NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Sequoyah, Unit 1	0 5 0 0 0 3 2 7	1 OF 0 2

TITLE (4)

Containment Building Ventilation Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 9	1 7	8 4	8 4	— 0 5 8	— 0 0	1 0	1 6	8 4		0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
1	20.402(b)	20.405(c)	XX 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 1 0 0	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Glenn E. Duggin, Compliance Section Engineer	AREA CODE 6 1 5 8 7 0 - 6 1 4 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	I L	- - F S	3 4 4 5	Yes							

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	XX NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A high radiation alarm was actuated, which caused a containment ventilation isolation (CVI) to occur. Investigation revealed that due to a leak in the manway cover on steam generator number one, steam entered the containment atmosphere and the resulting increase in moisture saturated the particulate filter and caused the iodine sample flow alarm to actuate. The contacts on the flow switch were very noisy and generated electromagnetic interference (EMI) which caused the high radiation alarm to actuate. There was no actual high radiation level in containment and no personnel were contaminated.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Sequoyah, Unit 1	0 5 0 0 0 3 2 7	8 4	— 0 0 5	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

The containment ventilation isolation (CVI) occurred at 0109 CST on 09/17/84 while unit 1 was in mode 1 (100 percent power, 2235 psig, 578 degrees F). The CVI was reset immediately and the radiation monitor was returned to service at 1330 CST on 09/17/84. All associated equipment and personnel responded and performed as expected during the CVI. The operator responded to the alarm (RM-90-112B) and determined that the alarm was not caused by a high radiation level. The radiation monitor (RM) channel was blocked and the CVI reset at 0115 CST on 09/17/84.

A steam leak from the upper manway cover of steam generator (S/G) number one increased the moisture content of the containment building enough to saturate the particulate filter. The steam leak was on the secondary side of the S/G and the leak did not result in any release of radioactivity to the containment environment. The manway cover gasket had been cut by steam and had to be replaced. The manway cover was repaired under a special maintenance instruction. This particulate filter is in line with the iodine sample chamber to keep the iodine chamber clean. The saturation of the particulate filter with water slowed the flow of air and actuated the low flow switch. The low flow switch had noisy contacts, i.e., the contacts generated a lot of electromagnetic interference (EMI), which caused a spike to occur on the radiation monitor high enough to bring in the high radiation alarm. The high radiation alarm actuated the CVI. The alarmed channel in the radiation monitor was blocked and the CVI was reset. The flow switch and particulate filter were replaced and the radiation monitor was returned to service at 1330 CST on 09/17/84 by performing Surveillance Instruction (SI)-206, "Radiation Monitoring System Sample Flow Calibrations and Functional Tests".

This RM (112) has had a three-second time delay installed on the alarm circuit to help prevent inadvertent spikes from causing a false alarm. In this particular instance, the EMI spikes were high and continuous. A waiver of reporting requirements for this type of event has been requested from the NRC.

There was no effect on public health or safety and no plant safety margins were exceeded. Radiation levels were not above normal during this time.

Previous occurrences - SQRO-50-327/84001, 84003, 84009, 84012, 84014, 84017, 84020, 84022, 84027, 84047, and 84056. These CVIs have occurred on unit 1 for the year 1984.

TENNESSEE VALLEY AUTHORITY
Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

October 16, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO. 50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT SQRO-50-327/84058

The enclosed licensee event report provides details concerning the containment ventiiation isolation due to a spike on a radiation monitor. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

P.R. Wallace

P. R. Wallace
Plant Manager

Enclosure
cc (Enclosure):

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NRC Inspector, NUC PR, Sequoyah

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