

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 5	PAGE (3) 1 OF 0 1
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TITLE (4) Primary Containment Isolation of Reactor Water Cleanup System Due To Room Differential Temperature High

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																																																															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																																																																													
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<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td colspan="12">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)</td> </tr> <tr> <td colspan="3">OPERATING MODE (9) 1</td> <td colspan="3">20.402(b)</td> <td colspan="3">20.406(c)</td> <td colspan="3"><input checked="" type="checkbox"/> 50.73(a)(2)(iv)</td> <td colspan="3">73.71(b)</td> </tr> <tr> <td colspan="3">POWER LEVEL (10) 0 8 8</td> <td colspan="3">20.406(a)(1)(i)</td> <td colspan="3">50.38(c)(1)</td> <td colspan="3"><input type="checkbox"/> 50.73(a)(2)(v)</td> <td colspan="3">73.71(c)</td> </tr> <tr> <td colspan="3"></td> <td colspan="3">20.406(a)(1)(ii)</td> <td colspan="3">50.38(c)(2)</td> <td colspan="3"><input type="checkbox"/> 50.73(a)(2)(vii)</td> <td colspan="3" rowspan="4">OTHER (Specify in Abstract below and in Text, NRC Form 365A)</td> </tr> <tr> <td colspan="3"></td> <td colspan="3">20.406(a)(1)(iii)</td> <td colspan="3">50.73(a)(2)(i)</td> <td colspan="3"><input type="checkbox"/> 50.73(a)(2)(viii)(A)</td> </tr> <tr> <td colspan="3"></td> <td colspan="3">20.406(a)(1)(iv)</td> <td colspan="3">50.73(a)(2)(ii)</td> <td colspan="3"><input type="checkbox"/> 50.73(a)(2)(viii)(B)</td> </tr> <tr> <td colspan="3"></td> <td colspan="3">20.406(a)(1)(v)</td> <td colspan="3">50.73(a)(2)(iii)</td> <td colspan="3"><input type="checkbox"/> 50.73(a)(2)(x)</td> </tr> </table>												THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)												OPERATING MODE (9) 1			20.402(b)			20.406(c)			<input checked="" type="checkbox"/> 50.73(a)(2)(iv)			73.71(b)			POWER LEVEL (10) 0 8 8			20.406(a)(1)(i)			50.38(c)(1)			<input type="checkbox"/> 50.73(a)(2)(v)			73.71(c)						20.406(a)(1)(ii)			50.38(c)(2)			<input type="checkbox"/> 50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 365A)						20.406(a)(1)(iii)			50.73(a)(2)(i)			<input type="checkbox"/> 50.73(a)(2)(viii)(A)						20.406(a)(1)(iv)			50.73(a)(2)(ii)			<input type="checkbox"/> 50.73(a)(2)(viii)(B)						20.406(a)(1)(v)			50.73(a)(2)(iii)			<input type="checkbox"/> 50.73(a)(2)(x)		
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LICENSEE CONTACT FOR THIS LER (12)

NAME M. J. Pastva, Jr., Regulatory Technician	TELEPHONE NUMBER AREA CODE 9 1 9 4 5 7 - 2 3 1 5
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 1/10/85, at 2100, the Unit 1 Reactor Water Cleanup (RWCU) System inlet primary containment inboard isolation valve, 1-G31-F001, automatically closed due to a Group 3 primary containment isolation "A" subsystem trip. This trip was actuated by RWCU System room differential temperature high instrument, 1-G31-DTS-N602A. This event was revealed to the Unit 1 Control Operator through the RWCU System Steam Leak Detection Differential Temperature High annunciator on the Unit 1 Control Room reactor turbine gauge board. At the time, Unit 1 was operating at 88% power.

N602A actuated, in accordance with design, as the result of two minor steam leaks in the vicinity of the respective instrument temperature detector. The subject leaks were appropriately repaired in accordance with approved procedures, and the RWCU System was returned to service on January 12, 1985, at 0305.

This event occurred in the most limiting plant conditions with no significant impact on safety of the unit.

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Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429

February 1, 1985

FILE: B09-13510C
SERIAL: BSEP/85-0126

NRC Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-85-004

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/smp/LETSMP

Enclosure

cc: Mr. R. C. DeYoung
Mr. J. P. O'Reilly

IE22
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