of nitrogen as a test gas, in lieu of steam. It has been concluded that this substitution resulted in the safety valves' pressure setpoints being in error.

On 4/7/84, at Wyle Lab., the above-mentioned valves were calibrated for set pressure and tested for leakage with steam as the test medium. Both valves were certified as acceptable.

As a result of these findings, future safety valve testing/calibration will be performed with steam as the test medium as indicated by Station Procedure 3.M.4-7.

This event did not impact the health and safety of the public.

A search of records indicates no previous occurrences of a similar nature.

BOSTON EDISON COMPANY  
800 BOYLSTON STREET  
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON  
SENIOR VICE PRESIDENT  
NUCLEAR

October 10, 1984

BECo Ltr. #84-171

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U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

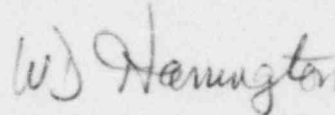
Docket Number 50-293  
License DPR-35

Dear Sir:

The attached update Licensee Event Report 84-004-01, "Safety Valve Setpoints Below Requirement of Technical Specifications," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,



W. D. Harrington

PH:caw

Enclosure: LER 84-004-01

cc: Dr. Thomas E. Murley  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

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