

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
BYRON STATION

DOCKET NUMBER (2)  
0 5 0 0 0 | 4 5 4

PAGE (3)  
1 OF 2

TITLE (4)  
1A STEAM GENERATOR LOW LOW LEVEL

EVENT DATE (5)			LER NUMBER (6)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	DOCKET NUMBER(S)
01	04	85	85	008	00	02	01	85	0 5 0 0 0
									0 5 0 0 0

OPERATING MODE (9) 4

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

20.402(b)	<input checked="" type="checkbox"/>	20.406(a)	<input type="checkbox"/>	20.72(a)(2)(vi)	<input type="checkbox"/>	73.71(b)	<input type="checkbox"/>
20.406(a)(1)(i)	<input type="checkbox"/>	20.406(a)(1)	<input type="checkbox"/>	20.72(a)(2)(vii)	<input type="checkbox"/>	73.71(c)	<input type="checkbox"/>
20.406(a)(1)(ii)	<input type="checkbox"/>	20.406(a)(2)	<input type="checkbox"/>	20.72(a)(2)(viii)	<input type="checkbox"/>	OTHER (Specify in Abstract below and in Text, NRC Form 300A)	<input type="checkbox"/>
20.406(a)(1)(iii)	<input type="checkbox"/>	20.72(a)(2)(i)	<input type="checkbox"/>	20.72(a)(2)(ix)	<input type="checkbox"/>		
20.406(a)(1)(iv)	<input type="checkbox"/>	20.72(a)(2)(ii)	<input type="checkbox"/>	20.72(a)(2)(x)	<input type="checkbox"/>		
20.406(a)(1)(v)	<input type="checkbox"/>	20.72(a)(2)(iii)	<input type="checkbox"/>	20.72(a)(2)(xi)	<input type="checkbox"/>		
20.406(a)(1)(vi)	<input type="checkbox"/>	20.72(a)(2)(iv)	<input type="checkbox"/>	20.72(a)(2)(xii)	<input type="checkbox"/>		

LICENSEE CONTACT FOR THIS LER (12)

NAME: Larry Alexander, Tech Staff Engineer, Ext. 388

TELEPHONE NUMBER: 815 234-5441

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
A	JIB	ISIG	/ / / / /	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE: )  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

With the plant in mode 4, a reactor trip and Auxiliary Feedwater actuation occurred due to a Steam Generator low-low level alarm. The operators inadvertently allowed the level to drop in the Steam Generator while engaged in a shift turnover and in a Main Steam valve lineup to drain condensation. The Operating Staff is being appraised for this event through the required reading program.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  BYRON, UNIT 1	DOCKET NUMBER (2)  0 5 0 0 0 4 5 4	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	- 0 0 8	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 308A's) (17)

On 1/4/85 at 2248, with the plant operating in Mode 4, the 1A Steam Generator experienced a low-low level alarm resulting in a reactor trip and Aux Feedwater actuation. Prior to the event, the Steam Generator Blowdown System was valved in at 1800 to decrease the level in the Steam Generators in preparation for heat-up. At 1849, Steam Generator Blowdown was secured and at 2115 the 1A Steam Generator low level alarm annunciated in the Main Control Room. The Unit Operator suspected a leaky blowdown valve and manually isolated the system at this time. However, the Unit Operator, involved in his shift turnover, did not recheck his level instrumentation (assuming the blowdown isolation corrected the problem), and at 2248 the low-low Steam Generator level alarm initiated the reactor trip. In the time between the low level and the low-low level alarms, a Main Steam valve line-up to drain condensation that might have formed in the steam lines was being conducted. This valve line-up took long enough to create the additional level loss in the 1A Steam Generator due to condensation. There was no indication of an increase in water inventory in the Blowdown System.

Plant and public safety were not affected by this event since the control rods were fully inserted and there was no decay heat in the core. The operating staff is being appraised of this event through the station's required reading program.

Previous occurrences: None



**Commonwealth Edison**  
Byron Nuclear Station  
4450 North German Church Road  
Byron, Illinois 61010

February 1, 1985

BYRON LTR: 85-0170

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington D.C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR 50.73 (a)(2)(iv) which requires a 30 day written report.

This report is number 85-008-00, Docket No. 50-454.

Very truly yours,

R. E. Querio  
Station Superintendent  
Byron Nuclear Power Station

Enclosure: Licensee Event Report No. 85-008-00

cc: J. G. Keppler, NRC Region III Administrator  
J. Hinds, NRC Resident Inspector  
INPO Record Center  
CECo Distribution List

IE22  
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