

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 50-298

COOPER NUCLEAR STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 154 License No. DPR-46

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nebraska Public Power District (the licensee) dated October 7, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations:
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-46 is hereby amended to read as follows:
 - 2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 154, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of iscuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John T. Larkins, Director Project Directorate IV-1

Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 22, 1992

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Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE PAGES	INSERT PAGES
67 a	67a
67b	67b
82 a	82 a

COOPER NICLEAR STATION
TABLE 3.2 1

POST-ACCIDENT MONITORING INSTRUMENTATION REQUIREMENTS*

Elevated Release Point (ERF) Monitor (High Range Noble Cas) Turbine Building Ventilation Exhaust Monitor (High Range Noble Gas) Exhaust Monitor (High Range Noble Gas) Primary Containment Gross
Exhaust Monitor (High Range Noble Gas) Radwaste/Augmented Radwaste RHV-RM-308 Exhaust Monitor (High Range Noble Gas) Primary Containment Gross RMA-RM-40A Radiation Monitor

Other Post-Accident Monitoring Instrumentation is located in Table 3.2.F. Drywell Pressure, PC-FR-1A and 1B. Suppression Chamber/Torus Wacer Level PC-LR-1A and 1B Note:

NOTES FOR TABLE 3.2.H

Actions

- A. With the number of operable components less than required by the minimum component operable requirements, initiate the preplanned alternate method of monitoring the appropriate parameter(s) within 72 hours, and:
 - Either restore the inoperable component(s) to operable status within 7 days of the event, or
 - Prepare and submit a Special Report to the Commission within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.
- B. The following actions will be taken if the minimum number of operable instrument channels as required are not operable:
 - With the number of operable channels one less than the minimum number of operable channels shown, restore the inoperable channel to operable status within 30 days or be in at least hot shutdown within the next 12 hours.
 - 2) With no operable channels available, restore at least one channel to operable status within 7 days or be in at least hot shutdown within the next 12 hours.

Notes

- 1. These instruments are required to be operable at all times except when the reactor is in cold shutdown or in the REFUEL mode during a refueling outage.
- 2. With two channels operable, the normal condition is with one analyzer in the standby mode

COOPER NUCLEAR STATION TABLE 4.2.H

CALIBRATION PREQUENCY FOR POST-ACCIDENT MONITORING INSTRUMENTATION*

Instrument	Instrument ID Number	Function Test Frequency	Calibration Frequency
Elevated Release Point (ERP) Monitor (High Range Noble Gas)	RMP-RM-3B	Once/Month	Once/Cycle
Turbine Building Ventilation Exhaust Monitor (High Range Noble Gas)	RMV-RM-20B	Once/Month	Once/Cycle
Radwaste/Augmented Radwaste Exhaust Monitor (High Range Noble Gas)	RMV-RM-30B	Cnce/Month	Once/Cycle
Primary Containment Gross Radiation Monitors**	RMA-RM-40A RMA-RM-40B	Once/Mon'h	Once/Cycle
Primary Containment Hydrogen Concentration Analyzer	PC-AN-H ₂ /O ₂ I PC-AN-H ₂ /O ₂ II	Once/Month	Once/Quarter

*Note:

Other Post-Accident Monitoring Instrumentation culibration requirements are in Table 4.2.F-Dr, well Pressure, PC-PR-1A and 1B, Suppression Chamber/Torus Water Level PC-LR-1A and 1B.

^{**}CHANNEL CALIBRATION shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10 R/hr and a one point calibration check of the detector below 'O R/hr with an installed or portable gamma source.