Update Report: Previous Report Date - May 27, 1983

NRC FORM 366 (12-81) 10 CFR 50

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

APPROVED BY OMB 3150-0011

CONTROL BLOCK: [| | | | | |] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	LICENSEE CODE: P A S E S 1 (2)	LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 (3)	LICENSE TYPE: 4 1 1 1 1 (4)	CAT (5)

CON'T

01	REPORT SOURCE: L (6)	DOCKET NUMBER: 0 5 0 0 0 3 8 7 (7)	EVENT DATE: 0 5 1 7 8 3 (8)	REPORT DATE: 0 1 3 1 8 5 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 Flow rates required to maintain a positive pressure in the Control Structure
03 appeared to exceed limits specified in Tech. Specs. A surveillance procedure re-
04 view showed the flow limit acceptance criteria to be missing. The system involved
05 is used to minimize radiation exposure of Control Room personnel during accident
06 conditions. Testing on 5/16/84 showed actual flow to be less than indicated flow;
07 Instruments were calibrated to represent actual flow. During the event there were
08 no effects on the public's health and safety.

09	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
	S G (11)	D (12)	Z (13)	Z Z Z Z Z Z (14)	Z (15)	Z (16)

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
8 3	—	0 7 0	/	0 1	X	—	1

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER (26)
X (18)	Z (19)	C (20)	Z (21)	0 0 0 0	Y (23)	N (24)	Z (25)	Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The system is required to maintain one-eight in. Positive pressure with a
11 system flow of 5810 scfm. This flow limit was inadvertently omitted from the
12 procedure. The last surveillance (7-13-82) recorded flows of 6000 and 6200 scfm
13 on the respective subsystem. The procedure was corrected, the test boundary was
14 reworked as necessary and the test successfully completed on 5/23/83.

15	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
	B (28)	0 0 0 (29)	NA	A (31)	Procedure review

16	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

17	NUMBER	TYPE	DESCRIPTION (39)
	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

18	NUMBER	DESCRIPTION (41)
	0 0 0 (40)	NA

8502120460 850131
PDR ADOCK 05000387
S PDR

LOSS OF OR DAMAGE TO FACILITY (43)

19	TYPE	DESCRIPTION
	Z (42)	NA

PUBLICITY

20	ISSUED	DESCRIPTION (45)	NRC USE ONLY
	N (44)	NA	

NAME OF PREPARER B.L. Wilks PHONE: (717)542-3914

IE22 1/1

ATTACHMENT

LER # 83-070/01X-1

Pennsylvania Power & Light Company
Susquehanna Steam Electric Station
Docket Number: 50-387

The Control Room Emergency Outside Air Supply System (CREOASS) is required to maintain the Control Structure at one eighth inch positive pressure, relative to outside atmosphere, to limit radiation exposure to Control Room personnel during accident conditions. The pressurization mode is used if radiation is sensed. The Control Structure is isolated and CREOASS goes into recirculation if chlorine is sensed. The Technical Specification Flow Limit for the pressurization mode is 5810 CFM and established to limit radioactive isotope introduction into the Control Room atmosphere. This limit was inadvertently omitted from the surveillance test procedure.

Upon discovery of this omission, the previous test results were checked (test frequency is 18 months). The previous test performed on 7/13/82 showed Loop A flow to be 6200 CFM with pressures from .125 to .132 inches, Loop B flow was 6000 CFM with pressures from .15 to .17 inches.

A procedure change was issued to incorporate the flow limit of 5810 CFM. The test boundary, which includes Control Structure elevations 698' to 783', was inspected to identify and correct any leakage paths. The test was successfully completed on 5/23/83 and the system declared operable. During performance of the test some additional flow measurements were taken (for informational purposes, outside of the PP&L quality program) which indicated that actual flow rates may be significantly below Technical Specification limits. Test (TP-030-002) performed on 5/16/84 showed the actual CREOASS common duct flow rate was less than that indicated by the measuring instrument's recorders. At the same time, traverse measurements of the common duct indicate the location at which duct flow is measured was not a problem. From this data, setpoint changes were performed to calibrate CREOASS recorded flow with the actual discharge flow as seen by the system's flow transmitters. Measurements taken on 6/12/84 indicate actual average CREOASS flow to be well within the ±10% of 5810 CFM required by Technical Specification section 4.7.2.b.1.

At the Technical Specification limit of 5810 CFM the dose to Control Room personnel during a Design Basis Accident (DBA) would be 14 REM (thyroid) per FSAR, Chapter 15 Accident Analysis. The 10CFR50, Appendix A, Criteria 19, Exposure Limit for Control Room personnel, is 30 REM (thyroid). The additional exposure which would be expected at a flow rate of 6200 CFM, compared to 5810 CFM, is less than one REM, which would still be approximately 50 percent of 10CFR50 limits.

A review of similar procedures was conducted to ensure incorporation of Technical Specification requirements and limits. During this event there were no occurances requiring this system operation and there were no effects on the public's health and safety.

PP&L

SUSQUEHANNA STEAM ELECTRIC STATION
PO BOX 467, BERWICK, PA 18603

January 31, 1985

Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 83-070/01X-1
ER 100450 FILE 841-23
PLAS-033

Docket No. 50-387
License No. NPF-14

Dear Dr. Murley:

Attached is Licensee Event Report 83-070/01X-1. This event was determined to be reportable per Technical Specification 6.9.1.8.b, in that surveillance procedures did not reflect Technical Specification Flow Limits for the Control Room Emergency Outside Air Supply System (CREOASS), Surveillance Requirement 4.7.2.d.3. Revision 1 of this Licensee Event Report contains information concerning the corrective actions taken as a result of this event.

H.W. Keiser
Superintendent of Plant-Susquehanna

BLW/pjg

Attachment

cc: Mr. R.H. Jacobs
Senior Resident Inspector
U.S. Nuclear Regulatory Commission
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