

NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

AUG 03 1984

Ms. Ophelia G. Williams
J/R/A Associates
Regulatory Information and Support Systems
1304 Redditch Court
Largo, MD 20772

IN RESPONSE REFER TO FOIA-84-262

Dear Ms. Williams:

This is in further response to your letter dated April 5, 1984, in which you requested, pursuant to the Freedom of Information Act, a copy of the site audits of environmental qualification of equipment at the following plants since October 1982:

Waterford 3 (1/4 - 6/83) Midland 1 (1/11 - 13/83) Perry 1 (1983) Catawba 1 (3/84) Zimmer 1 (1983) Palo Verde 1 (1983) Clinton 1 (1983) Wolf Creek (5/83) WPPSS 2 (1983)

Appendix A is a list of documents which address audits for: WPPSS 2, Wolf Creek, Palo Verde, and Midland. These documents, other than item five, are presently in the PDR. You may obtain access to these documents by referencing the accession number listed on the Appendix. Item five is being placed in the PDR in file folder 84-262.

Additionally, document six is an internal memorandum containing the results of an audit conducted at TMI on March 20-21, 1984, with the results not yet finalized. This document does not contain any reasonably segregable factual portions. Release of this document would tend to inhibit the open and frank exchange of ideas essential to the deliberative process. The document reflects the predecisional process and, therefore, is exempt from mandatory disclosure pursuant to Exemption (5) of the Freedom of Information Act (5 U.S.C. 552(b)(5)) and 10 CFR 9.5(a)(5).

Pursuant to 10 CFR 9.9 of the Commission's regulations, it has been determined that the information withheld is exempt from production or disclosure and that its production or disclosure is contrary to the public interest. The persons responsible for this denial are the undersigned and Mr. Harold R. Denton, Director, Office of Nuclear Reactor Regulation.

This denial may be appealed to the Commission's Executive Director for Operations within 30 days from the receipt of this letter. As provided in 10 CFR 9.11, any such appeal must be in writing, addressed to the Executive Director for Operations, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should clearly state on the envelope and in the letter that it is an "Appeal from an Initial FOIA Decision."

This completes NRC's action on your request.

Sincerely,

J. M. Felton, Director Division of Rules and Records

Office of Administration

Enclosure: Appendix A

Re: FOIA-84-262

APPENDIX A

NUREG-0892, Supplement 3, May 1983 - Accession No. 8306010043 1. 4/20/83 (PDR) (46 pages); Supplement 4, December 1983 - Accession No. 8401040036 (PDR) (142 pages); and Supplement 5, April 1984 -Accession No. 8404240005 (PDR) (41 pages) 2. NUREG-0881, Supplement 4, December 1983 - Accession No. 8401090322 (PDR) (91 pages) 3. NUREG-0857, Supplement 5, Appendix B - Accession No. 8312090366 (PDR) (61 pages) 4. NUREG-1002, November 1983, Section 3.11 - Accession No. 8312280092 (PDR) (100 pages) 5. 7/7/83 Meeting Summary prepared by Darl Hood 6. Memorandum for John F. Stolz from Vincent S. Noonan, 4/6/84

Subject: "Results of Electrical Equipment Environmental Qualification Audit for Three Mile Island - Unit 1"

J./R/A

J/R/A/ ASSOCIATES

Regulatory Information and Support Systems
1304 REDDITCH COURT, LARGO, MARYLAND 20772 (301) 336-4982

April 5, 1984 0GW-84-026

Mr. J.M. Felton, Director Division of Rules and Records Office of Administration U.S. Nuclear Regulatory Commission Washington, D.C. 20555 FREEDOM OF INFORMATION
ACT REQUEST

POIA -84-262 accid 4-9-84

Dear Mr. Felton:

Research of docket files at the Public Document Room of the NRC indicates that site audits were held of environmental qualification of equipment at the following plants:

- Waterford 3 (1/4 6/83)
- Midland 1 (1/11 13/83) 329
- Perry 1 (1983) 446
- Catawba 1 (3/84)4/3
- Zimmer 1 (1983) 250
- Palo Verde 1 (1983)
- Clinton 1 (1983) 46
- Wolf Creek (5/83)
- WNP 2 (1983)

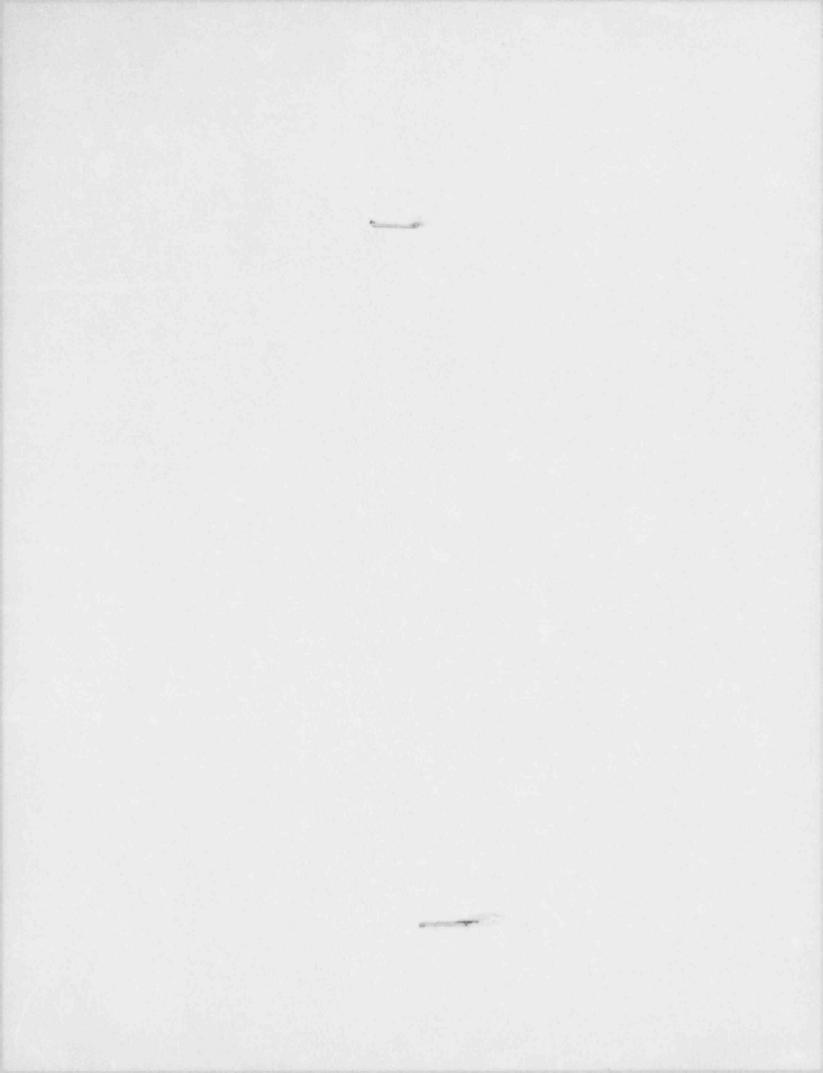
Pursuant to the Freedom of Information Act, I am herein requesting that reports resulting from those audits be placed in the Public Document Room. Additionally, I would like to view any and all reports from audits held since October 1982.

Thank you.

Oh ://

Ophelia G. Williams

850118 0051



POLITING AND	TRANSMITTAL SLIP	Date	8-3	
(Name, office symbol, building, Agency/Pos	room number,		Initials	Date
Chair				
Chus,				
				1
			TO GIVE	
				_
	File		te and Ret	
Action	For Clearance	1 1	Conversa	
Approval As Requested	For Correction		epare Repl	iy
Circulate	For Your Information	-	e Me	
Comment	Investigate	218	gnature	
Coordination	Justify		-	-
	tach the 2 (8-3 l	5 to	to)	
The dacus	or (8-31) the release	ne	el	
Pls at 84-21 The G dacus	orm as a RECORD of approved clearances, and similar act	als. co	el	
Pls at 84-21 The G dacus	orm as a RECORD of approve clearances, and similar act symbol, Agency/Post)	ca o	neurrance	No.

July 8, 1983

Docket Nos: 50-330

Mr. J. W. Cook Vice President Consumers Power Company 1945 Hest Parnall Road Jackson, Michigan 49201

Dear Mr. Cook:

Subject: Preliminary Safety Evaluation Report and Audit

Summary on Environmental Qualification of Electric Equipment Important to Safety and Safety-Related

Mechanical- Equipment for Midland Unit 2

The enclosed Safety Evaluation Report addresses the environmental qualification program for electrical equipment important to safety as defined in 10 CFR 50.49 for Midland Unit 2. This review was conducted in accordance with the requirements of 10 CFR 50.49 and the guidance in NUREG-0588 and Section 3.11 of NUREG-0800.

The staff requires that outstanding information identified in SER Sections 3.11.3 and 3.11.4 be provided as indicated in the conclusions of this preliminary SER. A principal objective of this transmittal is to provide for expeditious resolution of open items prior to inclusion of this matter in a future supplement. If Consumers deems a meeting or telephone conference necessary to discuss open items with the NRC staff, inform Melanie Miller, at (301) 492-4259, within one week of receipt of this letter. Your comments, including schedules for completion of any further analyses or other work associated with resolution of open items, are requested prior to any scheduled meeting or telephone conferences.

The enclosed preliminary SER, in part, describes the environmental qualification audit conducted by NRR and its consultant on January 11-13, 1983. Hence, the enclosure also serves as NRC's meeting summary for the January 11-13, 1983 audit.

Sincerely,

Thomas M. Novak, Assistant Director For Licensing Division of Licensing

DISTRIBUTION:

NRC PDR Local PDR

MMiller.

MDuncan

EAdensam

ACRS (16)

Attorney, OELD

JSniezak, I&E JStone. I&E

PRC System LB #4 r/f EAdensam DHood

Docket Nos. 50-330

Enclosure: As stated

cc: See next page

83\$719\$423 PDR

OFFICE DL:LB #4 DL:LB #4 LA:DL:LB #4 DC:LB #4 ADL:DL

SURNAME MMMAEP/hmc DHood W MDURCON EAdensom TNovak

DATE 6/27/83 6/27/83 6/27/83 6/27/83 76/C/83 76/19/83



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

July 8, 1983

Docket Nos: 50-330

Mr. J. W. Cook Vice President Consumers Power Company 1945 West Parnall Road Jackson, Michigan 49201

Dear Mr. Cook:

Subject: Preliminary Safety Evaluation Report and Audit

Summary on Environmental Qualification of Electric Equipment Important to Safety and Safety-Related

Mechanical Equipment for Midland Unit 2

The enclosed Safety Evaluation Report addresses the environmental qualification program for electrical equipment important to safety as defined in 10 CFR 50.49 for Midland Unit 2. This review was conducted in accordance with the requirements of 10 CFR 50.49 and the guidance in NUREG-0588 and Section 3.11 of NUREG-0800.

The staff requires that outstanding information identified in SER Sections 3.11.3 and 3.11.4 be provided as indicated in the conclusions of this preliminary SER. A principal objective of this transmittal is to provide for expeditious resolution of open items prior to inclusion of this matter in a future supplement. If Consumers deems a meeting or telephone conference necessary to discuss open items with the NRC staff, inform Melanie Miller, at (301) 492-4259, within one week of receipt of this letter. Your comments, including schedules for completion of any further analyses or other work associated with resolution of open items, are requested prior to any scheduled meeting or telephone conferences.

The enclosed preliminary SER, in part, describes the environmental qualification audit conducted by NRR and its consultant on January 11-13, 1983. Hence, the enclosure also serves as NRC's meeting summary for the January 11-13, 1983 audit.

Sincerely,

Thomas M. Novak, Assistant Director

For Licensing

Division of Licensing

Enclosure: As stated

cc: See next page

8307190423

Mr. J. W. Cook Vice President Consumers Power Company 1945 West Parnall Road Jackson, Michigan 49201

cc: Michael I. Miller, Esq.
Ronald G. Zamarin, Esq.
Alan S. Farnell, Esq.
Isham, Lincoln & Beale
Three First National Plaza,
51st floor
Chicago, Illinois 60602

James E. Brunner, Esq.
Consumers Power Company
212 West Michigan Avenue
Jackson, Michigan 49201

Ms. Mary Sinclair 5711 Summerset Drive Midland, Michigan 48640

Stewart H. Freeman Assistant Attorney General State of Michigan Environmental Protection Division 720 Law Building Lansing, Michigan 48913

Mr. Wendell Marshall Route 10 Midland, Michigan 48640

Mr. R. B. Borsum Nuclear Power Generation Division Babcock & Wilcox 7910 Woodmont Avenue, Suite 220 Bethesda, Maryland 20814

Cherry & Flynn Suite 3700 Three First National Plaza Chicago, Illinois 60602 Mr. Don van Farrowe, Chief Division of Radiological Health Department of Public Health P.O. Box 33035 Lansing, Michigan 48909

Mr. Steve Gadler 2120 Carter Avenue St. Paul, Minnesota 55108

UTS. Nuclear Regulatory Commission Resident Inspectors Office Route 7 Midland, Michigan 48640

Ms. Barbara Stamiris 5795 N. River Freeland, Michigan 48623

Mr. Paul A. Perry, Secretary Consumers Power Company 212 W. Michigan Avenue Jackson, Michigan 49201

Mr. Walt Apley c/o Mr. Max Clausen Battelle Pacific North West Labs (PNWL) Battelle Blvd. SIGMA IV Building Richland, Washington 99352

Mr. I. Charak, Manager NRC Assistance Project Argonne National Laboratory 9700 South Cass Avenue Argonne, Illinois 60439

James G. Keppler, Regional Administrator
U.S. Nuclear Regulatory Commission,
Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

cc: Mr. Ron Callen
Michigan Public Service Commission
6545 Mercantile Way
P.O. Box 30221
Lansing, Michigan 48909

Mr. Paul Rau Midland Daily News 124 McDonald Street Midland, Michigan 48640

Billie Pirner Garde
Director, Citizens Clinic
for Accountable Government
Government Accountability Project
Institute for Policy Studies
1901 Que Street, N.W.
Washington, D. C. 20009

Mr. Howard Levin, Project Manager TERA Corporation 7101 Wisconsin Avenue Bethesda, Maryland 20814

Ms. Lynne Bernabei Government Accountability Project 1901 Q Street, N.W. Washington, D. C. 20009



SAFETY EVALUATION REPORT OFFICE OF NUCLEAR REACTOR REGULATION

MIDLAND UNIT 2 DOCKET NO. 50-330

3.11 Environmental Qualification of Electric Equipment Important to Safety and Safety Related Mechanical Equipment

3.11.1 Introduction

Equipment which is used to perform a necessary safety function must be demonstrated to be capable of maintaining functional operability under all service conditions postulated to occur during its installed life for the time it is required to operate. This requirement, which is embodied in General Design Criteria 1 and 4 of Appendix A and Sections III, XI, and XVII of Appendix B to 10 CFR 50, is applicable to equipment located inside as well as outside containment. More detailed requirements and guidance relating to the methods and procedures for demonstrating this capability has been set forth in 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants," and NUREG-0588, "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment." This NUREG supplements - IEEE Standard 323, and various NRC Regulatory Guides and industry standards.

3.11.2 Background

NUREG-0588 was issued in December 1979 to promote a more orderly and systematic implementation of equipment qualification programs by industry and to provide guidance to the NRC staff for its use in ongoing licensing reviews. The positions contained in this report provide guidance on (1) how to establish environmental service conditions, (2) how to select methods which are

CRAFT

considered appropriate for qualifying equipment in different areas of the plant, and (3) other areas such as margin, aging, and documentation.

In February 1980 the NRC requested certain near term Operating License (OL) applicants to review and evaluate the environmental qualification documentation for each item of safety related electric equipment and to identify the degree to which their qualification programs comply with the staff positions discussed in NUREG-0588. IE Bulletin 79-01B "Environmental Qualification of Class IE Equipment," issued January 14, 1980, and its supplements dated February 29, September 30, and October 24, 1980 established environmental qualification requirements for operating reactors. This bulletin and its supplements were provided to OL applicants for consideration in their review.

A final rule on environmental qualification of electric equipment important to safety for nuclear power plants became effective on February 22, 1983. This rule, Section 50.49 of 10 CFR Part 50, specifies the requirements to be met for demonstrating the environmental qualification of electrical equipment important to safety located in a harsh environment. In accordance with 10 CFR 50.49, the electrical equipment in Midland Units 1 and 2 may be qualified in accordance with the acceptance criteria specified in Category II of NUREG-0588.

The qualification requirements for mechanical equipment are principally contained in Appendices A and B of 10 CFR 50. The qualification methods defined in NUREG-0588 can also be applied to mechanical equipment.

In order to document the degree to which their environmental qualification program complies with NRC's environmental qualification requirements and criteria, the applicant provided equipment qualification information by letters dated November 1981, April 30 and December 13, 1982, and February 17, 1983, to supplement the information contained in Section 3.11 of the FSAR.

3.11.2.1 Purpose

The purpose of this SER is to evaluate the adequacy of Midland Units 1 and 2 environmental qualification program for electric equipment important to



safety as defined in 10 CFR 50.49, and for safety-related mechanical equipment. The staff position relating to open items, as well as any unresolved issues, is provided in this report.

3.11.2.2 Scope

The scope of this report includes an evaluation of the completeness of the list of systems and equipment to be qualified, the criteria which they must meet, the environments in which they must function, and an assessment of the qualification documentation for the equipment. The principal area of review was the qualification of safety-related equipment which must function in order to prevent or mitigate the consequences of a loss-of-coolant accident (LOCA) or high energy line break (HELB) inside or outside of containment, while subjected to the harsh environments associated with these accidents.

3.11.3 Staff Evaluation

The staff evaluation of the applicant's environmental qualification program included an onsite examination of equipment, audits of qualification documentation, and a review of the applicant's submittals for completeness and acceptability of systems and components, qualification methods, and accident environments. The criteria described in NUREG-0800, Section 3.11, Rev. 2 and NUREG-0588 Category II form the basis for the staff evaluation of the adequacy of the applicant's qualification program. Revision 1 of NUREG-0588 was utilized to clarify staff positions as required.

The staff performed an audit of the applicant's qualification documentation and installed electrical equipment on January 11-13, 1983. The audit consisted of a review of 11 files containing equipment qualification documentation. The staff's finding during the audit are discussed in detail in Section 3.11.4.2. of this report.

3.11.3.1 Completeness of Equipment Important to Safety

The applicant was directed to (1) establish a list of systems and components located in a harsh environment that are required to prevent or mitigate a LOCA



or a HELB and (2) identify components needed to perform the function of safety-related display instrumentation, post-accident sampling and monitoring, and radiation monitoring.

The applicant's systems list and safety function(s) for the environmental qualification program was provided and is included as Appendix D of this report. However, the applicant must provide a comparison between each of the systems listed in Appendix D and those listed in Table 3.2.1 of the FSAR. Justification should be provided for the exclusion of safety-related system(s) in Table 3.2.1 from the environmental qualification program (e.g. not required for accident mitigation, all components located in a mild environment, etc.).

The applicant has identified the equipment required by NUREG-0737, "Clarification of TMI Action Plan Requirements", but the qualification status has not been established for all items. Prior to granting of an operating license, the applicant should identify the qualification status for all TMI Action Plan equipment. For any TMI Action Plan equipment not yet installed and that will not be installed prior to operation, a description of the plans for qualification, including the schedule for completion of qualification, must be submitted. (All TMI Action Plan equipment currently installed or that will be installed prior to operation must be qualified or justifications for interim operation provided prior to granting of an operating license, in accordance with 10 CFR 50.49.)

To comply with 10 CFR 50.49, the following information must be submitted by the applicant prior to granting of an operating license.

(a) A list of all nonsafety-related electrical equipment, located in a harsh environment, whose failure under postulated environmental conditions could prevent satisfactory accomplishment of safety functions by the safety-related equipment. A description of the methods used to identify this equipment must also be included. The nonsafety-related equipment identified must be included in the environmental qualification program.

- DRAT
- (b) A statement that all safety-related electrical equipment in a harsh environment, as defined in the scope of 10 CFR 50.49, is included in the list of equipment identified in the December 1982 submittal (the systems list discussed in Section 3.11.3.1 should be taken into consideration in this statement).
- (c) a list of all post-accident monitoring equipment currently installed, or that will be installed prior to plant operation, that is specified as Category 1 and 2 in Revision 2 of Regulatory Guide 1.97 and is located in a harsh environment. The equipment identified must be included in the environmental qualification program.

The Applicant's equipment qualification program does include the equipment required by Reg. Guide 1.97. However, all items installed or to be installed prior to the granting of an operating license must be qualified or justification for interim operation must be provided.

3.11.3.2 Qualification Methods

3.11.3.2.1 Electrical Equipment in a Harsh Environment

Detailed procedures for qualifying safety-related electrical equipment in a harsh environment are defined in NUREG-0588. The criteria in this NUREG are also applicable to other equipment important to safety defined in 10 CFR 50.49. Type testing of equipment in a sequence consisting of pre-aging, seismic and dynamic loading, and exposure to LOCA/HELB conditions (where applicable) is the preferred method of qualification. However, in some cases the applicant has extrapolated partial test data to establish the equipment qualification.

For several items of equipment in the containment, the applicant utilized thermal lag analysis for qualification to the short term, high temperatures of main steam line breaks. For tests already completed, NUREG-0588 permits analysis to supplement LOCA test data to demonstrate qualification for MSLB's. The staff has reviewed this analysis and finds the approach utilized to be acceptable.



3.11.3.2.2 Safety-Related Mechanical Equipment in a Harsh Environment

Although there are no detailed requirements for mechanical equipment, General Design Criteria 1, "Quality Standards and Records," and 4, "Environmental and Missile Design Bases;" Appendix B to 10 CFR 50, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants," Section III, "Design Control" and XVII, "Quality Assurance Records;" and SRP Section 3.11, Revision 2 contain the following requirements related to equipment qualification:

- Components shall be designed to be compatible with the postulated environmental conditions, including those associated with Loss-ofcoolant accidents.
- Measures shall be established for the selection and review for suitablility of application of materials, parts, and equipment that are essential to safety-related functions.
- Design control measures shall be established for verifying the adequacy of design.
- Equipment qualification records shall be maintained and shall include the results of tests and materials analyses.

The staff review is concentrated on materials which are sensitive to environmental effects, for example, seals, gaskets, lubricants, fluids for hydraulic systems, and diaphragms. An audit of qualification documentation was conducted by the staff, whereby the approach to qualification was found acceptable to verify conformance with the above criteria. However, prior to granting of an operating license the applicant should submit the results of the review performed for all safety-related mechanical equipment located in areas of potentially harsh environments, including a description of any problems that were identified and the corrective actions taken. Justifications for interim operation, based on the considerations specified in 10 CFR 50.49(i), must be provided for all equipment whose qualification cannot be established prior to granting of an operating license.

3.11.3.3 Service Conditions

NUREG-0588 defines the methods to be utilized for determining the environmental conditions associated with loss-of-coolant accidents or high energy line breaks, inside or outside containment. The review and evaluation of the adequacy of these environmental conditions are described below. The staff has reviewed the qualification documentation to ensure that the qualification conditions envelop the conditions established by the applicant.

3.11.3.3.1 Temperature, Pressure, and Humidity Conditions Inside Containment

The applicant provided the LOCA/MSLB profiles used for equipment qualification. The peak values resulting from these profiles are as follows:

	Maximum	Maximum	
	Temperature °F	Pressure, psig	Humidity, %
LOCA	320	70	100
MSLB	465	60	100

The staff has reviewed these profiles and finds them acceptable for use in equipment qualification; i.e., there is reasonable assurance that the actual pressures and temperatures will not exceed these profiles anywhere within the specified environmental zone (except in the break zone).

3.11.3.3.2 Temperature, Pressure, and Humidity Conditions Outside Containment

High-energy piping system failures are discussed in FSAR Section 3.6. The applicant has provided pressure temperature and humidity conditions associated with high energy line breaks outside containment. However, Table 1-6 of the equipment qualification (EQ) submittal shows a maximum temperature of 104°F under accident conditions for areas outside containment. Table 1-6 also indicates (in note 5) that maximum accident temperatures are under evaluation. In the EQ submittal, subsection 1.4.3.2.2 and Figures 1-28 through 1-34 shows a maximum accident temperature of 212°F that drops to 140°F after 27.7 hours.

Because of these inconsistencies, environmental conditions outside containment is and will remain an open item until the applicant can state specifically and consistently the various conditions.

The applicant stated that the maximum specified room temperature following a loss of HVAC are currently being verified. However, the applicant must identify the temperature environment base on a LOCA/HELB with a loss of offsite power for plant areas that are not served by Class 1E HVAC systems. The temperature rise effects from a loss of HVAC must be included in the environmental review for accident monitoring and post accident monitoring (AM/PAM) equipment required during and following a LOCA or HELB.

3.11.3.3.3 Submergence

The maximum submergence levels have been established by the applicant in the environmental qualification program. The maximum calculated water level inside containment established by the applicant is 603 ft. Equipment located below this level has been evaluated for submergence qualification. The effects of flooding on safety-related equipment outside containment are discussed in Sections 3.4 and 3.6 of the FSAR and in the applicant's environmental qualification program. The applicant stated that design changes were made subsequent to a flooding analysis conducted in 1977, and that a new flooding study is currently being undertaken to verify the results of the old study. The applicant also stated that flooding shall not prevent the ability to achieve and maintain a stable cold shutdown condition. Therefore, flooding is and shall remain unresolved pending the completion of the study currently being conducted by the applicant that must confirm that all equipment important to safety subjected to submergence is or will be qualified for submergence or for the time duration necessary for it to complete its safety function. This issue must be resolved prior to the granting of an operating license.

3.11.3.3.4 Chemical Spray

Chemical spray is available for containment heat removal following a design basis accident. The specified composition of the spray is 13000-14,130 PPM boric acid buffered with disodium phosphate with hydrazine added for iodine

removal, with a pH of 4.5-7.5. Equipment inside containment was reviewed for qualification under the above conditions.

3.11.3.3.5 Aging

.1

NUREG-0588, Category II delineates two aging-program requirements. Valve operators committed to IEEE Standard 382-1972 and motors committed to IEEE Standard 334-1971 must meet the Category I requirements of the NUREG. This requires the establishment of a qualified life, with maintenance and replacement schedules based on the findings. All other equipment must be subjected to an aging program which identifies aging-susceptible materials within the components.

In addition to the above, a maintenance/surveillance program should be implemented to identify and prevent significant age-related degradation in electrical and mechanical equipment. The applicant has committed to follow the recommendations in Regulatory Guide 1.33, Revision 2, "Quality Assurance Program Requirements (Operation)," which endorses American National Standard ANS-3.2/ANSI N18.7-1976, "Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants," as noted in Section 17 of the staff SER for Midland Plant Units 1 and 2 (NUREG-0793). This standard defines the scope and content of a maintenance/surveillance program for safety-related equipment. Provisions for preventing or detecting age-related degradation in safety-grade equipment are specified and include a) utilizing experience with similar equipment, b) revising and updating the program as experience is gained with the equipment during the life of the plant, c) reviewing and evaluating malfunctioning equipment and obtaining adequate replacement components, and d) establishing surveillance tests and inspections based on reliability analyses, frequency and type of service, or age of the items, as appropriate. A commitment to implementation prior to granting of an operating license must be made by the applicant and will be verified by the staff. The outline of this program is acceptable for the purposes of the environmental qualification program.

3.11.3.3.6 Radiation (Inside and Outside Containment)

The applicant has provided values for the radiation levels postulated to exist following a LOCA. The application and methodology employed to determine these values were presented to the applicant in NUREG-0588 and NUREG-0737, "Clarification of TMI Action Plan Requirements." The staff review determined that the values to which equipment was qualified enveloped the requirements identified by the applicant.

: 2

The values specified for use in equipment qualification in the containment are total integrated doses (gamma and beta) ranging from 6.6×10^5 to 2.9×10^8 rads. In the auxiliary building, total integrated doses of up to 1.96×10^8 rads were specified. The applicant provided sample calculations that were reviewed by the staff and found acceptable. Consequently, the values presented here are acceptable for use in the qualification of equipment.

3.11.3.4 Outstanding Equipment

For all items not having complete qualification documentation, the applicant must provide commitments for corrective action and schedules for completion. For items that will not have full qualification prior to granting an operating license, an analysis must be performed in accordance with the requirements of Paragraph (i) of 10 CFR 50.49 to ensure that the plant can be operated safely pending completion of environmental qualification. The analyses must be submitted for staff review and approval prior to the granting of an operating license.

3.11.4 Qualification of Equipment

The following subsections present the staff's assessment of equipment based on the applicant's submittal, audits of documentation at the plant site, information in the NRC Equipment Qualification Data Bank, and previous staff evaluations of equipment in other plants.



3.11.4.1 Electrical Equipment in a Harsh Environment

The staff has separated the electrical equipment in a harsh environment into three categories: (1) equipment demonstrated to be not qualified, (2) equipment requiring additional qualification information or corrective action, and (3) equipment considered acceptable pending implementation of the maintenance and surveillance program. An appendix listing equipment in each of these categories is provided.

3.11.4.1.1 Equipment Demonstrated to be Not Qualified

Appendix A identified equipment which has been demonstrated to be not qualified. There is no equipment in this category for Midland Units-1 and 2.

3.11.4.1.2 Equipment Requiring Additional Information and/or Corrective Action

Appendix B identifies equipment in this category. Corrective action or deficiencies are noted by a letter relating to the legend identified below.

Legend

A - material-aging evaluation; replacement schedule; ongoing equipment surveillance

CS - chemical spray

EXN - exempted equipment justification inadequate

H - humidity

I - HELB evaluation outside containment not completed

M - margin

P - pressure

QI - qualification information being developed

QM - qualification method

OT - qualification time

R - radiation

RPS - equipment relocation or replacement schedule provided

RPN - equipment relocation or replacement schedule not provided



RTS - retest, schedule provided

S - submergence

SEN - separate effects qualification justification inadequate

T - temperature

QC - qualification criteria (Category II in lieu of Category I)

These deficiencies do not necessarily mean that the equipment is unqualified. However, the deficiencies are cause for concern and require further case-by-case evaluation. The applicant should resolve these deficiencies and document the resolutions in an auditable form.

3.11.4.1.3 Equipment Considered Acceptable or Conditionally Acceptable

Based on the staff review, the items identified in Appendix C have been determined to be acceptable, pending implementation of the maintenance/surveillance program.

3.11.4.2 Environmental Qualification Audit

The staff, together with its consultant EG&G of Idaho. performed an audit of the applicant's qualification documentation on January 11-13, 1983. The audit consisted of a review of 11 files containing information regarding equipment qualification. In general the staff was in agreement with the findings by the applicant regarding the qualification status of the equipment. Based on the review of the files, the following observations and comments were made by the staff:

- (1) The staff agreed to accelerate review efforts on the thermal equivalence analysis provided to the staff during the audit.
- (2) The staff also agreed to complete the review of the applicant's chemical spray analysis at the earliest possible date.

- URAST
- (3) Applicant committed to update all files to include NSSS test reports or test report summaries as appropriate.
- (4) Generic writeups must be revised or an explanation should be provided in test reports for all equipment that has the potential for exposure to a high energy line break (HELB) and are not required for that event. In addition, Table 1-6 must be revised to list the appropriate curve(s) for all equipment which are required for a HELB.
 - (5) The staff suggested that, upon completion of the field verification report (currently in draft form), the applicant should incorporate copies of the applicable pages into the corresponding EQ files.
- (6) The staff noted that discussions on maintenance restrictions on qualification has been centrally located. This should help to avoid confusion that could develop in the absence of these centrally located files.
- (7) As stated above, 11 files were audited, however, it was determined that one file, (M1.35C) RTD and Connector Head, should be removed from the harsh environment qualification program and that the applicant's EQ submittal should be updated to reflect this change. The applicant must document that this equipment is not required to perform a safety function during or after any design basis accident, and also must determine that the failure of this equipment will have no effect on other safety-related equipment, nor mislead the operator.

The following comments are specific to individual files as indicated. However, the applicant should update all files to incorporate these comments where applicable.

(M-54A) Reactor Building Spray Pump Motors

(a) Add surveillance requirement for the motor windings based on the humidity requirement.

- (b) Reference for humidity qualification of lubricant should be revised from reference 4 to the appropriate reference in the file (from ref. 4 to ref. 6).
- (c) Files should be revised to indicate that the seals are metallic.
- (d) Files should be updated to indicate that reference 7 curves are the appropriate curves instead of Reference 2 curves.
- (e) There should be an explanation regarding the exemption from the HELB environment for this equipment.

(M-163A) Recirculation Air Cooling Unit Motors .

Three failures are listed as unknown (Reference 2, Appendix D, page 1). A copy of the telex from the vendor, stating that the three failures are not related to problems with the test specimen(s) but are the result of other causes, should be added to the files.

(J-255A) Valve Operators

The applicant has committed to revise the information on the Equipment Environmental Qualification Sheet (EEQS) to conform to the test documentation (i.e., model number, temperature, pressure). The applicant has also agreed to revise the EQ submittal to reflect these changes.

(F-002A) Terminal Block (States)

The applicant has stated that the location of this component(s) in a harsh environment cannot "presently" be determined. This will remain an open item until the location and correct installation is verified by the NRC resident inspector, or until the applicant certifies that this component(s) is not installed in a harsh environment.

E-020A Electrical Penetration Assembly (Amphenol)

Prior to accepting this file as qualified, the applicant should review the applicability of IE Bulletin 82-04 and incorporate the resolution in the EQ files.

In addition to the six files noted above the following were also audited.

(E22A) Low Voltage Power Cable

(E26B) 600V Control Cable Rework

(J-249A) Acoustic Flow Monitor

(M1.35A) Differential Pressure Transmitters

(E120A) Conax Penetration Modification -

3.11.5 Conclusions

The staff has reviewed the Midland Unit 1 and 2 program for the environmental qualification of electric equipment important to safety and safety related mechanical equipment as noted in this report. The purpose of the review was to determine the adequacy of the program including the systems selected for qualification, the environmental conditions resulting from design basis accidents, and the methods used to demonstrate qualification.

The results of the review have been detailed in this report. The applicant must provide the required information, identified in Sections 3.11.3.1, 3.11.3.2.2, 3.11.3.3.2, 3.11.3.3.3, 3.11.3.3.5, and 3.11.3.4 of this report, to demonstrate full compliance with 10 CFR 50.49 and all applicable regulations. The qualification information should be provided to allow sufficient time for staff review and approval prior to issuance of an operating license.

Upon review of the required information a further supplement to this SSER will be issued.



APPENDIX A EQUIPMENT DEMONSTRATED TO BE NOT QUALIFIED (Category 3.11.4.1.1)

No equipment in this category



APPENDIX B EQUIPMENT REQUIRING ADDITATION INFORMATION OR CORRECTIVE ACTION (Category 3.11.4.1.2)

EQUIPMENT ITEM	MANUFACTURER	MODEL NO. C	DEFICIENCY/ ORRECTIVE ACTION
600 Volt Control Cable (E026A)	Rockbestos	-053-3098	A,M
600 Volt Control Cable (E026A)	Rockbestos _	C51-3035	Α,Μ
500 Volt Control Cable (E026A)	Rockbestos	C53-3099	A,M
600 Volt Control Cable (E026A)	Rockbestos	C53-3100	A,M
500 Volt Control Cable (E026A)	Rockbestos	C53-3101	A,M
600 Volt Control Cable (E026A)	Rockbestos	053-3102	A,M
500 Volt Control Cable (E026A)	Rockbestos	C51-3034	A,M
600 Volt Control Cable (E026A)	Rockbestos	051-3036	A,M
500 Voit Control Cable Insula- tion Rework KXL-760 (E026B)	Rockbestos	EQ Rework	Α,Μ
500 Volt Control Cable Insula- tion Pework KXL-420 (E026C)	Rockbestos	Rework	A,M
Coaxial, Triaxial, and Twinaxial Caple (E055A)	Rockbestos	Rss-5-112	A,CS,T
(E055A)	Rockbestos	Rss-6-100	4,CS,T
(ESB6A)	Rockbestos	Rss-6-109	A,CS,T
800V Instrumentation			
Cable (ECSCA)	Rockbestos	051-3125	A,M
lat'e (E050A)	Rockbestos	053-3289	A,M
Cable (EUSCA)	Rockbestos	C51-3125	A,M
Caple (E060A)	Packbestos	053-3288	A,M
Cable (E060A)	Rockbestos	053-3290	4,M
Capie (EDECA)	Rockbestos	146-3698	4,8
Capte (EDECA)	Focktestos	146-3699	±, 4
Cable (EDSCA)	Rockbestos	145-9700	4,%
Caple (EDEOA)	Pockbestos	146-3701	A,M



EQUIPMENT ITEM	MANUFACTURER	MODEL NO. C	DEFICIENCY/ CRRECTIVE ACTION
Cable (E060A)	Rockbestos	146-3702	Α,Μ
Cable (E060A)	Rockbestos	146-3703	A,M
Cable (E060A)	Rockbestos	145-3707	A,M
Cable (E060A)	Rockbestos	146-3704	A,M
Cable (E060A)	Rockbestos	146-3705	A,M
Cable (E050A)	Rockbestos	146-3706	A,M
Cable (E060A)	Rockbestos	167-3167	A,M
Cable (E060A)	Rockbestos	167-3168	A.M
Cable (E060A)	Rockbestos	167-3169	A,M
Cable (EO60A)	Rockbestos	146-3709	A,M
able (E060A)	Rockbestos	167-3170	A,M
Cable (E060A)	Rockbestos	167-3185	A,M
Cable (EC6CA)	Rockbestos	C53-3291	A,M
Cable (E060A)	Rockbestos	146-3708	A,M
Cable (E060A)	Rockbestos	146-5050	Α,Μ
Insulation Rework For Instru- mentation Cable KXL 760 (E060B)	Rockbestos	Rework	A,M
Insulation Rework For Instru- mentation Cable KXL-420 (E060C)	Rockbestos	Rework	A,M
Pesistance Temperature De- detector Assemblies (J233A)	Weed Instrument Co.	512-180-0-5 0-22.75-0-0	
Pasistance Temperature De- tetector Assemblies (J233A)	heed Instrument Co.	512-180-0-6 0-20-0-0	
Resistance Temperature De- tetector Assemblies (U233A)	Weed Instrument Co.	512-18-0-3- 0-12.8-0-0	QI
Resistance Temperature De- tetector Assemblies (U233A)	Weed Instrument Co.	512-8-0-3- 0-12.5-0-0	QI

EQUIPMENT ITEM	MANUFACTURER	MODEL NO.	DEFICIENCY/ CORRECTIVE ACTION
Radiation Monitoring System (J244A)	Victoreen	877-1	QI
Cable/Connector Assembly (J244B)	Victoreen	878-1	RPN
Flow Transmitter (J245A)	Rosemount	1153004	QI
Lével Transmitter (J245A)	Rosemount	1153HD5	QI I
Level Transmitter (J245A)	Rosemount	1153005	QI
Level Transmitter (J245A)	Rosemount	1153004	QI
Differential Pressure XMTR (3245A)	Rosemount	1153007	QI
Pressure Transmitter (3245A)	Rosemount -	· 1153GD9	QI
Pressure Transmitter (J245A)	Rosemount	1153AD6	01
Pressure Transmitter (J245A)	Rosemount	1153AD7	QI
Solenoid Valve (d255A)	Target Rock Corp.	768-006	01
Solenoid Valve (J256A)	Target Rock Corp.		QI
Solenoid Valve (J256A)	Target Rock Corp.		QI
Solenoid Valve (J256A)	Target Rock Corp.		QI
Scienoid Valve (J256A)	Target Rock Corp.	768-058	QI
Splenoid Valve (J256A)	Target Rock Corp.	768-016	QI
Solenoid Valve (J255A)	Target Rock Corp.	768-013	QI
Solenoid Valve (J256A)	Target Rock Corp.	758-019	01
Solenoid Valve (J256A)	Target Rock Corp.	765-050	QI
Solenoid Valve (J256A)	Target Rock Corp.		QI .
Sclenoid Valve (0256A)	Target Rock Corp.	758-056	QI
Solenoid Valve (C255A)	Target Rock Corp.	788-005	ÇI
Solenoid Valve (U256A)	Target Rock Corp.		QI
Solehofd /elle (U255A)	Tanget Rock Corp.		QI.
Sclencia Value (J255A)	Target Rock Corp.		QI
Position Switch (J255A)	Gordos	Seed	QI
Splenoid Valve (J255A)	Target Rock Corp.	768-057	ŞĪ
Level Switch (J278A)	Vitro	3119-1112	
Level Switch (3278A)	Dela.al-Gens	LS-55490	175
Lavel Monitor Probe (U297A)	Delaval-Dams	27-14154	21

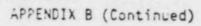


EQUIPMENT ITEM	MANUFACTURER	MODEL NO. CO	DEFICIENCY/ RRECTIVE ACTION
Level Monitor Probe (J297A)	Delaval-Gems	XM-54852/53	QI
Valve Operator (L-DC-IN)	Limitorque	\$8-0-15-1900	
Valve Operator (L-DC-OUT)	Limitorque	SMB-0-25-190	
Temperature Thermostat (M349A)	Pen Controls	A19 B4C6	RPS
Fission Chamber (JC-100A)	Gamma-Metrics	RCS 400A	QI
PAM Connector (JC-100B)	Ampheno1	52975-1051	01
PAM Connector (JC-100B)	Ampheno1	53250	QI
	Gamma-Metrics -		Qi . :
Position Switch (M1.11A)	Target Rock Corp.	81CC-001	
			QI
PCRV (M1.11A)	Target Rock Corp.	8100-001	QI
Connection Kit (E-Ray A)	RAYCHEM	NMCK (Motor Conn Kit)	A,M
Heat Shrink Tubing (E-Ray B)	RAYCHEM	WCSF-N	A,T,M
Pressurizer Heaters (M1.10A)	Weigand	14KW- 1100786-0	QI
Fir Heater Connector Assy. (M1.10B)	Gulton	1120150D	QI
480V AC Fower Panel (E45A)	Square D Company	Type HCN NEMA 12	01
Valve Operator (J255A)	ITT General Controls	";+91	A,T,M
Valve Spenator (J255A)	ITT General Controls	:-93	A,T,M
Motor (Punc) (MOSEA)	Westinghouse	SBOP	RPN
Chiller (Aux. Blog. Safeguards) (*1245A)			
Local Control Station (EDS4A)			RPN
Lisal Control Station (ECS4A)	Soula/Brown Soveria	Vendor Print 7220-E34-10-	
Schenoid Valve (M118B)	Keane	13253-115-00	DO-E QI
Praition Switch (MIISER)	NAMED Controls	EA-740-80100	O RPS
	NAMCO Controls		

EQUIPMENT ITEM	MANUFACTURER	MODEL NO. CORR	EFICIENCY/ ECTIVE ACTION
Position Switch (123CD)	NAMCO Controls	EA 180-11302	RPN
Position Switch (123CD)	NAMCO Controls	EA 180-12302	RPN
Solenoin Valve (M125CC)	ASCO	8302D26RU	QM,T
Limit Switch (1250D)	Honeywell	DTF2-2RN	RPS
Fan Motor (M149A)	Westinghouse	Type LLT/ 143T/.75/17	RPS
Fan Motor (M149A)	Westinghouse	Type LLT/ 182T/3/1725	RPS
Fan Motor (M149A)	Westinghouse	Type LLT/ 184T/5/1738	RPS
Fen Motor (M149A)	Westinghouse	Type LLT/	RPS
		184T/5/1735	
Air Cooling Unit (M163A)	Joy Mfg.	P/N 600287-47	RPS
Pump Motor (Hydrazine) (M374A)	Reliance Electric	Type PB, Frame 182T	QM, T
Pump Motor (M1.16A)	General Electric	5K83C947A7	A,T
Pump Motor (Decay Heat Removal) (M1.17A)	General Electric	5k811055444	A,T
Neutron Flux Element (M1.31A)	Westinghouse	WL 23636B	M, A
Neutron Flux Connector (M1.31B)	Ampheno1	52975-1051	M,A,T
RTD (M1.35C)*	Rosemount	177Hw/79-235	
In-Come Thermoscuple (M1.35D)	SELF4B		QI
In-Core Thermosouple Connector (MISSE)	Cannon		QI
Electric Conductor Seal (3:32A)	Conax	N-11003-09	A.T,QI
E'ectric Conquetor Seal (3038A)	Conex	N-11008-11	A.T.QI
Electric Opholotor Seal (30334)	Conax	N-11003-39	A,T,QI
Codel Control Station (E034A)	Gould, Erown Boverfa		
Local Control Station (E334A)	Gould/Brown Baleria		RPN
Terminal Block (FOCCA)**	Multi-AMP, States Div.		
Terminal Block (FCC2B)	General Electric	04151	ŞĪ

.,

EQUIPMENT ITEM	MANUFACTURER	MODEL NO. CORR	EFICIENCY/
Terminal Block (F002B)	General Electric	CR2960	01
Valve Operator (L-AC-IN)	Limitorque	SMS-00-5-3600	QI
Valve Operator (L-AC-IN)	Limitorque	SMB-00-5-1800	QI
Valve Operator (L-AC-IN)	Limitorque	SMB-000-5-1800	
Valve Operator (L-AC-IN)	Limitorque	SB-3-80-1800	QI
Valve Operator (L-AC-IN)	Limitorque	SB-3-80-3600	QI
Valve Operator (L-AC-IN)	Limitorque	SBD-3-175-3500	
Valve Operator (L-AC-IN)	Limitorque	· 5M8-0-25-1800	QI
Valve Operator (L-AC-IN)	Limitorque	SMB-0-25-3600	QI
Valve Operator (L-AC-IN)	Limitorque	SMB-00-25-1800	OI
Valve Operator (L-AC-IN)	Limitorque	SE-00-15-3600	QI
Valve Operator (L-AC-IN)	Limitorque	SMB-000-2-1800	QI
Valve Operator (L-AC-IN)	· Limitorque	SMB-0-40-1700/ H48C	QI
Valve Operator (L-AC-IN)	Limitorque	SME-0-40-1700/ H4BC	QI
Valve Operator (L-AC-IN)	Limitorque	SMB-0-15-3400 H3BC	QI
Valve Operator (L-AC-OUT)	Limitorque	SMB-000/2 HIBC	QI
Valve Operator (L-AC-OUT)	Limitorque	58-0-40-1900	QI
Valve Operator (L-AC-OUT)	Limitorque	SM8-0-40-1900	QI
Valve Operator (L-AC-OUT)	Limitarque	SE-005	QI
Value Operator (L-40-00T)	Linitarque	SMB-00	21
.el.e Cperator (L-AC-DUT)	l'eftencie	\$ 8-00-15	01
Value Operator (L-AC-DUT)	. Tittingue	SME-0-05	QI
Value Operator (L-AC-DUT)	Linitanque	18-0-25	ÇI
Value Sperator (1-40-007)	Limitanque	5118-0-15-3600	01
.ei.e Operator (L-40-OUT)	linitarque	578-8-100-8600	QI
Value Operator (L-40-00T)	L'm'iorque	18-2-40-1600	ÇĪ
Value Stemator (L-40-OuT)	l'aftenque	378-00-10-1800	01



EQUIPMENT ITEM	MANUFACTURER	MODEL NO. CORRECTIVE ACTION	
Valve Operator (L-AC-OUT)	Limitorque	SME-000	QI
Valve Operator (L-AC-OUT)	Limitorque	SM3-00-10	QI
Valve Operator (L-AC-OUT)	Limitorque:	SM3-00-2-1500	QI

Fact Note: ATAIS companent is not required for hemon environment program.

ATAPPLICANT cannot locate this component inside containment.

APPENDIX C EQUIPMENT CONSIDERED ACCEPTABLE OR CONDITIONALLY ACCEPTABLE (Category 3:11.4.1.3)

				DEFICIENCY,	
	EQUIPMENT ITEM	MANUFACTURER	MODEL NO.	CORRECTIVE ACTION	
				T C. C.	
	Electrical Penetration (E020A)	Ampheno1	Low V 12 IPS EPA	A	
	Electrical Penetration (E020A)	Ampheno1	Medium V 25 IPS EPA	A	
	600 Volt Power Cable (E022A)	Essex International	33373-06	A	
	500 Volt Power Cable (E022A)	Essex International	33373-02	A :	
•	500 Volt Power Cable (E022A)	Essex International	33373-52	A	
	600 Volt Power Cable (E022A)	Essex International	33373-59	A	
	500 Volt Fower Cable (E022A)	Essex International	33372-02	A	
	500 Voit Power Cable (E022A)	Essex International	33372-06	A	
	600 Volt Power Cable (E022A)	Essex International	33371-54	A	
	Low Voltage Power Cable (Rework) (E022B)	Essex International	Rework	Α	
	Flow Transmitter (J245B)	Rosemount	1152095	A	
	Pressure Transmitter (J245B)	Rosemount	1152GP6412PB	A	
	Flow Transmitter (J245B)	Rosemount	11520P4	A	
	Pressure Transmitter (J2458)	Rosemount	1152GP6	A	
	Flow Transmitter (U2450)	Rosemount	1153085	A	
	Ficw Transmitter (J245C)	Rosemount	1153084	A	
	Cable Assembly (J249A)	Endevco/Tec	2273-02	A	
	Fich Senson (CC49A)	Ensevoo/Tec	2273 5"1	A	
	Fice Transmitter (J249A)	Endevco/Tec	504A	A	
	Puro Motor (M018A)	Stemens-Allis, Inc.	PG-4047-8	A	
	Pump Motor (MCS4A)	Allis Chalmens	Type RG, Frame 445TS	S A	
	Valve Operator (M12300)	Rotork Inc.		A	
	Electric H, Recombiner (M1834)	Westinghouse		A	
	Flow Transfitter (M2.284)	Eafley Controls		4	
	Pressure Transmitter (M1.388)	Eafley Cartrols		4	

J. Hr.		
DEFICIENCY	/	

EQUIPMENT ITEM	MANUFACTURER	MODEL NO.	CORRECTIVE ACTION
Electric Penetration Modification (E120A)	Conax	7769-10000-03	A
Electric Penetration Modification (E120A)	Conax	7769-10000-04	A
Electric Penetration Modification (E120A)	Conax	7769-10000-05	A
Electric Penetration Modification (El2DA)	Conax	7769-10000-06	A
Electric Fenetration Modification (El20A)	Conex	7769-10000-07	. A
Electric Penetration Modification (120A)	Conax	7769-10000-08	A
Electric Penetration Modification (El2DA)	Conax	7769-10000-09	А
Electric Penetration Modification (El20A)	Conax	7769-10000-10	А
Electric Penetration Modification (E120A)	Conax	7769-10000-11	A
Electric Penetration Modification (E120A)	Conax	7769-10000-12	Α
Electric Penetration Modification (El20A)	Conax	7769-10000-13	Α .
Electric Penetration Mdiffication (El20A)	Conax	7769-10000-01	А
Electric Paretration Modification (Election)	Conax	7769-10000-02	А
#00V Wire 1/0 #8 (F001)	Araconda-Enfosson	8 133 NSIS-X500 Gray	A
800V Wire 1/0 #6 (F001)	Anaconda-Enfosson	6 133 NSIS-X600 Gray	



APPENDIX D

SAFETY-RELATED SYSTEMS WITH COMPONENTS WHICH PERFORM SAFETY FUNCTIONS

	Function	System (Startup System)				
1.	Emergency	Reactor Coolent and Pressure Control (68)				
	Reactor	Decay Heat Removal (BC)				
	Shutdown	Makeup, Purification, and Reactor Coolant Addition (EG) Sorated Water Storage (BN)				
		Unit Logic Cabinet C34 and Associated Window Arrays (RK)				
		Engineered Safeguards Actuation (SA)				
		Reactor Protection (SB)				
		Neutron Monitoring (SE)				
		Reactor Control (SF)				
		Safety Parameter Display Panel (SO)				
2.	Containment	Main Steam (AB)				
	Isolation	Feedwater (AE)				
		Decay Heat Removal (BC)				
		Makeup Purification, and Reactor Coolant Addition (BG)				
		Primary Water (BL)				
		Steam Generator Recirculation (EM)				
		Reactor Building Penetration Pressurization (BT)				
		Fuel Pool Cocling and Puriffication (EC)				
		Component Cooling Water (EG)				
		Reactor Building Combustible Gas Control (GS)				
		Reactor Building Air Purification, Clearus, and Ventila-				
		tion (GT)				
		Radwaste Gas (HA)				
		Beron Recovery (HE)				
		Engineered Safeguards Actuation (SA)				
		Phant Radiation Monitoring (SD)				

F				-			
	41.00	-	-	•	-	-	-

System (Startup System)

Reactor

Main Steam (AB)

Core

Auxiliary Feedwater (AL)

Cooling

Condensate Storage and Transfer (AP)

Reactor Coolant and Pressure Control (EB)

Decay Heat Removal (BC)

Makeup, Purification, and Reactor Coolant Addition (BG)

Core Flooding (BH)

Borated Water Storage (BN)

Component Cooling Water (EG)

Auxiliary Feedwater Pump Turbines (FC)

Engineered Safeguards Actuation (SA)

Reactor Protection (SB)

Safety Parameter Display Panel (SO)

4. Containment

Removal

Decay Heat Removal (BC)

Heat

Reactor Building Spray (BK)

Ecrated Water Storage (EN)

Service Water (EA)

Component Cooling Water (EG)

Reactor Building Heat Removal (GN)

Engineered Safeguards Actuation (SA)

5. Core

Demineralized Water Storage and Transfer (AN)

Residual

Decay Heat Pemoval (BC)

Heat Removal

Makeup, Purification, and Reactor Coolant Accition (BG)

Sarvice Water (EA)

Component Cooling Water (E3)

Safety Parameter Ofsplay Panel (50)

Release



6. Prevent

Radioactivity

Decay Heat Removal (BC)

Reactor Building Spray (BK)

Borated Water Storage (BN)

Reactor Building Penetration Pressurization (BT)

Fuel Pool Cooling and Purification (EC)

Reactor Coolant and Pressure Control (BB)

Auxiliary Building HVAC Including Emergency Spent Fuel

Pool Exhaust (GL)

Radwaste Gas (HA)

Unit Logic Cabinet C34 and Associated Window

Arrays (RK)

Engineered Safeguards Actuation (SA)

Plant Radiation Monitoring (SD)

Radioactive Liquid Sampling and Analysis (SJ)

Safety Parameter Display Panel (SO)

7. Support

Systems

Service Water Traveling Screens and Screen Wash (DC)

Service Water (EA)

Component Cooling Water (EG)

Central Chilled Water (GB)

Service Water Pump Structure HVAC (GD)

Safeguards Equipment Chilled Water (GJ)

Auxiliary Building HVAC Including Control Room (GL)

Diesel Generator Euilding FVAC (3M)

Standby Diesel Fuel Storage (UE)

Fuel Handling and Storage (KE)

Station Power Transformers and Auxiliaries (MA)

Lower Medium Voltage (4.18kV ron-Class 1E Power) (NB)

Miscellaneous Cable and Electrical Peretrations (NA)

Lover Medfum Voltage (4.15kV Class IE Power) (15)

Standby Generation (Diesel Generators) (PE)

tik Voltage Unit Substation (480V Class 18 7: 48) (23)

Low Voltage MCC (480V Class 15 Forem) (PH)

125.ds Cartral Paker (PK)

-	Function	System (Startup System)					
7.	Support	120Vac Preferred Power Panels (PN)					
	Systems	120Vac Control and Instrument Power Panels (PP)					
	(cont.)	Distribution Panels (480Vac) Unit (PR)					
		Process Instrumentation (RL)					
		Neutron Monitoring (SE)					
		Reactor Control (SF)					
		Miscellaneous Consoles and Systems (SO)					
		Miscellaneous Mechanical Equipment (ZZ)					
		Miscellaneous Electrical Equipment (NA)					
		Miscellaneous Structural Items (NA)					
		# (2014년 B 1988년 B 1984년 11 - 198					