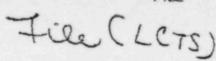
### ATTACHMENT 5





# MISSISSIPPI POWER & LIGHT COMPANY

P. O. BOX 1840, JACKSON, MISSISSIPPI 39205 January 10, 1983

#### PRICELEAR PRODUCTION DEPARTMENT

Mr. J. P. O'Reilly, Regional Administrator Office of Inspection & Enforcement Region II 101 Marietta Struat, N. W. Suite 3100 Atlanta, Georgia 30373

Dear Mr. O'Beilly:

SUBJECT: Grand Gulf Nuclear Station Units 1 & 2 Docket Nos. 50-416 and 50-417 License No. NPF-13 File 0260/15180 Technical Specification Interpretation AECM-83/015

In order to carry out a needed maintenance operation, it is necessary to enter Operational Condition 5 as a result of detensioning the reactor head closure holts. Technical Specification 3/4.3.6 requires that the Source Hange Monitor "detector not full in" interlock be operable (See Table 3.3.6-1). At this time, the detectors are full in, but the drive motor mechanisms (including the interlocks) have been removed for maintenance (which is acceptable in Operational Condition 4). The safety function of the interlock is to generate a rod block when the detectors are not full in. This safety function is satisfied even with the interlock hardware removed since the required rod block is also generated by the absence of the interlock hardware.

It is our interpretation of the Technical Specificatione that operability of the interlock is satisfactorily demonstrated by the presence of the rod block which will allow entry into Operational Condition 5 and Specification 3.0.4 to be met. Technical Specification Action statements for 3/4.3.6 will be met during any future work required to be performed on the datector drive mechanism once in Operational Condition 5. As discussed in a telephone conversation on January 10, 1983, with Messrs. Dean Bouston and Bob Rottimore of the Office of Suclear Reactor Regulation, Mr. Floyd Cantrell of the Region II Office of Inspection and Enforcement, and Mr. Al Wagner Suclear Regulatory Commission Senior Resident Inspector, it is requested that you formally endorse this interpretation of the Technical Specifications. In order to address concerns by members of your staff during the telephone conversation, the source range detectors will be verified full in at a minimism of once per day.

AECM-83/015 Page 2

#### MISSISSIPPI POWER & LIGHT COMPANY

Mississippi Fower & Light intends to apply for a permanent change to the Operating License to preclude such problems from occurring in the future.

Yours truly,

L. F. Dale

Manager of Ruclear Services

#### JOF/SHE/JDR: SAP

cc: Mr. N. L. Stampley

Mr. R. B. McGebee

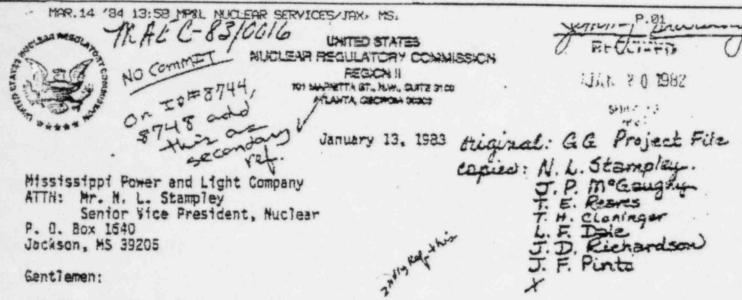
Mr. T. B. Conner

Mr. G. B. Taylor

Mr. Richard C. DeYoung, Director Office of Inspection & Enforcement U. S. Ruclear Regulatory Commission Washington, D. C. 20555

Br. Ployd Cantrell Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission Region II 101 Marietta St., N.W., Suite 3100 Atlanta, Georgia 30303

Fr. Dave Verelli
Office of Inspection and Enforcement
D. S. Nuclear Regulatory Commission
Region II
101 Marietta St., N.W., Suite 3100
Atlanta, Georgia 30303



SUBJECT: GRAND GULF NUCLEAR STATION UNITS 1 AND 2, REQUEST FOR TECHNICAL SPECIFICATION INTERPRETATION AECM-83/015

In your letter on the same subject dated January 10, 1983, you provided a basis for a change to the Technical Specifications at Grand Gulf Muclear Station Units 1 and 2. This was also discussed with you in a conference call on January 10, 1983.

Spac fically, and consistent with our conference call mentioned above, your let ar of January 10, 1983 requests that your Technical Specifications for Units 1 and 2 at Grand Gulf be interpreted such that operability of the Source Gange Monitor "detector not full in" interlock be satisfactorily demonstrated by the presence of the rod block. The last paragraph of your letter states your commitment to apply for a permanent change to your Operating License in the near future.

Based on the above and on independent review, Region II hereby concurs in your interim interpretation of Technical Specifications for Grand Gulf as described in your letter dated January 10, 1983. We consider that this is a one-time interpretation that will be corrected when your permanent change to the Technical Specifications is requested and accepted.

Stncerely.

R. C. Lewis, Director Division of Project and Resident Programs

co: J. B. Richard, Managing Director Middle South Energy, Inc. C. K. McCoy, Plant Manager

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ENCLOSURE 3

#### Enclosure 3

## Information provided to Licensee

- Attachment 1 Exerpt from 10/28/83 NRC memo from R. W. Houston to T. Novak providing evaluation of a change request submitted 10/11/83 regarding RCIC initiation
- Attachment 2 NRC memo dated Nov. 7, 1983, from V. Benaroya to C. O. Thomas providing evaluation of Items 1 and 23 submitted 9/9/83 regarding fire protection instruments, and fire hose stations
- Attachment 3 February 9, 1984, memorandum from R. C. Lewis (RII) to D. G. Eisenhut (NRR) regarding comments on NRC staff's second proof and review in the fall of 1983
- Attachment 4 NRC Inspection Report 50-416/84-06 regarding Technical Specification discripancies
- Attachment 5 NRC staff comments and recommended changes for Item 29 of the 9/9/83 submittal requesting a change in Technical Specification nomenclature for radioactive effluent specifications
- Attachment 6 NRC recommended low priority changes for radioactive effluent technical specifications

# ATTACHMENT 1

From 10/28/83 Mous -

By letters dated October 11, 1983, from J. McGaughy

(MP&L) to H. Denton (NRC) and October 14, 1983, from

L. F. Dale (MP&L) to H. Denton the licensee provided

the results of their review of this item. The licensee

reviewed the Grand Gulf - Unit 1 Technical Specification

requirements for the reactor trip, isolation actuation,

emergency core cooling actuation and reactor core isola
tion cooling (RCIC) actuation instrumentation. From the

results of this review the licensee found that in most cases the FSAR incorrectly states the number of instrument channels provided and proposed to submit corrections in the annual FSAR update scheduled for June 1984. For the RCIC actuation instrumentation the licensee proposed a Technical Specification change to increase the number of instrument channels required to be operable from two to four, thus enhancing RCIC reliability and plant safety.

Based on the results of its review the licensee has confirmed that the single failure criterion is satisfied for the reactor trip, isolation actuation, emergency core cooling actuation, and RCIC actuation instrumentation when the Technical Specification requirements are met. With the additional information provided and incorporation of the proposed Technical Specification changes the NRC staff considers this issue resolved.