U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO 3150-0104 EXPIRES 8/31/85 LICENSEE EVENT REPORT (LER) DOCKET NUMBER (2) PACILITY NAME (1) OF 0 | 2 0 |5 | 0 | 0 | 0 | 2 | 9 | 6 1 Browns Ferry - Unit 3 Momentary Inoperable Reactor Core Isolation Cooling Due To A Personnel Error OTHER FACILITIES INVOLVED (8) EVENT DATE (6) LER NUMBER (6) REPORT DATE (7) DOCKET NUMBER(S) SEQUENTIAL MONTH DAY MONTH DAY YEAR YEAR 0 | 5 | 0 | 0 | 0 | 0 1 0 4 8 5 8 5 8 0 | 5 | 0 | 0 | 0 0 0 0 0 0 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR \$ (Check one or more of the following) (11) OPERATING MODE (9) N 73.71(6) 20.405(e) 50.73(a)(2)(iv) 50.73(a)(2)(v) 73.71(c) POWER LEVEL (10) 20.406(a)(1)(i) 50.36(c)(1) OTHER (Specify in Abstract below and in Text, NRC Form 366A) 1 10 10 50 73(a)(2)(vii) 20.405(a)(1)(ii) 50 38(+)(2) 50.73(a)(2)(viii)(A) 20.408(a)(1)(iii) 50 73(a)(2)(viii)(B) 20 405(4)(1)(()) 80 73(4)(2)(6) 20.405(a)(1)(v) 50.73(a)(2)(iii) 50.73(s)(2)(x) LICENSEE CONTACT FOR THIS LER (12) TELEPHONE NUMBER NAME AREA CODE David L. Smith 21015 712191-13181615 COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) REPORTABLE TO NPROS MANUFAC TURER TO NPROS CAUSE SYSTEM COMPONENT CAUSE SYSTEM COMPONENT SUPPLEMENTAL REPORT EXPECTED (14) MONTH YEAR EXPECTED DATE (15) YES (If yes, complete EXPECTED SUBMISSION DATE) X

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

During normal operation, an employee accidentally caused the Reactor Core Isolation Cooling System to be inoperable for approximately two minutes. All other required core standby cooling systems were fully operable. No other occurrences accompanied this event.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85

| FACILITY NAME (1) | | DOCKET NUMBER (2) | | | | | | | | | LER NUMBER (6) | | | | | | | | | PAGE (3) | | | | | |
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| | | | | | | | | | | | Y | EAR | | SEC | UENTI | AL | | REVISION NUMBER | | | | | | | |
| Browns Fer | ry | - | Unit | 3 | 0 | 5 | 10 | 10 | 10 | 12 | 19 | 16 | 8 | 5 | _ | 0 | 0 1 | - | - | 0 0 | 0 | 2 | OF | 0 | 2 |

TEXT (If more space is required, u'a additional NRC Form 366A's) (17)

During normal plant operations, unit 1 was operating at 98 percent, unit 3 at 98 percent, and unit 2 was in a refueling outage.

As an employee was performing system checks to update plant configuration drawings, he accidentally hit the Reactor Core Isolation Cooling (BN) turbine stop valve (ISV) actuator. This in turn caused the valve, FCV-71-9, to close giving the unit 3 licensed reactor operator an annunciator (ANN) alarm (ALM) 71-49A. Within two minutes, the operator initiated valve reopening.

All required core standby cooling systems were fully operable during this event. The employee was cautioned to be more vigilant around critical equipment. No other factors were involved, and since this was random in nature, no further corrective action is required.

Responsible Plant Section - OPS

Previous Similar Events - None

TENNESSEE VALLEY AUTHORITY
Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

January 29, 1985

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 3 - DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE REPORT BFR0-50-296/85001

The enclosed report provides details concerning momentary inoperability of the Reactor Core Isolation Cooling due to a personnel error. This report is submitted in accordance with 10 CFR 50.73 (a)(2)(v).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

G. T. Jones Plant Manager

Browns Ferry Nuclear Plant

Enclosures

cc (Enclosures):
Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

NRC Resident Inspector, BFN

INPO Records Center Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339

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