ARKANSAS POWER & LIGHT COMPANY
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February 4, 1985

1CAN028501

Director of Nuclear Reactor Regulation
ATTN: Mr. J. F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Arkansas Nuclear One - Unit 1
Docket No. 50-313
License No. DPR-51
ANO-1 Second 10-Year Inservice
Inspection Program Relief
Request Submittal

Gentlemen:

Our letter dated January 31, 1985, (1CAN018507) indicated that we would transmit the relief requests for the ANO-1 second 10-year Inservice Inspection Program under separate cover. Our requests for relief are enclosed with this letter.

Very truly yours,

J. Ted Enos
Manager, Licensing

JTE:DET:ds

Attachment

A047
1/1

ATTACHMENT TO 1CANØ285Ø1

ASME CLASS I COMPONENTS
NDE INSERVICE INSPECTION RELIEF REQUESTS
FOR ANO-1 BASED ON ASME SECTION XI-1980
CODE THROUGH WINTER 1981 ADDENDA

IWB-2500-1 ITEM NO.	EXAMINATION CATEGORY	SYSTEM OR COMPONENT	AREA TO BE EXAMINED	IMPRACTICAL CODE REQUIREMENT	REASON FOR REQUEST	LICENSEE PROPOSED ALTERNATE EXAMINATION
B5.10	B-F	Reactor Vessel	Two Core Flood Safe End Butt Welds; Weld Nos. 01-025 and 01-026.	Surface Examination of Safe End Circumferential Butt Welds.	Access would require removal of the canal seal plate, shielding bricks, shielding supports in the nozzle areas, and insulation removal. This would require approximately 300 manhours of work in a 700-1000 mR/hr area.	None - These welds will be examined 100% from the I.D.
B9.11	B-J	Reactor Vessel	Two Inlet Nozzle to Pipe Circumferential Butt Welds; Weld Nos. 01-019 and 01-022.	Surface Examination of Inlet Nozzle to Pipe Circumferential Butt Welds.	Access would require removal of the canal seal plate, shielding bricks, shielding supports in the nozzle areas, and insulation removal. This would require approximately 300 manhours of work in a 700-1000 mR/hr area.	None - These welds will be examined 100% from the I.D.

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B9.11	B-J	Piping Pressure Boundary	Circumferential Pipe Welds: High Pressure Injection Lines; Weld Nos. 20-008A 21-026, 22-024A and 22-027A. Core flood lines; weld nos. 19-019B and 19-021A.	The area of examination shall include 100% of the weld length.	High pressure injection welds 200-008A, 21-026, 22-024A and 22-027A are inside penetrations Nos. 13, 34, 8 and 15 respectively in the shield wall and are not accessible for examination. Core flood welds 19-019B and 19-021A are inaccessible for examination due to pipe supports. These welds were not examined during baseline and this face is documented in the preoperational inspection report.	None
B12.10	B-L-1	Reactor Coolant Pump Pressure Boundary	Pressure Retaining Welds in Pump Casings	Volumetric: 100% of the pressure retaining weld in at least one pump in the 120 month inspection interval.	Pump Casing internal surfaces are inaccessible without dismantling the pump.	Volumetric: Approximately 95% of the pressure retaining weld in at least one pump in the 120 month inspeciton interval.

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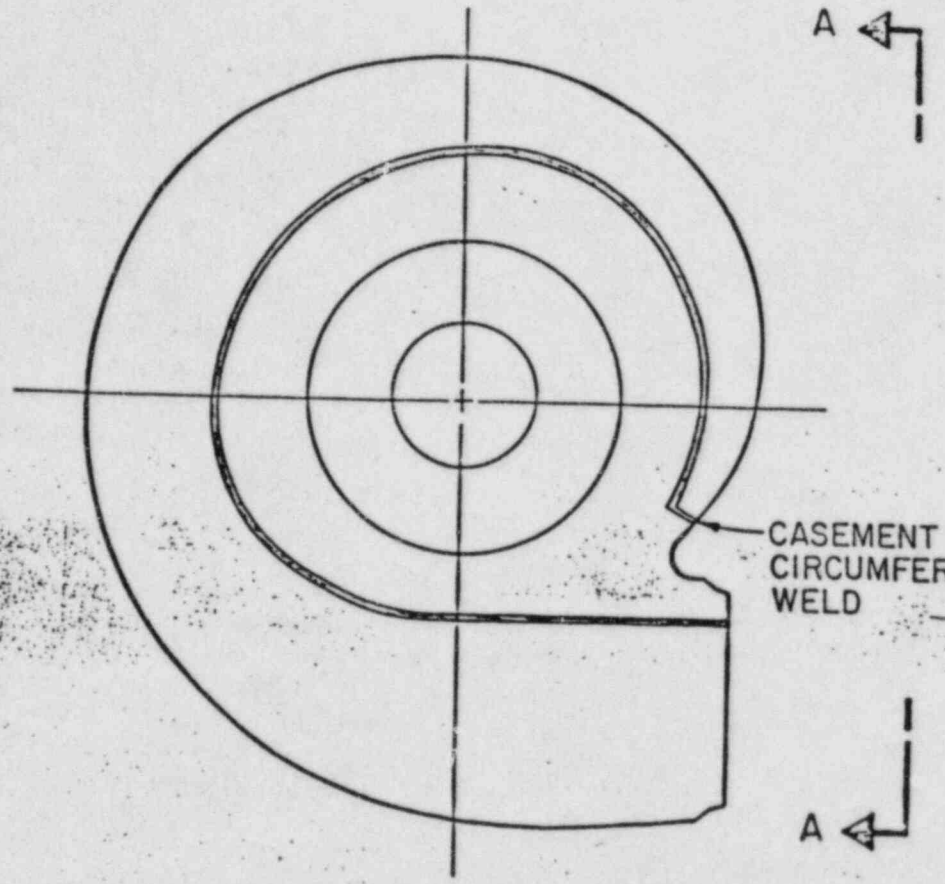
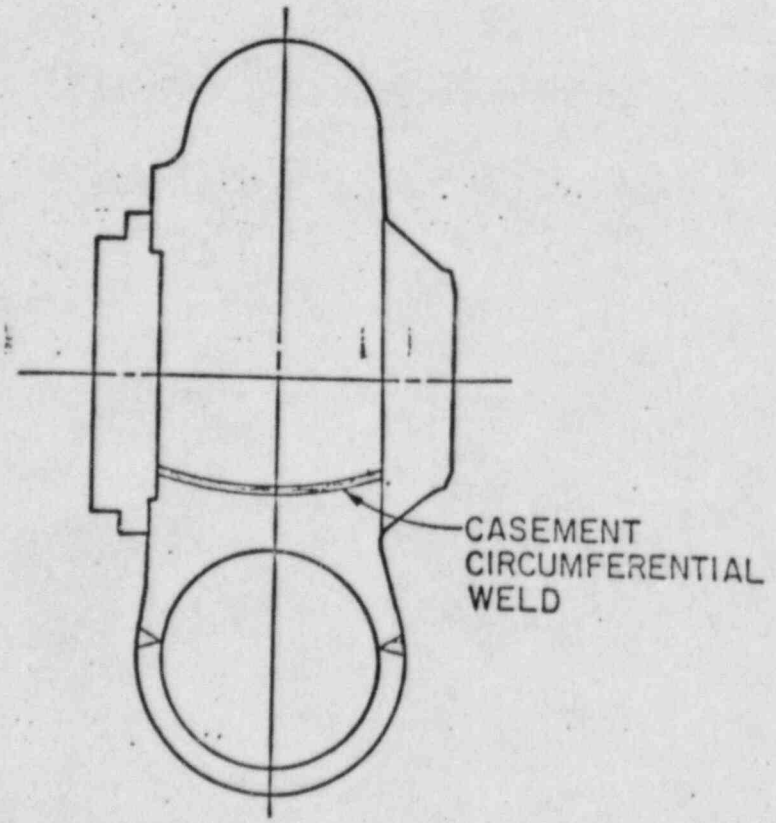
B12.20	B-L-2	Reactor Coolant Pump Pressure Boundary	Pump Casing Internal Pressure Boundary Surfaces.	Visual	Pump casing internal surfaces are inaccessible without dismantling the pump.	Partial surface replication obtainable from casing weld volumetric examination.

FIGURE 1





VIEW A

ARKANSAS NUCLEAR ONE
REACTOR COOLANT PUMP
UNIT 1