

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SURRY POWER STATION, UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 2 8 0	PAGE (3) 1 OF 0 2
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TITLE (4)
REACTOR TRIP (LOW STEAM GENERATOR LEVEL)

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	1	3	8	5	0	0	2	0			0 5 0 0 0
0	1	3	8	5	0	0	2	0			0 5 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10) 0 2 0	20.402(b)	<input type="checkbox"/>	20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	<input type="checkbox"/>	73.71(b)	<input type="checkbox"/>	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
	20.406(a)(1)(ii)	<input type="checkbox"/>	50.36(c)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	<input type="checkbox"/>	73.71(c)	<input type="checkbox"/>		
	20.406(a)(1)(iii)	<input type="checkbox"/>	50.36(c)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	<input type="checkbox"/>		<input type="checkbox"/>		
	20.406(a)(1)(iv)	<input type="checkbox"/>	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	<input type="checkbox"/>		<input type="checkbox"/>		
	20.406(a)(1)(v)	<input type="checkbox"/>	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	<input type="checkbox"/>		<input type="checkbox"/>		
	20.406(a)(1)(vi)	<input type="checkbox"/>	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)	<input type="checkbox"/>		<input type="checkbox"/>		

LICENSEE CONTACT FOR THIS LER (12)

NAME R. F. SAUNDERS, STATION MANAGER	TELEPHONE NUMBER
	AREA CODE: 8 0 4 3 5 7 - 3 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On January 13, 1985, the unit was at 20% power with a load increase in progress following a brief shutdown to repair a primary system component leak. At 1656 hours during a transfer from the bypass to the main feedwater regulating valves, a reactor trip occurred due to a low low level in "B" steam generator.

The operator failed to adequately control steam generator level.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5 -	0 0 2 -	0 0	0 2	OF 0 2

TEXT (if more space is required, use additional NRC Form 366A's) (17)

1. Description of the Event

On January 13, 1985, the unit was at 20% power with a load increase in progress following a brief shutdown to repair a primary system component leak. At 1656 hours during a transfer from the bypass to the main feedwater regulating valves, a reactor trip occurred due to a low low level in "B" steam generator.

2. Probable Consequences

The reactor trip on low steam generator level will ensure adequate heat sink.

The auxiliary feedwater system remained operable throughout this event as a backup to the main feedwater system. Also, all other safety related systems remained operable and plant parameters remained within the bounds of the safety analysis. Therefore, an unreviewed safety question was not created and the health and safety of the public remained unaffected.

3. Cause

Control of steam generator level at low power levels during a startup is a generic problem at Westinghouse PWR's. The operator failed to adequately control steam generator level. No component malfunctions were identified.

4. Immediate Corrective Actions

The Operators performed all appropriate emergency and function restoration procedures to ensure that the plant was returned to stable conditions.

The STA performed the status tree reviews to ensure that specific parameters were noted and the appropriate procedures were used to maintain these parameters within safe bounds.

5. Additional Corrective Actions

Because there were no component malfunctions, no additional actions were taken.

6. Action Taken to Prevent Recurrence

A design and cost analysis to install feedwater bypass flow indication is in progress.

7. Generic Implications

As stated in section 3 above.

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY

Surry Power Station
P. O. Box 315
Surry, Virginia 23883

February 4, 1985

Serial No: 85-002

Docket No: 50-280

License No: DPR-32

U. S. Nuclear Regulatory Commission
Document Control Desk
015 Phillips Building
Washington, D.C. 20555

Gentlemen:

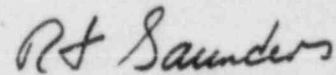
Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 1.

REPORT NUMBER

85-002-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,



R. F. Saunders
Station Manager

Enclosure

cc: Dr. J. Nelson Grace
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30323

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