

September 24, 1992

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Docket No. 50-219

Mr. John J. Barton  
Vice President and Director  
GPU Nuclear Corporation  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, New Jersey 08731

SUBJECT: SAFETY EVALUATION OF THE INSERVICE TESTING PROGRAM RELIEF REQUESTS  
FOR PUMPS AND VALVES, OYSTER CREEK NUCLEAR GENERATING STATION  
(TAC NO. M82017)

By letter dated October 11, 1991, GPU Nuclear Corporation submitted the Oyster Creek Nuclear Generating Station third ten year pump and valve inservice testing (IST) program. The third ten year interval started October 14, 1991. Additional information was provided in conference calls between the NRC and GPU Nuclear Corporation (GPUN) on June 17, 1992, and June 22, 1992. The enclosed Safety Evaluation (SE) was prepared by the staff to provide the results of the staff review of the relief requests contained in this submittal.

The Code of Federal Regulations, 10 CFR 50.55a, requires that inservice testing (IST) of certain ASME Code Class 1, 2, and 3 pumps and valves be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable addenda, except where specific written relief has been requested by the licensee and granted by the Commission pursuant to Sections (a)(3)(i), (a)(3)(ii), or (g)(6)(i) of 10 CFR 50.55a. Guidance on acceptable alternatives to Section XI requirements was provided by NRC in GL 89-04 for certain aspects of IST.

The staff, with technical assistance from Idaho National Engineering Laboratory (INEL), has reviewed the information concerning the IST program requests for relief submitted for the Oyster Creek Nuclear Generating Station. The staff adopts the evaluations and conclusions contained in the enclosed Technical Evaluation Report (TER) prepared by INEL. Relief is granted from the testing requirements which we have determined would be impractical to perform, where compliance would result in a hardship without a compensating increase in safety, or where the proposed alternative testing provides an acceptable level of quality and safety. A summary of the NRC actions is provided in Table 1 of the SE. For relief requests that have been denied, GPUN's testing should comply with the applicable Section XI requirements and GL 89-04 guidelines within 3 months of the date of this letter. Regarding the relief requests which have been denied, any additional information supporting the relief requested should be submitted within 3 months of the date of this letter. The IST program relief requests that are not denied are acceptable for implementation provided (1) anomalies identified in Appendix A of the TER are addressed within 1 year of the date of this letter and (2) procedural or program changes described in Appendix A are completed within 1 year of the date of this letter, by the end of the next refueling outage, or by the schedule specified in an individual action item, whichever is later. The

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Mr. John J. Barton

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granting of relief is based upon the fulfillment of any commitments made by GPUN in its basis for each relief request and the alternatives proposed. For relief requests which have been denied, testing is to comply with the Code requirements or Generic Letter 89-04, or additional relief requested, within 90 days of the date of this letter.

Program changes involving new or revised relief requests should not be implemented prior to approval by the NPC. New or revised relief requests that meet the positions in Generic Letter 89-04, Attachment 1, should be submitted to the NRC but can be implemented provided the guidance in Generic Letter 89-04, Section D, is followed. Program changes that add or delete components from the IST program should also be provided to the NRC.

The NRC staff has determined that granting relief, pursuant to 10 CFR 50.55a(a)(3)(i), (a)(3)(ii) or (g)(6)(i), is authorized by law and will not endanger life, property, or the common defense and security and is otherwise in the public interest giving due consideration to the burden upon the licensee that could result if the ASME Code requirements were imposed on the facility.

Sincerely,

Original signed by

John F. Stolz, Director  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Safety Evaluation
2. Technical Evaluation Report

cc w/enclosures:  
See next page

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Mr. John J. Barton  
GPU Nuclear Corporation

Oyster Creek Nuclear  
Generating Station

cc:

Ernest L. Blake, Jr., Esquire  
Shaw, Pittman, Potts & Trowbridge  
2300 N Street, NW.  
Washington, DC 20037

Resident Inspector  
c/o U.S. Nuclear Regulatory Commission  
Post Office Box 445  
Forked River, New Jersey 08731

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406

Kent Tosch, Chief  
New Jersey Department of  
Environmental Protection  
Bureau of Nuclear Engineering  
CN 415  
Trenton, New Jersey 08625

BWR Licensing Manager  
GPU Nuclear Corporation  
1 Upper Pond Road  
Parsippany, New Jersey 07054

Mayor  
Lacey Township  
818 West Lacey Road  
Forked River, New Jersey 08731

Licensing Manager  
Oyster Creek Nuclear Generating Station  
Mail Stop: Site Emergency Bldg.  
Post Office Box 388  
Forked River, New Jersey 08731