

NRC PDR



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

RECEIVED

Docket No. 50-416

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LICENSEE: PUBLIC SERVICE COMPANY OF MISSISSIPPI Power & Light

FACILITY: Grand Gulf Unit 1

SUBJECT: SUMMARY OF MARCH 14, 1984, MEETING REGARDING GRAND GULF UNIT 1 TECHNICAL SPECIFICATION REVIEW

The meeting was held to review the current status of processing Technical Specification changes, to obtain additional information on licensee's potential technical specification change requests and to give the licensee NRC staff comments resulting from its review of submitted requests for technical specification changes. Enclosure 1 is a list of attendees.

The licensee reviewed and discussed additional information regarding the potential technical specification changes (Enclosure 2). This information includes the licensee's definition of priorities and the estimated number of potential changes assigned to each priority. They currently estimate there are 32 out of the total of 240 potential changes that should be made prior to exceeding 5% power. The licensee presented and discussed information regarding these 32 items (Attachment 4, Enclosure 2). The licensee also discussed some of the remainder of the 240 problem areas (Attachment 3, Enclosure 2).

The NRC staff discussed its status of review of Technical Specifications which have been submitted. Information discussed and provided to licensee for its consideration are provided in Enclosure 3. Of the 36 requested changes still pending 5 appear to be satisfactory, 2 have been evaluated and accepted by NRC technical review branches and for 3 others, additional information is needed by NRC staff.

L. L. Kintner, Project Manager  
Licensing Branch No. 1  
Division of Licensing

Enclosures:

- 1. List of Attendees
- 2. Information Received from Licensee
  - Attachment 1 - Priority of potential Technical Specification changes
  - Attachment 2 - Sources of potential Technical Specification changes

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XA

ATTACHMENT 3

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 001

Priority: 1B

\_\_\_\_\_ / \_\_\_\_\_

\_\_\_\_\_

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.5.1

Problem Title: ADS Valves Operability Requirement

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec requires at least 7 operable ADS valves in Div. 1 & 2. However, the bases indicates that ADS controls only 7 selected SRVs, while the FSAR only takes credit for 6. Thus allowing one valve out of service for 14 days. It appears that tech specs should require 8 operable ADS valves, since ADS controls 8 valves not 7 and the FSAR assumes loss of one valve.

2. Safety Significance:

Tech Specs is non conservative since it requires only 7 operable valves versus 8.

3. Anticipated Resolution:

Change tech specs for required operable ADS valves from 7 to 8. After review of FSAR Chapter 15 Accident Analysis.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 002

Priority: 2C

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.7.1.2

Problem Title: HPCS Service Water Operability (TYPO)

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

TYPO: The note (\*) dealing with the Tech Spec Operational Conditions has the nomenclature HPSCS instead of HPCS.

2. Safety Significance:

None: TYPO

3. Anticipated Resolution:

Tech Spec change to correct typo.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_  
Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 003

Priority: 2C

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.6.6.3.d.3.c & d

Problem Title: FHA Vent Exhaust/Pool Sweep Radiation Terminology

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Specs indicates Hi FHA (Fuel Handling Area) Vent, Exhaust and Pool Sweep Exhaust Radiation trip initiates standby Gas Treatment System. Correct terminology should be Hi-Hi (vs. Hi).

2. Safety Significance:

Terminology change only to clarify tech specs.

3. Anticipated Resolution:

Change "high" to "high - high"

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 004

Priority: 2D

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: Bases 3/4.6.3

Problem Title: SPMU Bases Incorrect

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The statement under the bases for SPMU "During refueling, neither automatic nor manual action can open the makeup dump valves." is misleading as the SPMU System is not interlocked with the reactor mode switch. Tech Spec needs to be changed to clarify this statement. (Last Paragraph Page B 3/4 6-5)

2. Safety Significance:

None: There is a SPMU mode switch which is administratively controlled by procedures to be in 'OFF' in refueling which prevents automatic or manual action.

3. Anticipated Resolution:

Tech Spec change for clarification.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 005

Priority: 1B

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.3.2 - (3.3.2-1.4.h)

Problem Title: Min. Operable Channels for RWCU Isolation from SLCS initiation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Table 3.3.2-1.4.h lists the minimum operable channels per trip system as "NA" for RWCU isolation from a SLCS initiation. Therefore, the ACTION statements for the Tech Spec do not apply to the channel may remain out of service indefinitely with no action required.

2. Safety Significance:

Tech Specs are confusing and could lead to nonconservative plant operation.

3. Anticipated Resolution:

Change tech specs to require logic channel to be operable and an indicated ACTION taken if channel is inop.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 006

Priority: 2C

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.7.4

Problem Title: Deletion of Snubber List (Snubber Spec Replacement)

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

GGNS Tech Spec 3/4.7.4 presently includes a list of required snubbers by number. NRC recommends replacing existing Snubber spec with new Snubber spec (does not include Snubber List). This is a recommendation at this time. GGNS to determine if this is desired at this time.

2. Safety Significance:

None: Enhancement being considered. Present Tech Spec adequate.

3. Anticipated Resolution:

Tech Spec change to replace current spec with new STS Specification.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 007

Priority: 2E

NRR Verbal Request (Hoffman) /

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.8.1.1.2.d.13

Problem Title: D/G air start

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Delete tech spec requirement for 5 starts of D/G with low receiver press.  
Fire start test requirement has been removed from standard Tech Specs.

2. Safety Significance:

Tech specs are over conservative.

3. Anticipated Resolution:

Per NRC, testing is not required. Delete from tech specs.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 008

Priority: 2G

Ray Patterson /

Identified By                      Date

Responsible Supervisor

Tech Spec Reference: 3/4.4.1.1

Problem Title: Single Recirc Loop Operation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec presently requires two recirc loop operation in modes 1 and 2. Analysis requested from GE for single recirc. loop operation as part of extended operating domain analysis. Single loop operation increases operational flexibility.

2. Safety Significance:

None: Enhancement being considered in order to increase operation flexibility.

3. Anticipated Resolution:

Analysis requested from GE. ECD May 1984. Tech Spec change pending.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 009 Priority: 2C

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Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.6, 3/4.3.7.6, 3/4.9.2, 3.3.6-1.3

Problem Title: SRM Operability Requirements

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The ACTION statement requires different numbers of SRMs operable during the same modes of operation. Tech Spec 3.3.6-1.3 requires 4 SRMs during Op Cond 2 & 5. Tech Spec 3.3.7.6 requires 3 SRMs during Op Cond 2, 3, & 4. And Tech Spec 3.9.2 requires 2 SRMs during Op Cond 5.

2. Safety Significance:

Resolving different number of SRMs required operable for the same Op Cond would clarify tech specs.

3. Anticipated Resolution:

Resolve tech specs so that each Op Cond has only one number of SRMs required operable. (Consistency)

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 010

Priority: 2A

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3/4.3.7.7

Problem Title: TIP System Operability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec. requires only 3 TIPs operable when 5 are required to fully calibrate the LPRMs and map the entire core.

2. Safety Significance:

None: While Tech Spec requires only 3 operable TIPs, operational consideration will require 5 TIPs in order to calibrate the LPRM's and monitor the core.

3. Anticipated Resolution:

Tech Spec change to require 5 operable TIPs.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 011

Priority: 2A

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3.3.6-1.4.a

Problem Title: Control Rod Block Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

According to 3.3.6-1.4.a note d, IRM detector full in interlock is automatically by-passed when the IRM is on range 1. Contrary to this, the IRM design does not include an auto by-pass of detector full in when in range 1 (only IRM downscale is by-passed in range 1).

2. Safety Significance:

Note is confusing as it does not agree with system design. System tested in surveillance procedure per design.

3. Anticipated Resolution:

Delete note (d) from Table 3.3.6-1.4.a (Tech Spec Change).

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 012

Priority: 2C

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.6.3.3

Problem Title: TYPO - SSW Heat Exchanger

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The LCO for Tech Spec 3.6.3.3.b addresses an operable flow path for suppression pool cooling through an SSW heat exchanger. The Tech Spec is incorrect and should address flow through an RHR heat exchanger instead.

2. Safety Significance:

None: Typo.

3. Anticipated Resolution:

Tech Spec change to correct Typo.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 013

Priority: 3A

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.3.2-1

Problem Title: Hi Rad MSIV Isolation Terminology

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Table 3.3.2-1 lists MSIVs closure on Hi Rad. MSL Hi Rad Inop also closes MSIVs. Should it be listed?

2. Safety Significance:

Rad. Inop listing only would clarify Tech Specs. Rad Inop causing MSIV closure tested per existing surveillance procedures.

3. Anticipated Resolution:

Resolve Rad Inop question and list in tech specs if required.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 014

Priority: 2A

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.1.3, 4.1.3.1.4.b

Problem Title: SDV Level Sensor Response

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec surveillance 4.1.3.1.4.b requires the scram discharge volume to be determined operable by verifying proper level sensor response by the performance of a channel functional test. The definition of a channel functional test however does not include the sensor.

2. Safety Significance:

Surveillance Procedures perform Channel Functional Test instead of level sensor response test and could lead to a mode less conservative than required by Tech Spec.

3. Anticipated Resolution:

Evaluation to justify longer surveillance interval for level sensor response to the frequency of a calibration. Tech Spec change pending.

4. NRC Response to Item (NRR/IE):

NRR verbally addresses deleting the reference to channel functional test on 1/24/84.

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 015

Priority: 1D

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: Table 2.2.1-1, 3.3.2-2, 3.3.3-2, 3.3.8-2

Problem Title: Drywell/Containment Pressure Setpoints

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Drywell & containment instrumentation uses absolute pressure transmitters with a psig set point. Barometric pressure changes will affect instrumentation. Pressure set point change may be required to account for barometric pressure changes.

2. Safety Significance:

This is not a Tech Spec problem but a hardware problem of significance. Present setpoints are being adjusted through setpoint calculations to take into account variances in atmospheric pressures.

3. Anticipated Resolution:

NPE has this item for action (PMI-83/12028) to determine resolution. The setpoints have been lowered by .5 psi as an interim measure.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. F. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 016

Priority: 1B

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3/4.3.8-3.3.8-2.1.b

Problem Title: Containment Press Setpoints

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

According to the GE design spec the CTMT Press setpoint/allowable valves should be 8.25/8.85 vs 9.0/9.2 the #'s in the present Tech Spec were provided to MP&L by GE during proof and review period. GE subsequently issued different #'s in the design spec revision (after receipt of license).

2. Safety Significance:

This was identified by the surveillance review team in mid 1983; it was a low priority since procedural controls were in place to ensure compliance (calibration, function, channel checks procedures, etc.)

3. Anticipated Resolution:

Tech Spec change to correct setpoint valves.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 017

Priority: 2C

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 4.7.1.3.b

Problem Title: SSW tower fan operability requirements

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 3/4.7.1.3 does not identify if fans must only be run for 15 minutes or started and run for 15 minutes (ie must fans be started and run specifically for this spec or may fan operation for other tests quality.)

2. Safety Significance:

Enhancement to tech specs to clarify testing requirements.

3. Anticipated Resolution:

Add note to tech specs to clarify them.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 018

Priority: 3B

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.3.7.1.1.b and 3/4.3.7.12.b

Problem Title: Radiation Monitor Operability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Tech Spec action statement says that if the minimum number of operable channels is exceeded, then the channels must be restored within the specified time frame or explain why it was not restored in the next semi-annual release report. This is being interpreted that discharges may continue beyond the time frame provided action required is met and a report is submitted in the next release report.

2. Safety Significance:

None: Releases are still being monitored in accordance with the required action statements.

3. Anticipated Resolution:

NRR (Hoffman) to get position statement from METB. Plant Staff Tech Spec position statement issued (TSPS-018).

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 019 \_\_\_\_\_  
\_\_\_\_\_ / \_\_\_\_\_

Priority: 1B \_\_\_\_\_  
\_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.6.7.3.b.1

Problem Title: Drywell Purge Flowrate Definition

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Surveillance requires verifying a flow rate of 1000 cfm every 18 months. However, the flow rate should be considered in scfm, since the flow rate is temperature dependent. This could effect equipment environmental qualifications.

2. Safety Significance:

Changing from "cfm" to "scfm" would clarify flow requirements for surveillance tests.

3. Anticipated Resolution:

Change "1000 cfm" to "1000 scfm".

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_  
\_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 020

Priority: 2A

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.6.4

Problem Title: Pneumatic Testing of Certain CTMT Isolation Valves

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

In response to the CSB of NRC concerns MP&L reviewed the rationale and acceptability of hydrostatically leak testing valves in Tech Spec. This review concluded that 12 of the affected 38 penetrations did not meet the requirements of a strict application of the NRC single active failure criterion for water-filled systems and as such these penetrations should have been pneumatic tested.

2. Safety Significance:

The hydrostatic testing of the associated valves as presently required by Tech Spec is less conservative than required. Per AECM-83/0540 the valves will be pneumatic tested beginning at the next scheduled time period. These delays at the pneumatic testing is justified in AECM-83/0540.

3. Anticipated Resolution:

Tech Spec change initiated (TSCR-83/07).

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 021

Priority: 1C

\_\_\_\_\_ / \_\_\_\_\_

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.7.4

Problem Title: Snubber Added by DCP on RCIC

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

New snubber, Q1E51G18OR01, added per DCP 82/0546. (See item #21)

2. Safety Significance:

DCP requires a Tech Spec change. Snubber may affect operability of RCIC.

3. Anticipated Resolution:

Add new snubber to Tech Spec list of snubbers.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 022

Priority: 1D

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.4.1 (3.3.4.1)

Problem Title: ATWS Recirc Pump Trip

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The associated action statements are not consisted with system design. For example if one trip channel is inop, then the inop channel is suppose to be tripped, this action will trip one recirc pump. If two channels are inop. (depending on channels) then the trip system may be declared inop. with 72 hours time period to declare it operable.

2. Safety Significance:

The action statement for a less serve condition is more restrictive and could cause undue plant shutdown transients.

3. Anticipated Resolution:

Tech Spec change to modify action statements to fit plant design.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 023

Priority: 2A

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.4.2.1 (3.4.2.1, 3.4.2.2)

Problem Title: SRV Relief Function/Lo Lo Set Tech Spec

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The SRV relief function and Lo Lo set function have two trip systems (A & B solenoids). The tech spec does not address trip systems, minimum operable channels, allowable value, trip function, etc. which are terms normally associated with instrumentations.

2. Safety Significance:

no ACTION statement exists similar to these found in the instrumentation section of Tech Specs for the relief function or Lo Lo set spec. This can cause confusion.

3. Anticipated Resolution:

Tech Spec change to show functional relationship between relief function and Lo Lo set function. Also to change the spec to be more like and instrument spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 024

Priority: 2B

\_\_\_\_\_  
Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

\_\_\_\_\_  
Responsible Supervisor

Tech Spec Reference: 3/4.4.1.2

Problem Title: Jet Pump Operability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec requires Jet Pump operability in modes 1 and 2. Surveillance requires jet pumps to be proven operable prior to exceeding 25% thermal power. At what point in power ascension are jet pumps required to be demonstrated operable.

2. Safety Significance:

None: Tech Spec implementation problem. Jet pumps required to be determined operable prior to exceeding 25% for modes 1 and 2. However, Jet Pumps operability cannot be determined prior to the entry into modes 1 and 2. (Tech Spec 3.0.4 cannot be met).

3. Anticipated Resolution:

Tech Spec change change to add 4.0.4 exemption.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 025

Priority: 2C

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Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.4.7

Problem Title: MSIV Minimum Closing Stroke Time

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Evaluate changing surveillance limits for MSIV stroke time. Per GE and GE design specs, minimum MSIV stroke time is 2.5 seconds vs current testing limit of 3.0 seconds. (Chapter 15 of FSAR uses 3 seconds)

2. Safety Significance:

Present minimum stroke time is more conservative. Change would provide consistency between tech specs and GE design specs and provide more margin for MSIV's stroke time.

3. Anticipated Resolution:

Change minimum MSIV stroke time from 3.0 seconds to 2.5 seconds per GE and GE design specs after FSAR disparity resolved.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 026

Priority: 2C

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.8.1.1.2.c

Problem Title: TYPO - Diesel Fuel Oil Sampling Spec

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

TYPO: ASTM-D270-1975 should be ASTM-D270-1965 (Reapproved 1980).

2. Safety Significance:

None: TYPO

3. Anticipated Resolution:

Tech Spec changes to correct TYPO.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 027

Priority: 2D

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 6.10.2.L

Problem Title: Record Retention TYPO

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

TYPO - correct reference to snubber table in section 6.10.2.1 should be 3.7.4-1 and 3.7.4-2.

2. Safety Significance:

TYPO - No safety significance

3. Anticipated Resolution:

Change tech specs to correct typo

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: Denton (AECM-83/0565, Item 30)      9/9/83      /

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

/ \_\_\_\_\_  
Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 028

Priority: 2A

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.4.3.2 (Table 3.4.3.2-1)

Problem Title: RCS Interface Table 3.4.3.2-1 Specific Valves

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Tech Spec Table 3.4.3.2-1 "RCS Pressure Isolation Valve" does not address specific valves (i.e., E12-F041 should include valves A, B, C; E12-F042 should include valves A, B, C; E12-F050 should include valves A & B; and E12-F053 should include valves A & B). This items is included with the generic RCS Leakage spec problems.

2. Safety Significance:

While the Tech Spec Table does not address specific valves, etc. The correct valve number and testing is included in associated GGNS surveillance procedures.

3. Anticipated Resolution:

Tech Spec change to correct table.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: <u>Denton (AECM-83/0565, Item 8)</u>	<u>9/8/83</u>	<u>/</u>	
Individual Notified	Date	Time	

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 029

Priority: 3B

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: Table 1.2, Table 4.3.6-1

Problem Title: Changing Mode Switch Test Neutron Monitoring Inst

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

In order to perform neutron monitoring channel functional test (SRMs IRMs, APRMs) the reactor mode switch need to be placed in the refueling position, while in Op Cond 3 & 4. Position Statement 12 addresses plant staff interpretation of situation.

2. Safety Significance:

Resolution indicates that only surveillance procedure methods need to be altered.

3. Anticipated Resolution:

Method to test neutron monitoring instrumentation without changing mode switch was found. Change surveillance procedures and change/delete position statement 12.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 030

Priority: 2C

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\_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.5.1.2.b

Problem Title: HPCS Line Break Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec for HPCS Line Break not included in Standard Tech Spec format. Delete this spec from GGNS Tech Specs. Based on the system design the HPCS Line Break Alarm will normally be in until a reactor temperature of 430°F is reached. In addition is this doubtful whether the system would detect a line break or not (MNCR 823/83).

2. Safety Significance:

It is doubtful whether the system will actually meet its intended design function. This spec has been deleted from STS and is being considered as an enhancement for deletion from GGNS Tech Specs

3. Anticipated Resolution:

Tech Spec changed to delete Spec.

4. NRC Response to Item (NER/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 030

Priority: 2C

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 4.5.1.2.b

Problem Title: HPCS Line Break Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec for HPCS Line Break not included in Standard Tech Spec format. Delete this spec from GGNS Tech Specs. Based on the system design the HPCS Line Break Alarm will normally be in until a reactor temperature of 430°F is reached. In addition is this doubtful whether the system would detect a line break or not (MNCR 823/83).

2. Safety Significance:

It is doubtful whether the system will actually meet its intended design function. This spec has been deleted from STS and is being considered as an enhancement for deletion from GGNS Tech Specs

3. Anticipated Resolution:

Tech Spec changed to delete Spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 031

Priority: 2C

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.6.2.3.a

Problem Title: Drywell Air Lock Test Frequency

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Drywell Air Lock Seal Leakage test frequency is 8 hours in Tech Specs. But STS lists freq. as 72 hours (this is consistent with 10CFR 50 Appendix J criteria for containment airlocks).

2. Safety Significance:

Present testing is more conservative. Change would make GGNS tech specs consistent with STS and permit more time until testing is solved.

3. Anticipated Resolution:

Change test freq. to be consistent with STS.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 032

Priority: 2A

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.4.3.2

Problem Title: RCS Leakage Spec Implementation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Generic RCS leakage Tech Spec implementation problems (ie Spec list channel checks required for drywell air codes condensate flow instrument loop, however, no instruction are given it loop is inop).

2. Safety Significance:

While the Tech Spec is confusing and implementation is difficult, there is reasonable assurance that with appropriate controls the intent of Tech Specs can be met in ensuring operable equipment status.

3. Anticipated Resolution:

Review Spec and determine appropriate changes. Submit changes to NRC for review.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

                    Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_  
Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
                    Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 033

Priority: 1B

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: Table 3.3.8-2

Problem Title: Containment Spray/Plant Actuation Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Change containment spray timer setpoint and allowable value according to GE specs. Pending resolution of Containment Spray Timer issue.

2. Safety Significance:

Plant Staff has administratively increased the surveillance frequency to insure the instrument stays within the tech spec allowable value.

3. Anticipated Resolution:

Provide analysis of containment pressure for transient/accident which results in worst containment pressure. NPE has MNCR for action. Resolve timer requirements.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 034

Priority: 1B

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.3

Problem Title: Generic Instrumentation Problems

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

NRR I&C Branch mandated Tech Spec review of Tech Spec Instrumentation. Review includes submitting definitions of channels, trip systems, trip functions. Also included is review of action statements, minimum operable channels, LCO's and single failure criteria.

2. Safety Significance:

Tech Spec difficult to understand and implement. Could lead to non-conservative system operations.

3. Anticipated Resolution:

Review of Instrumentation Tech Spec with Tech Spec change pending.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 035

Priority: 2B

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 4.9.6

Problem Title: Refueling Platform Load Spec/Aux Platform up Travel

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Up travel limit for Auxiliary Platform hoist not identified. Refueling platform hoist load limit specs (load limit ± accuracy limit) confusing and inappropriate for equipment and their intended function.

2. Safety Significance:

3. Anticipated Resolution:

Define up travel limit and change load limit specs to more appropriate load limits (ie load limit).

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 036

Priority: 2D

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.12

Problem Title: Environmental Monitoring Spec

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Environmental Monitoring Tech Spec contains various items that are confusing and are incorrect. In addition some of the required reporting is too frequent. Revise Tech Spec per attached.

2. Safety Significance:

None: enhancements.

3. Anticipated Resolution:

Tech Spec Change

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 037

Priority: 1B

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3/4.3.2

Problem Title: Riley & Rosemont Trip Units

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Specs 4.3.1.1-1 and 4.3.3.1-1 require monthly cal of Rosemont trip units. Contrary to this, Spec 4.3.2 does not require monthly cal of Rosemont trip units. Tech Spec requirement for temp switch cal (Riley) is every 18 months. Vendor, however, recommends a yearly cal frequency.

2. Safety Significance:

3. Anticipated Resolution:

Change cal frequency for Riley Temp switches from once/18 months to once/12 months and the Rosemont trip unit cal from once/18 months to once/30 days.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 038

Priority: 1C

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.3.7.1-1.5

Problem Title: Carbon Bed Rad Mon Calibration Frequency

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec surveillance Table 4.3.7.1-1.5 list calibration frequency for the carbon Beck Vault Radiation Monitor as 18 months. The vendor recommends a calibration frequency of 12 months.

2. Safety Significance:

None: Annual calibration of the monitor is administratively controlled through surveillance program.

3. Anticipated Resolution:

Tech Spec change to change calibration frequency.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 039

Priority: 2F

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.7.2-1

Problem Title: Seismic Monitoring

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Some seismic monitors located on piping, which could experience large transients from plant operation then from a seismic event. Also, a review has shown that instrument range is not large enough for some anticipated transients.

2. Safety Significance:

DCP 82/625 requires a tech spec change deleting 3 seismic instruments and changing the range of 1 on Table 3.3.7.2-1.

3. Anticipated Resolution:

Per DCP 82/625 and in accordance with Reg Guide 1.12, remove 1C85-XR-R011, R012, and R013. Also, change range of 1C85-XR-R013 to .01 to 10 g's.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 040

Priority: 2E

Identified By

Date

Responsible Supervisor

Tech Spec Reference: Table 3.3.2-3

Problem Title: Isolation Instrumentation Response Time

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Evaluate eliminating response times for the instrumentation not used in FSAR analysis. Deleting response times in Table 3.3.2-3 deletes unnecessary testing.

2. Safety Significance:

None: Tech Spec over conservative requiring response time testing for instrument response not used in analysis.

3. Anticipated Resolution:

Review of Instrumentation Tech Spec to determine which response times can be eliminated. Tech Spec change pending.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 041

Priority: 2A

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.4.1.1

Problem Title: Recirc. Loop Operability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The surveillance requirements do not meet the requirements for the limiting condition for operation section 3.4.1.1. The surveillance check valve operation not loop operation.

2. Safety Significance:

The proposed change establishes better surveillance requirements, therefore, it does not involve any significant hazards or safety considerations. Present surveillance requirements are 18 month and can not change modes if not operable is adequate and verification of 2 recirc pumps is covered other ways.

3. Anticipated Resolution:

Change 4.4.1.1 to check loop operation and add tech spec requirements to check flow control valves.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 042

Priority: 2A

\_\_\_\_\_  
Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.4.1.3

Problem Title: Recirc Flow Nomenclature

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec nomenclature misleading. The reference to recirc flow needs to be changed to jet pump flow and the reference to rated recirc flow needs to be changed to effective core flow. This also applies to bases.

2. Safety Significance:

None: The Tech Spec as written can be used to determine recirc loop mismatch for the purpose of Tech Spec. The above change provides another method or way of meeting intent of spec. Operation does not lead to non-conservative conditions.

3. Anticipated Resolution:

Tech Spec change initiated (TSCR 83-06)

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                          Date                          Time

5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                          Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 043

Priority: 2C

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.8.1.1.2.9.3, 4.8.1.1.2.d.14

Problem Title: D/G Fuel Oil Pump Surveillance Table

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

D/G surveillance requirement 4.8.1.1.2.a.3 appears to be redundant to surveillance 4.8.1.1.2.d.14.

2. Safety Significance:

No safety significance as change is only to clarify tech specs (Does not change testing requirements).

3. Anticipated Resolution:

Delete one of redundant requirements in Tech Spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 044

Priority: 3B

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.3.3 (Table 3.3.3-2.D.2)

Problem Title: Div. 3 LOP Time Delay Trip

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Bechtel Engineering commented that contrary to Tech Spec offsite power trips without a time delay when Div. 3 voltage drops to 3045 volts.

2. Safety Significance:

None: Plant Staff disagrees with Bechtel and is of the opinion that the Tech Spec deals with the diesel attraction instrumentation which has a time delay associated and not with the feeder breaker trip for which there is no trip delay.

3. Anticipated Resolution:

Review and resolve above discrepancy. Tech Spec change pending.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 045

Priority: 2A

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.3.7.12 (T.S. 3.3.7.12 and Bases 3/4.3.7.12)

Problem Title: ODCM Setpoints

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Table 3.3.7.12-12 includes same instrument channels for which the alarm/trip setpoints are not calculated in accordance with ODCM.

2. Safety Significance:

Change to Tech Spec would only clarify source of setpoints. Change would not change testing requirements.

3. Anticipated Resolution:

Change bases to Table 3.3.7.12-1 to clarify that not all setpoints are not calculated from the ODCM.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

/

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 046

Priority: 2E

/

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.9.11.1

Problem Title: RHR Shutdown Cooling During Refueling

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec presently requires at least one shutdown cooling mode of RHR operable and in operation during Refueling. Plant Staff is of the opinion that this is an undue Tech Spec requirements due to the large heat sink available during refueling.

2. Safety Significance:

None: Enhancement being considered for operational considerations.

3. Anticipated Resolution:

Plant Staff review the Tech Spec change pending.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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\_\_\_\_\_

Date                      Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 047

Priority: 2A

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.4.2.3 (Surveillance 4.3.4.2.3)

Problem Title: EOC RPT Response Time Testing

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec requirements requires response time testing of the trip functions for Turbine Stop Valves and Control Valves every 18 months. The next sentence states that each test shall include at least the logic of one type of channel input (control valve fast closure or stop valve closure) such that both are tested every 36 months.

2. Safety Significance:

Present spec is confusing and presently controlled administratively by surveillance procedure frequency control.

3. Anticipated Resolution:

Change Tech Spec to clarify testing frequency requirements.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 048

Priority: 2G

Ray Patterson / \_\_\_\_\_

Identified By                      Date

\_\_\_\_\_ / \_\_\_\_\_  
Responsible Supervisor

Tech Spec Reference: 3/4.2.2

Problem Title: Extended Cycle Operation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Long Term Item: Change Tech Spec to allow for increased core flow and maximum extended operating domain (required by 1st outage). Requires GE analysis which is presently underway.

2. Safety Significance:

None: Enhancement to be considered for operational flexibility.

3. Anticipated Resolution:

Analysis presently underway. ECD May 1984. Tech Spec change pending.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 049

Priority: 2B

Ray Patterson /

Identified By                      Date

Responsible Supervisor

Tech Spec Reference: 3/4.2.2

Problem Title: APRM Gain Adjustment

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

ACTION statement requires restoration of APRM flow biased simulated thermal power high scram setpoint and flow biased neutron flux upscale control rod block trip setpoint to within limits within 2 hours. STS requires 6 hours. Delete "the required gain adjustment increment does not exceed 10% of RATED THERMAL POWER" PMI-84/2047, PCOL 84-01.

2. Safety Significance:

3. Anticipated Resolution:

Change Tech Specs

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 050

Priority: 2A

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.7.7 (Surveillance 4.3.7.7)

Problem Title: TIP Surveillance

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Tech Spec Surveillance 4.3.7.7 requires for operability the normalization of each required detector outputs within 72 hours prior to use when required for the monitoring of thermal limits or calibration. In order to use the raw TIP's for monitoring you cannot normalite the detector output. Hence the statement "for the above applicable monitoring or" should be deleted.

2. Safety Significance:

In order for monitoring of thermal limits using TIPs it is required that the TIP readings not be normalited.

3. Anticipated Resolution:

Tech Spec Change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 051

Priority: 2C

\_\_\_\_\_  
Identified By / Date Responsible Supervisor

Tech Spec Reference: 3/4.1.4.1, T.S.3.1.4.1

Problem Title: Control Rod Withdrawal - Typo

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
TYPO - Change "withdrawl" to "withdrawal".

2. Safety Significance:  
TYPO - No safety significance.

3. Anticipated Resolution:  
Change Tech Spec to correct typo.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 052

Priority: 2B

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 6.2.2.f

Problem Title: Organization; Title Change for Plant Manager

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec makes reference to the title Plant Superintendent, instead of Plant Manager as used by GGNS.

2. Safety Significance:

None: Change for clarification to meet specific plant nomenclature.

3. Anticipated Resolution:

Tech Spec change to reference to plant manager.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 053

Priority: 2D

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 2.1.4

Problem Title: Reactor Vessel Water Level Action Wording

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Wording in ACTION statement for 2.1.4 states "manually initiate the ECCS to restore the water level, after depressurizing the reactor vessel, if required." Statement is confusing. Why initiate ECCS after depressurizing? Will depressurizing worsen problem since more water will flash to steam?

2. Safety Significance:

Bases changed to clarify intent to tech spec - no safety significance.

3. Anticipated Resolution:

Tech Spec change (Bases) to clarify intent of spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 054

Priority: 1B

NRC (I&C plus NRR) /

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.8

Problem Title: CTMT Spray Min Operable Channel

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec Table 3.3.8-1 presently requires one min. operable channel for the Containment Spray System. The NRC concurs that the number of min operable channels be changed to 2 based on review of trip functions. In addition, the requirement for Reactor Vessel Water Level LLL, L1 should be deleted as it really does not apply. This is required to conform with present trip function definitions.

2. Safety Significance:

Could lead to less than MOC being required for operation.

3. Anticipated Resolution:

Tech Spec change pending review.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 055

Priority: 2C

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.4.4.c

Problem Title: Reactor Coolant System Surveillance Requirement Clarification

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

NRC recommends deleting "for up to 31 days". Surveillance requirement implies that conductivity recorder may be out of service "for up to 31 days" before inline conductivity measurements must begin.

2. Safety Significance:

Clarification of Tech Spec.

3. Anticipated Resolution:

Request Tech Spec change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 056

Priority: 3B

NRC (I&E plus NRR) /

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.5.1

Problem Title: HPCS Auto Transfer from CST to Supp. Pool

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

NRC recommended change to include requirement of operable auto transfer from CST to Supp. Pool.

2. Safety Significance:

None: Adequate operating instructions (administrative controls) can ensure above requirement is satisfied.

3. Anticipated Resolution:

Response to Denton explaining request is unjustified.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 057

Priority: 2A

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.6.1.2.K

Problem Title: 4.0.2 Exemptions

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

4.0.2 cannot exempt Appendix J requirements. Delete 4.0.2 exemption NOTE:  
4.0.2 still applies to section of 4.6.1.2. Research of Appendix J has shown  
that 3 year interval on 4.6.1.2.g should be added to 4.6.1.2.K.

2. Safety Significance:

3. Anticipated Resolution:

Response to NRC (Denton) explaining request is unjustified.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 058

Priority: 2C

NRC (I&E plus NRR) / \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.7.9

Problem Title: Spent Fuel Pool Temp Spec

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec requires that the spent fuel pool temp be maintained less than 150°F. NRC recommends that we review necessity for Tech Spec. This spec is not included in the STS or any other specific plant Tech Spec. Check to see if La Crosse has this spec.

2. Safety Significance:

None: Enhancement being considered to delete Tech Spec.

3. Anticipated Resolution:

Determine applicability and necessity of Tech Spec. Tech Spec change maybe pending.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 059

Priority: 2C

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.7.10

Problem Title: Embankment Stability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The wording in the LCO and the action needs to be reworded to state 1) where the clearing will occur and 2) what criteria will be used to verify stability.

2. Safety Significance:

Rewording further clarifies and defines tech spec requirements.

3. Anticipated Resolution:

Change tech spec to clarify above concerns.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 060 Priority: 2A

NRC (I&C plus NRR) /1/24/84

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.8.1.2

Problem Title: Diesel Generator Requirement (AC Sources - Shutdown)

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The safety analyses during operation conditions 4, 5, and \* does not assume a single failure with an initiation event coincident with a loss of offsite power as it does during op conditions 1, 2, and 3. Therefore, there is no requirement to have two diesel generators operable in this condition. Therefore, delete the "and" in "and/or" in the 1st and 2nd line of action A and change LCO 6.2.a to reflect only on diesel generator.

2. Safety Significance:

None: See problem description above.

3. Anticipated Resolution:

Tech Spec change submitted to NRC in AECM-83/0565 item 11 which meets the intent of the above, however, not word for word.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 061

Priority: 2C

NRR (Hoffman) / \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.8.4.2

Problem Title: Thermal Overloads

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Wording in Action Statement is confusing and needs clarification. Change "no OPERABLE" to "inoperable" and delete "take administrative action to"/

2. Safety Significance:  
Change is to clarify tech spec.

3. Anticipated Resolution:  
Change tech spec per above.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_  
\_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 062 Priority: 2D

NRC (I&E plus NRR) / 1/24/84

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.6.6.3.a, 4.7.2.a, Bases 3/4.6.6 and 3/4.7.2

Problem Title: Moisture Control Charcoal Bed Heaters

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The heaters need to be operated continuously over a 10-hour period every 31 days to reduce the buildup of moisture. Cumulative operation of the heaters for 10 hours over a 31 day period will not suffice. Change the reference to cumulative operation to continuous operation for 10 hours.

2. Safety Significance:

NOTE: Tech Spec surveillance 4.6.6.a and 4.7.2.a requires operation of the systems for at least 10 hours every 31 days with heaters operable. The present design of the system is such that the heaters operate to control moisture based on design criteria (SGTS heaters run whenever the system is running and the control room fresh air unit heaters cycle to control moisture switch.

3. Anticipated Resolution:

Review the above and determine if change is required

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 063

Priority: 2D

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 6.2.3.2

Problem Title: Composition of Independent Safety Eng Group (ISEG)

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec states "The ISEG shall be composed of a group with a minimum assigned complement of fire engineers or appropriate specialist". What are "appropriate specialist"?

2. Safety Significance:

None.

3. Anticipated Resolution:

Develop procedure approved by SRI regarding how the determination will be made to ensure appropriate specialists are utilized (ie qualifications of personnel).

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 064 Priority: 2D

NRC (I&E plus NRR) / 1/24/84  
Identified By Date Responsible Supervisor

Tech Spec Reference: 6.5.1.3

Problem Title: PSRC Alternates

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The time frame that the alternates can be used and the qualification of these alternates are not addressed. Change Tech Spec or proposed procedures which address: 1) length of time alternates may be utilized and the criteria for the times and 2) the qualification of the alternates.

2. Safety Significance:

None: Administrative controls required to better define PSRC alternates.

3. Anticipated Resolution:

Tech Spec change or admin. procedure change to address above requirements for PSRC alternates.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 065

Priority: 2D

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 6.5.2.3

Problem Title: SRC Alternatives

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Address qualifications of SRC alternatives and length of time they can serve

1) procedural define temporary 2) qualifications of personnel.

2. Safety Significance:

3. Anticipated Resolution:

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 066

Priority: 2C

NRC (I&C plus NRR)                      /1/24/84

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.3.1

Problem Title: Instr with Functions Required by Different Modes

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Presently there are several Tech Specs which do not have consistent requirements regarding the number of systems/components required to be operable for different functions and modes of operation such as between RPS/ECCS/PBM/Individual Systems.

2. Safety Significance:

None: The difference sections etc, while may not have consistent requirements, have the number of system/components required operable in order to meet or perform the intent of the associated Tech Spec.

3. Anticipated Resolution:

Review individual sections of Tech Spec to verify consistency of required functions and consistency of surveillance frequencies. Propose Tech Spec changes accordingly.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 067

Priority: 2C

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.6.1.2.C.1 & 4.6.1.2.C.3

Problem Title: Recommended Change to the Standard Tech Specs

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

According to the NRC (Division of System Integration) in a letter dated Nov. 3, 1983, surveillance requirement 4.6.1.2.C.1 and 4.6.1.2.C.3 of the Standard Technical Specifications are not well worded. The letter recommends specific changes to be incorporated. This requirement involves verification of Type A test accuracy.

2. Safety Significance:

3. Anticipated Resolution:

Further research into matter required.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 068

Priority: 3B

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3.6.1.9

Problem Title: Containment Purge System Limiting Cond. for Operation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

According to NRC (Division of Systems Integration) letter dated Nov. 3, 1983, the Limiting Condition for Operation 3.6.1.9 should be modified to indicate that the 20-inch and 6-inch purge systems shall not be used at the same time.

2. Safety Significance:

3. Anticipated Resolution:

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 069

Priority: 2D

/

Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3/4.6.7.2.a

Problem Title: H2 Igniter Surveillance

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Tech Spec required 41 to 45 H2 igniters operable. NRC questions which igniters may be inoperable and if spec adequately covers operability requirements.

2. Safety Significance:

3. Anticipated Resolution:

Letter to NRC indicating that it is an open Owners' Group item.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

/

Date	Time
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cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 070 Priority: 3B

NRR / 1/24/84

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.7.6.1, Action a & b and 4.7.6.1.1.c

Problem Title: Fire Suppression Water System

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The NRC CEB recommends rewriting action paragraphs a and b. The revision clarifies the existing action statements that an alternate backup pump or supply must be provided if inoperable equipment is not restored within 7 days. It also deletes the requirement for the Special Reports.

2. Safety Significance:

3. Anticipated Resolution:

Investigate the recommended change for incorporation into the GGNS Technical Specification.

4. NRC Response to Item (NRR/IE): CEB Notified NRR 11-7-83

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 071 Priority: 2C

NRC (I&E plus NRR) /1/24/84  
Identified By Date Responsible Supervisor

Tech Spec Reference: 3/4.7.6.2.3, 4, 5, 6 & 3/4.7.7

Problem Title: Deletion of Special Reporting Require Fire Protect

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

CEB of NRC recommends deleting special reporting requirements of Tech Specs 3.7.6.2, 3.7.6.3, 3.7.6.4, 3.7.6.5, 3.7.6.6, and 3.7.7.

2. Safety Significance:

None: Enhancement being considered for deletion of special reporting requirement.

3. Anticipated Resolution:

Tech Spec change to delete special reporting requirement in action statement in LCO.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 072

Priority: 2C

NRC (I&E NRR) /1/24/84

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.7.6.2

Problem Title: Visual Inspection of Nozzle Spray Area

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The CEB of NRC recommends that an additional surveillance requirement be added to Tech Spec Surveillance 4.7.6.2 which requires verification by visual inspection that each nozzle's spray area and pattern is not obstructed.

2. Safety Significance:

None: NRC recommended change to Tech Spec surveillance.

3. Anticipated Resolution:

Review to determine if justified.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 073

Priority: 1C

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.3.7.9

Problem Title: Fire Detection Instrumentation Format Revision

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Fire Detection Instrumentation Tech Spec has a confusing format which can lead to taking wrong actions. Also all the detections are not included. (ie. PGCC).

2. Safety Significance:

Interim controls may be necessary to ensure the appropriate action, are taken if detectors become inop.

3. Anticipated Resolution:

Final tech spec change proposal submitted on 9-9-93 (AECM-83/0565). Earlier proposal was denied by NRR since our submittal did not match the St. Luire Tech Spec format which the NRR likes.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 074 Priority: 2C

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.3.3 Tables 3.3.3-1, 3.3.3-2, 4.3.3.1-1

Problem Title: Unused Footnotes

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Check all tables and delete unused footnotes.

2. Safety Significance:

None.

3. Anticipated Resolution:

Delete all unused footnotes.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 075

Priority: 2A

Identified By

Date

Responsible Supervisor

Tech Spec Reference: Table 3.3.3.2.B.2.e

Problem Title: LPCI Pump B and C Discharge Pressure High Allow Valve

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

A Tech Spec change was submitted to Table 3.3.3-2 as item 3 of AECM-83/0338 dated June 14, 1983. The change modified the allowable values for items A.2.f and B.2.e from 122 psig increasing to 115 - 135 psig increasing. Amendment 8 changes only item A.2.f. Change still needed item B.2.e.

2. Safety Significance:

3. Anticipated Resolution:

Change Allowable Value of item B.2.e from "115" psig increasing to "115-135" psig increasing.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 076

Priority: 1B

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.3 Table 3.3.3-3.2.a

Problem Title: ECCS Response Time

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The low pressure coolant injection (LPCI) mode of RHR System is noted in Tech Specs Table 3.3.3-3 as having a pump response time of 45 seconds for trains A and B. In order to be consistent with the accident analysis assumptions as noted in FSAR Table 6.3-1 and 6.3-2 the correct response time is 40 seconds.

2. Safety Significance:

The proposed change is somewhat more limiting than the original requirement by its shorter response time and is consistent with the existing accident analysis.

3. Anticipated Resolution:

Change Tech Spec Table 3.3.3-3 item 2 to 40 seconds.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 077 Priority: 3A  
NRR / 1/24/84

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.7:4

Problem Title: Remote Shutdown Panel - Addition of Division II Instru.

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

During the meeting on 1-24-84 with MP&L, Region II and NRR, one of the "proof and review" NRC comments was to consider adding both divisions of Remote Shutdown Panels to the Tech Spec if they are both required. (NRR I&C Branch identified this question on 10-31-83, but NRC did not inform MP&L of concern until 1-24-84).

2. Safety Significance:

If a change is determined to be necessary, interim controls could be necessary in order to ensure conservative operation.

3. Anticipated Resolution:

Item was assigned to NPE on 2/6/84 (PMI-84/1677) to review present design and the major modification to be installed at 1st refueling outage and make recommendation 1st cycle operation Tech Spec and 2nd cycle operation Tech Spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 078

Priority: 1B

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.3.5 Table 3.3.5-1.a

Problem Title: RCIC Minimum Operable Channels

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Change minimum operable channels for RCIC Level 2 trip from 2 to 4. Since there is only one trip system, the ACTION statement referring to two trip systems should be deleted.

2. Safety Significance:

Administrative controls need to be in place ensure the operator knows all channels of LL2 logic are required for operability in order to ensure proper compliance.

3. Anticipated Resolution:

Change Tech Specs as described above.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 079

Priority: 2D

NRR / \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: Bases 3/4.3.1

Problem Title: Reactor Protect System Instrumentation Bases - IEEE 279

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Bases Section for Tech Spec 3/4.3.1 states that: "The system meets the intent of IEEE 279 for nuclear power plant protection systems". ICSB recommends deletion of the statement since the SER contains a detailed discussion on the conformance of the GGNS design to NRC regulatory requirements.

2. Safety Significance:

This change is editorial in nature and does not affect system operability.

3. Anticipated Resolution:

Delete the statement referencing IEEE 279 from the Bases for Tech Spec 3/4.3.1.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 080

Priority: N/A

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: N/A

Problem Title: N/A

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Item #80 duplicate of item #54.

2. Safety Significance:

None.

3. Anticipated Resolution:

None.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 081

Priority: 3A

NRR / 1/24/84

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.1

Problem Title: Reactor Mode Switch Surveillance

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Encl. 5, item 9 during the meeting on 1/24/84 with MP&L, Region II and NRR, one of the "proof and review" NRC comments was to consider adding limiting conditions for operating and surveillance requirements for safety related functions accomplished by the mode switch.

2. Safety Significance:

The test which MP&L performs on the Mode Switch verifies the bypass of Reactor Protection System signals appropriate for each mode switch position.

3. Anticipated Resolution:

Per letter AECM-83/0577 MP&L informed the NRC that current testing of the mode switch was adequate.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 082

Priority: 3A

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.7.1, 4.7.1.1

Problem Title: SSW Auto Initiation Circuit

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Specs addresses testing only automatic actuation of SSW System's valves. NRC requests periodic testing of the automatic actuation of the SSW system pumps and cooling tower fans.

2. Safety Significance:

We now test auto diesel start initiation but do not test the other ECCS auto - initiations.

3. Anticipated Resolution:

Determine need and submit if appropriate.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 083

Priority: ID

NRR 1/24/84 /

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.6.3.4

Problem Title: Suppression Pool Makeup Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The actuation instrumentation for the suppression pool makeup system (in Tech Spec 3/4.6.3.4) is not included in the instrumentation section of the Tech Specs.

2. Safety Significance:

Even though the instrumentation is not specifically in the Tech Specs, by definition of operability, the instrumentation is required to be operable per Tech Spec 3/4.6.3.4.

3. Anticipated Resolution:

The Tech Spec should be revised to include the Suppression Pool Makeup System Actuation Instrumentation.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 084

Priority: 3A

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.7.3

Problem Title: RCIC Turbine Protection Trips

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The design of the RCIC system includes protective features to automatically shut down the turbine by tripping the trip and thruttle valve closed if any of the following conditions are defected: turbine over-speed high turbine exhaust pressure, RCIC isolation signal, or low suction pressure. These items are not included in operability and surveillance requirements.

2. Safety Significance:

Tech Spec clarification.

3. Anticipated Resolution:

Review and add to Tech Specs if required.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 085 Priority: 2C  
NRR / \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.11.3

Problem Title: Solid Radioactive Waste-Changes to Reflect NRC Intent

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The STS has been revised (Rev. 3 Draft) changing Tech Spec 3.11.3 wording and actions, based on NRC experience, to better reflect the intent NRC regulations. Also definitions of Process Control Program and solidification were revised from draft of most recent Westinghouse STS.

2. Safety Significance:

Wording and actions modified to reflect intent of NRC regulations.

3. Anticipated Resolution:

Revise the Tech Spec to incorporate changes made to the STS.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 086

Priority: 2C

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 4.11.1.3.2 and 4.11.2.5.2

Problem Title: Liquid Waste Treatment & Ventilation Exhaust Treat

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The wording and actions do not totally reflect the intent of the NRC regulation, NUREG-0472, Rev. 3 Draft (1/4/83).

2. Safety Significance:

The proposed change has no impact on safety but it changes the frequency of the requirements.

3. Anticipated Resolution:

Radwaste/operation to review for applicability and if applicable add frequency requirement.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 087  
NRR /

Priority: 2C

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: Table 4.11.2.1.2-1 note c

Problem Title: Rad Gaseous Waste Sample & Analysis Program-Exemptions

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Note c of Table 4.11.2.1.2-1 does not match Note g of Table 4-11-2 Rev. 3  
Draft of the STS.

2. Safety Significance:

Change reflects the intent of NRC guidance.

3. Anticipated Resolution:

Revise Note c of Table 4.11.2.1.2-1 to add exemption from sampling per Rev. 3  
Draft of STS.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 088

Priority: 2C

\_\_\_\_\_ / \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 6.15.1

Problem Title: Major Changes to Radwaste Treatment System

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

To clarify the requirements of Tech Spec 6.15.1 for implementation

2. Safety Significance:

The proposed change has no impact on safety.

3. Anticipated Resolution:

Add footnote to Tech Spec 6.15 on page 6-26 to provide alternative means of reporting changes in liquid, gaseous, and solid waste treatment system.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 089

Priority: 2C

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 4.11.1.2

Problem Title: Radioactive Effluents Dose Calibration-Clarify

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Phrase was omitted from Tech Spec 4.11.1.2 which would clarify what time period the cumulative dose contribution is applicable.

2. Safety Significance:

This phrase was included in Tech Spec 4.11.2.2 and 4.11.2.3 but inadvertently omitted from Tech Spec 4.11.1.2.

3. Anticipated Resolution:

Add the phrase "for the current calendar quarter and current calendar year" following "...liquid effluents..."

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 090

Priority: 2C

\_\_\_\_\_  
Identified By / Date Responsible Supervisor

Tech Spec Reference: 3/4.11.4

Problem Title: Radioactive Effluents

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

To clarify and reduce reporting requirements by replacing 3/4.11.4 with total dose spec of NUREG-0472, Rev. 3 Draft (1/4/83).

2. Safety Significance:

The proposed change has no impact on safety, it changes time for calculating radioactivity and radiation releases actions and surveillance requirements.

3. Anticipated Resolution:

Change Tech Spec 3/4.11.4 to reflect the new requirements.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 091

Priority: 2C

NRR /

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.12.2

Problem Title: Typographical Error

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
in Tech Spec 4.12.2 typo - "chesus" vs "census".

2. Safety Significance:  
Typographical Error

3. Anticipated Resolution:  
Correct Typo

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 092

Priority: 2C

\_\_\_\_\_ / \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.12.3

Problem Title: Radiological Environ. Monitoring/Comparison Program

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

A tech spec change would be required if the NRC changed from the EPA to another organization for their intercomparison program.

2. Safety Significance:

The proposed change has no impact on safety.

3. Anticipated Resolution:

Change Tech Spec 4.12.3, delete last sentence.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

\_\_\_\_\_  
\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 093

Priority: 2D

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 6.9.1

Problem Title: Routine Reports & Reportable Occurrences

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

10CFR 50.73 Licensee Event Report System revised the reporting requirements of Tech Spec 6.9.1.

2. Safety Significance:

Administrative control is required to ensure reporting requirements of 10CFR 50.73 are met.

3. Anticipated Resolution:

Revise Tech Spec Section 6.9.1 to incorporate requirements of 10CFR 50.73.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 094

Priority: 2C

\_\_\_\_\_  
Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

\_\_\_\_\_  
Responsible Supervisor

Tech Spec Reference: 3/4.7.1

Problem Title: HPCS Serv. Water

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

GGNS Tech Spec does not require reverifying every 18 months testing each pump starts automatically to maintain service water pressure greater than or equal to 60 psig.

2. Safety Significance:

We test pump characteristics; including pressure quarterly per T.S.4.0.5 and test service initiation logic every 18 months per T.S. 4.8.1.1.12.d.4.b.2.

3. Anticipated Resolution:

Determine need and submit if appropriate.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_  
\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 095 Priority: 2D

/

Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: Figure 6.2.1-1

Problem Title: Revision to Offsite Organization Chart

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Existing Figure 6.2.1-1 of the Tech Specs does not accurately reflect current organization.

2. Safety Significance:

Change is required to update chart, does not affect plant operation.

3. Anticipated Resolution:

Update Figure 6.2.1-1 to reflect current MP&L offsite organization.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
  Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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\_\_\_\_\_ / \_\_\_\_\_  
  Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 096 / \_\_\_\_\_ Priority: 2D \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 6.5.2.7

Problem Title: SRC Duties - ALARA

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

FSAR requires semi-annual review of ALARA appraisals by SRC. NRC wants to add this to Tech Specs.

2. Safety Significance:

None.

3. Anticipated Resolution:

Determine need and submit if appropriate.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 097

Priority: 2D

\_\_\_\_\_  
Identified By / Date Responsible Supervisor

Tech Spec Reference: Tech Spec Index

Problem Title: Renumbering of Tech Spec Index

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Index to the Tech Specs require revision to incorporate changes made by amendments, (e.g., additional pages, sections added, etc.)

2. Safety Significance:

This change is editorial in nature and does not affect plant operation.

3. Anticipated Resolution:

Renumber the Index to the Tech Specs to reflect changes made by Amendments.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 098

Priority: 2F

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.3.7.8.4.7.2.d.2.b & Bases 3/4.3.7.8

Problem Title: Deletion of Chlorine Detector Specification

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Per Bechtel analysis performed on chlorine in PCOL 83-125 the requirement for control room isolation on chlorine detection is no longer justified. Bechtel recommends that the above specs dealing with chlorine be deleted from Tech Specs.

2. Safety Significance:

None: Enhancement

3. Anticipated Resolution:

Review and determine if Tech Spec change justified.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_  
\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 099

Priority: 2F

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.3.3

Problem Title: ADS

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Commitment to install instrumentation to monitor ADS air receiver pressure (see AECM-83/0672 and AECM-83/0510). ADS leak test frequency change, allowable leakage rate, and ADS ACTION statements (AECM-83/0677 item 9).

2. Safety Significance:

3. Anticipated Resolution:

Change total specs to reflect plant changes as necessary

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 100

Priority: 3B

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.7.8 Item c

Problem Title: ESF Electrical Room Max Temperature

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Bechtel has generated an 0588 FSAR change to the ESF Electrical Room max temp. from 104°F to 90°F.

2. Safety Significance:

If appropriate; would mean a more conservative temperature.

3. Anticipated Resolution:

Resolve with Bechtel, if correct.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 101

Priority: 2D

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: Figure 6.2.2-1

Problem Title: New Plant Organization

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Figure 6.2.2-1 of the Tech Specs requires revision to show new Unit Organization, with three Assistant Plant Managers.

2. Safety Significance:

The change in organization were made to utilize personnel resources. Positions requiring a license of certification have not been reduced by this change.

3. Anticipated Resolution:

Revise Figure 6.2.2-1 to reflect new Unit Organization and add note to allow changes to Unit Organization without prior Tech Spec approval.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 102

Priority: 1C

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.7.9

Problem Title: Fire Detection Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Revise Tech Specs to include smoke detectors as specified MPB 83/0328. Also reference AECM-84/0093.

2. Safety Significance:

Adding smoke detectors will enhance the capability to detect a fire in safety-related equipment.

3. Anticipated Resolution:

Add to Tech Specs.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 103

Priority: 2A

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.3.2-1.2.d

Problem Title: Main Steam Line Flow Min. Operable Channels

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

In an effort to standardize definitions of "Channel", "Trip System", and "Trip Function", it was found that Main Steam Line Flow Instrumentation minimum operable channels was based on a channel of Four instruments versus 4 instrument channels.

2. Safety Significance:

Spec is unique in referring to an isolation channel versus an instrument channel, clarity can be gained by changing the spec. With or without a change the spec requires 16 instruments to be operable.

3. Anticipated Resolution:

Submit a Tech Spec change which would require "8 minimum operable channels per trip system" instead of "2 (with notes)".

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 104

Priority: 2D

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 6.1.2

Problem Title: Title Change

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Change title from shift superintendent to shift supervisor.

2. Safety Significance:

Will clarify administrative controls.

3. Anticipated Resolution:

Incorporate change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 105

Priority: 2D

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 5.1.2

Problem Title: Effluent Release Boundary for Gaseous & Liquid Eff

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The terminology in Tech Spec 5.1.3 is incorrect for gaseous and liquid effluents "Effluent Release Boundary" is appropriate for the Spec versus "Unrestricted Area Boundary". Also Figure "5.1.4-1" in note 3 of Figure 5.1.1-1 should be Figure "5.1.3-1".

2. Safety Significance:

The change is purely administrative in nature since it involves only the correction of terminology and a typo.

3. Anticipated Resolution:

Correct the terminology in Tech Spec 5.1.3 and correct the typo in Figure 5.1.1-1.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 106

Priority: 2D

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 6.5.1

Problem Title: Change in PSRC Membership

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The proposed changes added two members to the PSRC. The plant responsibilities of the proposed new members are described in FSAR Section 13.1.2.2.10.

2. Safety Significance:

This change only expands the membership of the PSRC by the addition of two members whose job specification requires a broad-based knowledge of GGNS operation.

3. Anticipated Resolution:

Implement change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

/

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 107

Priority: 2D

\_\_\_\_\_  
Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: Bases 3/4.6.1.7

Problem Title: CTMT-to-Aux. Bldg. and Enclosure Bldg. Differential

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
The range for initial containment-to-auxiliary and Enclosure Bldg. differential pressure is incorrectly stated in Basis 3/4.6.1.7. The correct range is (-)0.1 to 1.0 psid. Per Tech Spec 3.6.1.7.

2. Safety Significance:  
None: Plant enhancement correction of bases.

3. Anticipated Resolution:  
Correct the basis to the correct range for initial differential pressure.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 108

Priority: 2B

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.1.3.2

Problem Title: Maximum Scram Insertion Time

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The subject Tech Spec address the maximum scram insertion times and the scram accumulator operability requirements for Ops. Cond. 1 & 2. The proposed change exempts the specs from the applicability of 3.0.4.

2. Safety Significance:

Exemption to 3.0.4 can be provided when startup with inoperable equipment would not affect plant safety as noted in the bases for 3.0.4.

3. Anticipated Resolution:

Add exemptions.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

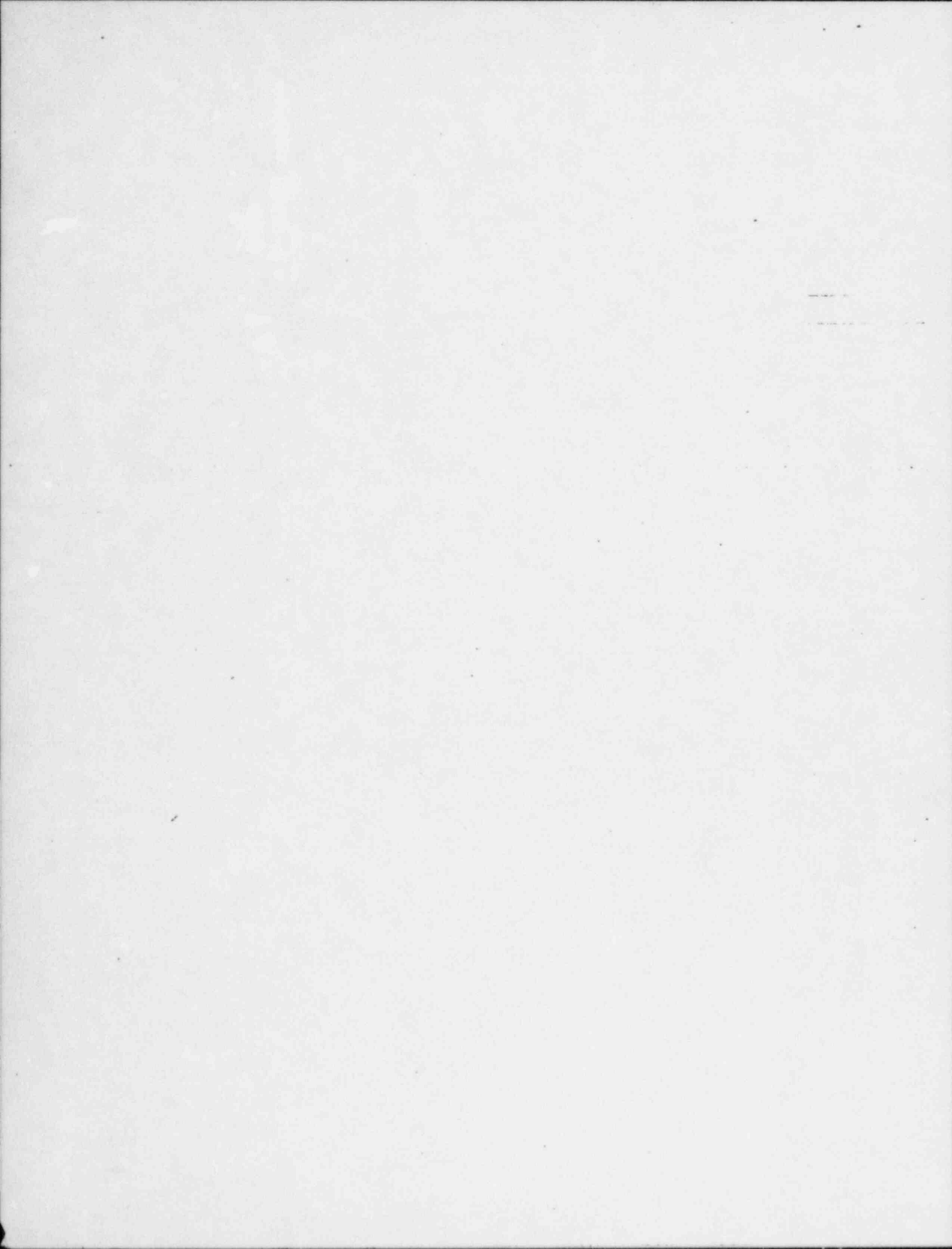
Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 109

Priority: 2C

Identified By

Date

Responsible Supervisor

Tech Spec Reference: Table 3.3.1-1

Problem Title: Tech Spec Table 3.3.1-1 Action 2 Editorial Change

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Action 2 of Tech Spec Table 3.3.1-1 requires verification that all control rods are inserted in the core. Standard Tech Spec requires verification that all "insertable" control rods are inserted in the core, to allow for inoperable rods that are immovable.

2. Safety Significance:

The change is administrative in nature clarifying the existing Tech Spec Action Statement.

3. Anticipated Resolution:

Change Action 2 of Table 3.3.1-1 to require verification that all "insertable" control rods are inserted in the core to conform to STS.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 110

Priority: 2A

Identified By \_\_\_\_\_ / \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.3.2-x (Table 3.3.2-3.5.a)

Problem Title: RCIC Flow Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Generic Letter 83-02 (MAEC-83/0034) provided guidelines with regard to pipe break detection circuitry. Using these guidelines would add the "###" in Table 3.3.2-2 to read: ###: Includes time delay of 3 to 7 seconds.

2. Safety Significance:  
The change to prevent spurious isolations of RCIC on Steam Line Flow-high does not reduce safety margins.

3. Anticipated Resolution:  
Add note.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_  
NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 111

Priority: 2C

Identified By

Date

Responsible Supervisor

Tech Spec Reference: Table 3.3.2-2 (Table 3.3.2-2 item 1g, 3c, 3a)

Problem Title: Radiation Isolation Trip Setpoints

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The trip setpoints for items 1.G, 3.c and 3.d in Table 3.3.2-2 were incorrectly stated. The values listed were derived from maximum alarm values in Table 3.3.7.1-1 items 7, 8, and 9. The correct setpoints were intended to be item 1c ==3.6mR/hr\*\* item 3c ==3.6mR/hr\*\*, and item 3d ==30mR/hr\*\*.

2. Safety Significance:

The new setpoints are still within the allowable values. Original setpoints were incorrectly listed (editorial change).

3. Anticipated Resolution:

To provide the intended separation between the alarm and trip function the Tech Spec is to be changed to reflect the correct setpoints.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 112

Priority: 1D

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.2 (TS 3.3.1.c, TS 3.3.2.b)

Problem Title: Isolation Instrumentation Channel/Trip

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The proposed change brings present action "b" of T.S. 3.3.2; its associated "\*" notation, and the "\*" notation of T.S. 3.3.1 in accordance with STS. The STS wording does not require an inoperable channel to be replaced in a tripped condition where this would cause the Trip Function to occur.

2. Safety Significance:

This change helps to clarify the GGNS Tech Specs.

3. Anticipated Resolution:

Incorporate Change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 113

Priority: Closed

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Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.2.1-1.2.e (Table 4.3.2.2-1.2.e)

Problem Title: Condensor Vacuum-Low Surveillance Exception

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec Table 4.3.2.1-1 item 2.e required surveillance to be performed in OP Cond 1, 2\*\*, and 3\*\*. The "\*\*\*" note requires the surveillances for conditions 2 and 3 to be performed when reactor pressure is  $\approx$ 1045 psig and/or any turbine stop valve is open. However, Tech Spec 3.4.6.2 requires reactor steam dome pressure to be 1045 in Condition 1 and 2.

2. Safety Significance:

3. Anticipated Resolution:

Change footnote \*\* on Table 4.3.2.1-1 and add footnote \*\* to Table 3.3.2-1.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 114

Priority: 2A

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Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.3 (Table 3.3.3-2.c.1.e; Table 3.3.5-2.d)

Problem Title: Suppression Pool High Level

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Table 3.3.3-2 and 3.3.5-2 currently identify the allowable value for the Suppression Level-High HPCS and RCIC isolation setpoints as 6.5". This should be corrected to 7" in both tables.

2. Safety Significance:  
The level criteria for HPCS and RCIC isolation has not changed; only the reference elevation of the instrumentation was revised.

3. Anticipated Resolution:  
Implement change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 115

Priority: 2C

Identified By

Date

Responsible Supervisor

Tech Spec Reference: Table 3.3.3-1

Problem Title: T.S. Table 3.3.3-1 Footnote (e) & \*\* Clarification

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Footnote (e) to Table 3.3.3-1 is not used and is unclear. Footnote \*\* requires revision to clarify that it is applicable only when the "associated" ESF equipment is required operable.

2. Safety Significance:

The change is editorial in nature.

3. Anticipated Resolution:

- a. Delete Footnote (e).
- b. Change to indicate "...when applicable ESF equipment..."

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 116

Priority: 1B

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.3.1 (Table 4.3.3.1-1.A.2.e)

Problem Title: LPCS Pump Discharge High

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

A reference to note (a) is to be added to the CHANNEL CALIBRATION entry for item A.2.e.

2. Safety Significance:

Proposed change adds a requirement to calibrate the above trip unit monthly. This revision makes this trip unit consistent with those of other Division 1, 2, and 3 trip systems.

3. Anticipated Resolution:

Implement Change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 117 Priority: Closed

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 4.3.4.2.3 (T.S. 4.3.4.2.3 & Bases 3/4.3.4)

Problem Title: End-of-Cycle Recirc Pump Trip Breaker Arc Supp.

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 4.3.4.2.3 states that the time allotted for breaks are suppression, is 50 milliseconds. This is incorrect. The 50 msec is for breaker interrupting time which includes 3.5-45 msec from energization of the breaker trip coil to opening of the breaker contracts and 4-12 msec from opening of breaker contracts to complete suppression of the electric arc.

2. Safety Significance:

3. Anticipated Resolution:

Change Tech Spec 4.3.4.2.3 to indicate arc suppression time to be 12 msec and change bases 3/4.3.4 to clarify that 190 msec response time is the total from initiation of valve motion to complete suppression of the breaker electric arc.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 118

Priority: 2C

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Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.6 (Table 3.3.6-1.3.a, Table 4.3.6-1.3.a)

Problem Title: SRM "Detector Not Full In" Rod Block Operability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The proposed change consists of adding a note (e) to Table 3.3.6-1 to state that provisions of T.S. 3.0.4 are not applicable for entering Ops Cond. 5.A note (\*\*\*) is added in Table 4.3.6.1 to require verification that SRM detectors are "full in" at least once per 24 hours when detector drive motor module is removed.

2. Safety Significance:

Operability of SRM "detector not full in" rod block is satisfied even with the drive motor module removed.

3. Anticipated Resolution:

Implement Change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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\_\_\_\_\_

Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 119

Priority: 2A

Identified By \_\_\_\_\_ / \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: (Table 3.3.7.1-1.10.1.3)

Problem Title: Area Radwaste Monitors Dryer Storage Area Minimum Channels

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Amendment 7 to the Tech Spec added Item 10.a.3 to Table 3.3.7.1-1 however, the minimum number of channels operable was not included.

2. Safety Significance:  
The problem is only editorial in nature.

3. Anticipated Resolution:  
Add "1" under the "Minimum Channels Operable" column for item 10.a.3 of Table 3.3.7.1-1.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 120

Priority: 2C

\_\_\_\_\_ / \_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.3.7.1 (Table 3/4.3.7.1-1)

Problem Title: Offgas Pretreatment & Post-treatment Offgas

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
T.S. that apply to each instrument are not consistent among the tables. The change deletes the specs for above in Table 3.3.7.1-1 and 4.3.7.1-1 and leaves the specs in Table 3.3.7.12-1 and 4.3.7.12-1.

2. Safety Significance:  
Change is purely administrative in nature and is made to promote consistency in the tech specs.

3. Anticipated Resolution:  
Implement Changes.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
  Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_  
\_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
  Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 121

Priority: Closed

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 4.3.7.6.c

Problem Title: Min Allowable Count Rate for SRM Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 4.3.7.6.c requires that prior to withdrawal of control rods, the SRM count rate be at least 3 counts/sec. Table 3.3.6-2 requires a SRM downscale trip setpoint of 3cps for the control rod block function to be considered operable. Based on the current SRM count rate, it is estimated that the source strength will be insufficient to maintain 3cps due to normal decay of the sources.

2. Safety Significance:

3. Anticipated Resolution:

Lower the minimum SRM count rate to 0.5 cp. Also lower downscale rod block to 0.5 cps for the allowable rate and 0.7 cps for the trip setpoint.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 122 Priority: 2C

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: Table 4.3.7.12- (items 1.d, 3.d, 4.d, 5.d)

Problem Title: Flow Rate Monitors Functional Test

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Delete the channel functional test requirement for the above.

2. Safety Significance:

Since there are no alarm or trip functions from the flow rate monitors. The deletion of the quarterly channel functional test presents no significant hazards consideration.

3. Anticipated Resolution:

Implement Change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 123

Priority: 2C

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: T.S. 3.4.1.4

Problem Title: Reactor Vessel Steam Coolant Temp Measurement

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 3.4.1.4 requires measurement of the vessel steam spare coolant temp. This is accomplished by taking a pressure measurement and using the process computer convert into a temperature value. However, at ambient pressure temperature below 212°F are not available. (Used to determine difference in temperature between bottom head drain line coolant & vessel steam space coolant for idel recirc loop startup.

2. Safety Significance:

The present Tech Spec places unnecessary restrictions on recirculation loop operation when vessel pressure is at ambient because temperature will indicate 212°F.

3. Anticipated Resolution:

Change the Tech Spec so that measurement of the vessel steam spare coolant temperature is not required below vessel pressure of 25 psig.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 124

Priority: 2B

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.1.1.c (Surveillance 4.1.1.c)

Problem Title: Shutdown Margin Demonstration Immovable Rod

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Change Time Constraint from 1 to 6 hours for determining shutdown margin after detection of a immovable control rod.

2. Safety Significance:

Enhancement for operating convenience.

3. Anticipated Resolution:

Evaluate and Propose change accordingly.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 125

Priority: Closed

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.4.2.1

Problem Title: SRV Low-Low Set

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

NSSS vendor analysis of loss of feedwater events indicates that an isolation transient may result in only 1 SRV (with 1103 psig setpoints opening, initially. The resultant pressure transient, with 1 SRV opened may not react 1113 (the low scenario, a subsequent failure of the 1103 psig setpoint SRV later in the isolation event could result in the simultaneous opening of the 10. ganged SRV with a setpoint of 1113 psig.

2. Safety Significance:

3. Anticipated Resolution:

Maintain the original arming logic at 1113 psig but add logic which also arms the low-low set relief upon the opening of the lowest setpoint SRV (1103 psig).

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 126

Priority: 2C

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.5.3.1 (TS 4.5.2.1.a, TS 3.6.3.1.c, Base 3/4.6)

Problem Title: Suppression Pool Operability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The present 12'5" listed in the above T.S. is a "typo" and should be corrected to read 12'8" as in other specs. There is also a change needed to T.S. 3.6.3.1a which corrects present suppression pool high water volume of 138,851 ft<sup>3</sup> to 138,701 ft<sup>2</sup>. Also the bases needs to be changed.

2. Safety Significance:

The above changes are considered purely administrative to promote consistency between the FSAR and Tech Specs are to correct typographical errors.

3. Anticipated Resolution:

Incorporate changes.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 127

Priority: 2F

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.6.2.5 and Bases 3/4.6.2.5

Problem Title: Drywell to CTMT Differential Press Allow Limits

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 3.6.2.5 incorrectly states that the allowable limits for the Drywell to Containment dp is (-)0.1 and (+) 2.0 psid. However, the values should be (-)0.26 to 2.0 psid to make the Tech Spec representative of plant design which incorporates changes to resolve one of the Humphrey concerns.

2. Safety Significance:

This change requires a Tech Spec change prior to implementation of the DCP to change the setpoints.

3. Anticipated Resolution:

Change Tech Spec 3.6.2.5 to correct values and change Tech Spec Bases 3/4.6.2.5 value of (+)1.0 psid to (+)0.26 psid. DCP is pending this Tech Spec Change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 128

Priority: 2C

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: Bases 3/4.6.7

Problem Title: Upgrade Revision Date of NUREG

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The proposed change updates the effective revision date of Reg. Guide 1.7.

The applicable revision according to commitments made in FSAR is Rev. 1, dated Sept. 1976.

2. Safety Significance:

Change is administrative in nature.

3. Anticipated Resolution:

Incorporate change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 129

Priority: 2A

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.7.1.1 (T.S. 3.7.1.1)

Problem Title: SSW Spec Inconsistent with Other ESF Specs

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Standby Service Water (SSW) Tech Spec 3.7.2.2 LCO and Action Statement are inconsistent with Tech Specs of ESF Equipment which are supplied by SSW. The associated system Tech Specs are: 3.4.9.1, 3.4.9.2, 3.5.2, 3.9.11.1, 3.9.11.2 and 3.8.1.2.

2. Safety Significance:

The existing Tech Spec is confusing as to which SSW subsystem is required to be operable, however administrative controls can ensure conservative operation.

3. Anticipated Resolution:

Revise Tech Spec 3/4.7.2.2 LCO to be more consistent with those for the associated ESF systems.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 130

Priority: Closed

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.7.6.1 (4.7.6.1.3)

Problem Title: Diesel Fire Pump Battery

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Correct diesel driven fire pump battery spec.

2. Safety Significance:

Incorporated in Amendment 12.

3. Anticipated Resolution:

Incorporated in Amendment 12.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_  
Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 131

Priority: 2F

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: Table 3.7.6.5-1

Problem Title: Additional Firehose Stations to Table 3.7.6.5-1

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Two additional hose stations are required to be added to Tech Spec Table 3.7.6.5-1 one station (13G) is added per design change and the other hose station (SSC) was inadvertently omitted from the table.

2. Safety Significance:

3. Anticipated Resolution:

Add fire hose stations 13G and 55C to Table 3.7.6.5-1.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 132

Priority: 2A

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.7.8 (Table 3.7.8-1.c)

Problem Title: Control Room Temperature Limits

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

In view of the intended concern with equipment qualification the temperature for the control room was set too low. The 77°F limit should be changed to 90°F. The proposed change also deletes the "Equipment Not Operating" column and the heading "Equipment Operating".

2. Safety Significance:

The change in the limit will not impact equipment operability. The deletion of the columns is administrative and involves conforming to present Tech Specs.

3. Anticipated Resolution:

Incorporate change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 133

Priority: 2C

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.7.10

Problem Title: Embankment Stability - T.S. 3.0.3 & 3.0.4 N/A

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 3.0.3 requires that when an LCO is not met, action must be taken to put the unit in an operational condition in which the Tech Spec is not applicable. Since Tech Spec 3.7.10 is applicable "at all times," it is not possible. Tech Spec 3.0.4 intent is to ensure that unit operation is not initiated with either required equipment or system inoperable or other limits being exceeded. Startup with slope embankment not within specs will not affect plant safety.

2. Safety Significance:

The existing Tech Spec is adequate and provisions are being complied with.

3. Anticipated Resolution:

Changes Tech Spec to indicate that the provisions of Tech Spec 3.0.3 and 3.0.4 are not applicable.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2. 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 134

Priority: 2C

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Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.8.1.1 (Surv. 4.8.1.1.2.d.9 NOTE \*)

Problem Title: Typed Incorrect Reference.

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Correct a typo in the notation "\*" at bottom of page 3/4 8-6. The Tech Spec reference should be changed from 4.8.1.1.2.d.4.a to 4.8.1.1.2.d.7.a.

2. Safety Significance:

Change is correction of a typo and is purely administrative in nature.

3. Anticipated Resolution:

Incorporate.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 135

Priority: N/A

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 4.8.2.1.d.2.b

Problem Title: Division II Battery DC Load Profile

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 4.8.2.1.d.2.b specifies the DC load profile for the Division II ESF battery. The spec did not accurately reflect the "as built" load or the planned change in the Division II inverter load.

2. Safety Significance:

3. Anticipated Resolution:

Change the Spec to reflect "as build" load profile.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 136

Priority: 2C

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.8.4.2 (Table 3.8.4.2-1)

Problem Title: Correct Valve Numbers

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The proposed change corrects the listing valves Q1M71F593A and Q1P41F018 which should be Q1M71F593 and Q1P41F218A, respectively.

2. Safety Significance:

This change corrects two typos.

3. Anticipated Resolution:

Incorporate change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 137

Priority: 2A

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.8.4.2.1.a

Problem Title: Channel Functional Test of MOV Thermal Overload bypassed in Accident Condition

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 4.8.4.2.1.a requires that the bypass circuitry for MOV thermal overloads which are normally in force during plant operation and bypass during an accident condition be tested once per 92 days. By definition of a Channel Functional Test this would require testing the entire channel. This would result in disruption of plant operation since this would require simulation of an ECCS actuation signal.

2. Safety Significance:

Circuits are presently tested as described in "Anticipated Resolution" section with the concurrence of the commission.

3. Anticipated Resolution:

Change Tech Spec to require testing the individual valve thermal overload bypass circuitry once per 92 days and of the entire channel once per 18 months.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 138

Priority: 2C

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: Table 4.11.2.1.1-1

Problem Title: Radioactive Gaseous Waste Sampling

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Two additional points for monitoring gaseous release proposed for addition to Table points are Radwaste Bldg. vent exhaust and fuel handling area vent exhaust figure Note "6" as applied to sampling frequency for CTMT and Turbine Bldg. Ventilation Exhaust systems is moved to apply to only the principal gamma emitters and not to H-3. Another change involves the deletion of the "f" note. (Ref. PMI-83/11172, PCOL-83/22).

2. Safety Significance:

None: Enhancement for clarification.

3. Anticipated Resolution:

Submit Tech Spec change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 139

Priority: 1C

\_\_\_\_\_  
Identified By                                  Date                                  Responsible Supervisor

Tech Spec Reference: Table 3.7.4-2

Problem Title: Corrections to Snubber Table 3.7.4-2

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Change to Tech Spec Table 3.7.4-2 Mechanical Snubber Table requires the following corrections: 1) Addition of snubbers that were inadvertently left off the table. 2) Deletion of snubbers that have been voided, superceded or incorrect. 3) Correct typos. 4) Add Non-Q Mech. snubbers included in stress analysis of Q-piping.

2. Safety Significance:

The Tech Spec Table is non-conservative, some snubbers included in design analysis are missing.

3. Anticipated Resolution:

Reference Item No. 006-Existing snubber Tech Spec to be revised, will not have list of snubbers.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
   Individual Notified                                  Date                                  Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                                  Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 140

Priority: 2C

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3/4.6.4 and 3/4.6.6.2

Problem Title: Isolation Valve Closure Time Table 3.6.4-1/3.6.6.2-1

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The associated table's maximum valve closure isolation times do not contain consistent bases for the times given. The tech Spec change as addressed in AECM-83/0449 applies a margin to valve times as tested in pre-op. The margin as applied were obtained using bases provided in the ASME Section XI criteria.

2. Safety Significance:

None: Provides a common data base for valve closure times.

3. Anticipated Resolution:

Tech Spec change submitted in AECM-83/0449.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 141

Priority: 2E

\_\_\_\_\_  
Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: Table 4.8.1.1.2-1

Problem Title: D/G Test Schedule -Applicability to Different D/G Type

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Grand Gulf has two different types of diesel generators of different design and size. Division I and II diesel generators are Delaval (7000kw) and Div. III is EMD (3300kw). Reg. Guide 1.108 paragraph C.2.a.9 implies the diesel generators of different types should be evaluated separately.

2. Safety Significance:

Tech Specs are presently being interpreted in the most conservative possible way. Revision of Spec would eliminate the requirement to start a diesel more frequently because a diesel of a different type experience a failure.

3. Anticipated Resolution:

Review Tech Spec Table 4.8.1.1.2-1 and Reg Guide 1.108 for feasibility of evaluating diesel failures and frequency of testing on diesel types vs. total diesels.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 142

Priority: 2E

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.8.1.1.1

Problem Title: D/G Fast Starts

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Delete timing requirement on D/G loading for fast cold starts. Fast cold starts have been demonstrated to cause accelerated degradation of D/G. See NUREG 1024 section 3.3.

2. Safety Significance:

3. Anticipated Resolution:

Change Tech Specs.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 143

Priority: 2F

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.8.2.1.d.2.a

Problem Title: Div I ESF Battery DC Load Profile

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Presently the load profile of Tech Spec 4.8.2.1.d.2.a does not allow sufficient margin to accommodate future loads.

2. Safety Significance:

Increasing the values for the Div I DC load profile will allow addition of future loads without having to wait for a Tech Spec change.

3. Anticipated Resolution:

Evaluate benefits/feasibility of increasing the load profile in Spec 4.8.1.2.d.2.a so that additional loads may be added to the DC bus without having to change Tech Spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 144

Priority: 2A

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.6.1.1.a

Problem Title: Primary Containment Integrity

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 4.6.1.1.a states the equipment send hatches must be leak rate tested if any penetration requiring type B testings is opened. Equipment send hatches should only be tested if they have been affected.

2. Safety Significance:

Tech Spec clarification.

3. Anticipated Resolution:

Change Tech Spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 145

Priority: 2E

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.8.1.1.2.d.6

Problem Title: D/G Testing Deletion of 4.8.1.1.2.d.6

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

NRC recommends that surveillance requirement 4.8.1.1.2.d.6 regarding D/G testing be deleted because of inconsistency with the Spec requirements and the provisions of GDC 17, RG 1.108 and SRPs 8.2 and 8.3.1 (Generic Letter 83-30).

2. Safety Significance:

None: Enhancement

3. Anticipated Resolution:

Tech Spec Change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 146

Priority: 1B

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: SER 9.5.4.1/Tech Spec 6.8

Problem Title: Control Room Ceiling Work

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Per SER, work in control room ceiling may only be performed in cold shutdown.

2. Safety Significance:

Unimplemented SER requirements.

3. Anticipated Resolution:

Revise Admin section of Tech Specs and implement controls.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_  
\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_

Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 147

Priority: 3B

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.3.3; SER 8.4.4.1.d and SER 16

Problem Title: Elect Distribution System Voltage Monitoring and Trip

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

SER 8.4.4.1.d requires that Tech Spec include maximum and minimum limits on trip setpoints and allowable valves for the second-level voltage protection sensors and associated time delay devices. Page 3/4 3-28.

2. Safety Significance:

Tech Specs may not conform to SER commitments.

3. Anticipated Resolution:

Review Tech Spec Table 3.3.3-2 and determine if table includes above requirements.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

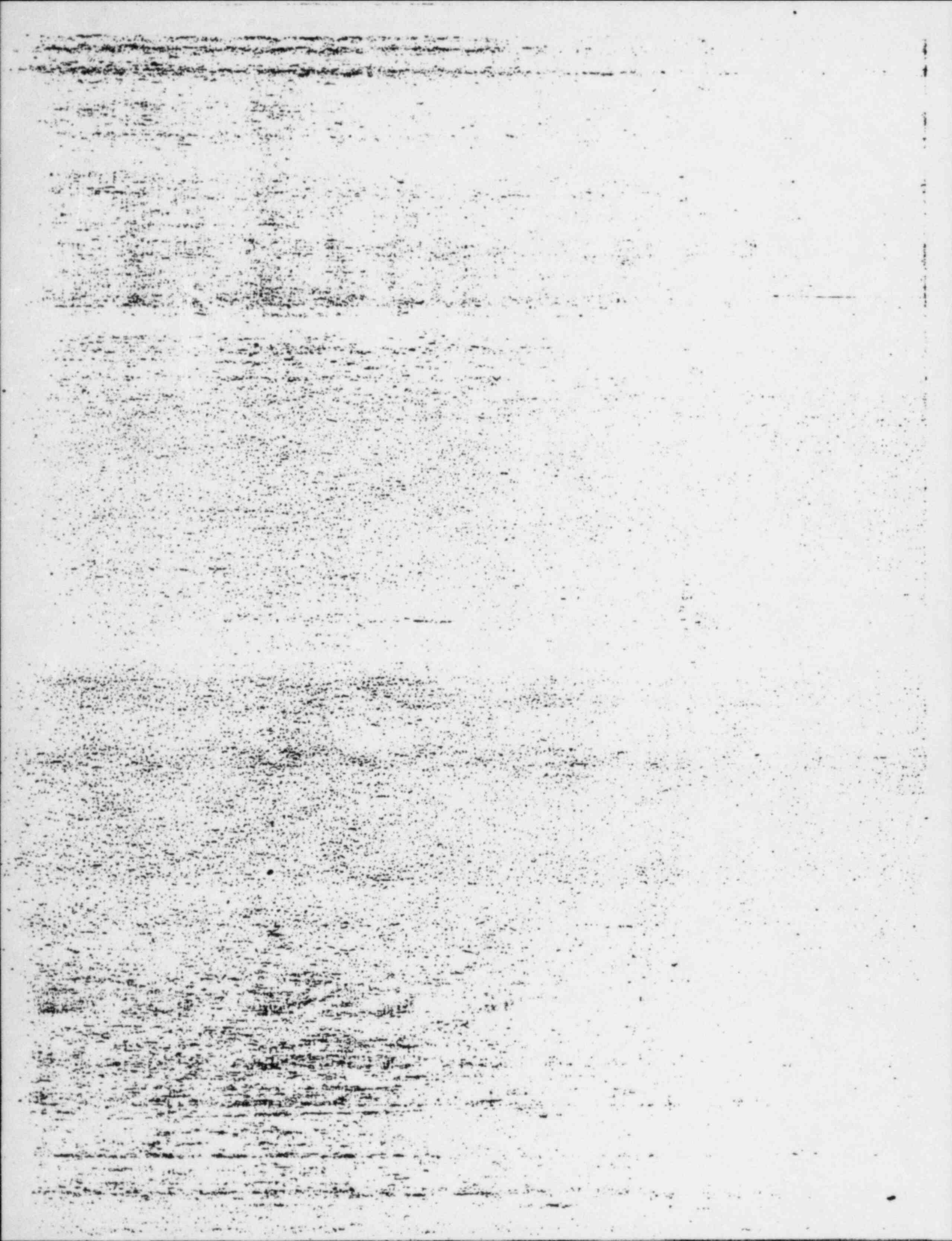
Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 148

Priority: 3A

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: SER 16 3/4.7 (New Page required)

Problem Title: Turbine Stop and Control Testing

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Biweekly testing of turbine and low pressure stop and control valves must be included in Tech Specs.

2. Safety Significance:

Does not conform to SER commitments.

3. Anticipated Resolution:

Is commitment valid and what is required frequency of testing.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 149

Priority: 3B

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: SER 16 3/4.3 Addressed to Instrumentation Spec

Problem Title: Tech Spec Instrumentation 3/4.3

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Availability, set points, and surveillance of turbine bypass system, thermal power monitor, and level of water level trip must be addressed in Tech Specs (Item 397 - previously closed, addressed thermal power monitor).

2. Safety Significance:  
Unknown

3. Anticipated Resolution:  
Investigate, submit Technical Specification changes are required.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_  
Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 150

Priority: 2F

\_\_\_\_\_  
Identified By \_\_\_\_\_  
Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: SSER 4 22.2 II

Problem Title: ECCS Instrumentation 3/4.3.3 Table

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Before start-up from first refueling outage, provide proposed Tech Spec surveillance procedures for ADS manual inhibit switch and timer (new timer)

2. Safety Significance:

NONE

3. Anticipated Resolution:

Investigate

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 151

Priority: 3B

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 2.2.1-1.2.C, SER 7.2.2, FSAR 7.2

Problem Title: APFM Power Setpoint

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

SER states "Appor signal trip setpoint to be specified in Tech Specs, but no more than 112.5% power according to current FSAR". Tech Spec, however, lists allowable setpoint as 113% power.

2. Safety Significance:

NONE: Tech Specs are written to current G.E. design documentation.

(G.E. Design Spec data sheets 22A3739AE, Rev. 4, Sheet 11)

3. Anticipated Resolution:

Propose change to FSAR/SER.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 152

Priority: 2D

Identified By \_\_\_\_\_ / \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.1.1.b (Bases 3/4.1.1 page B 3/4 1-1)

Problem Title: Shutdown Margin Delta k/k for BWR/3

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
The referenced Spec is a BWR/3 Spec. The bases to this Spec should explain this situation.

2. Safety Significance:  
NONE: Bases change only for clarification.

3. Anticipated Resolution:  
Determine and implement bases rewording in order to clarify issue.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 153

Priority: 2G

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.1.2

Problem Title: Exxon fuel will probably not use term "Rod Density"

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Exxon fuel reactivity anomoly checks may not be based on "Rod Density"

2. Safety Significance:

None for first cycle - need to be worded correctly for cycle 2.

3. Anticipated Resolution:

Put in appropriate wording in a generic submittal that handles Exxon fuel.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: _____	/	_____
Individual Notified	Date	Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

_____	/	_____
Date		Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 154

Priority: 2C

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3.13.2.C.4

Problem Title: Slow Rod Action Statement Incorrect Reference

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 3.1.3.2.C.4 references 3.1.4.3.C, but should reference 4.1.3.2.C

2. Safety Significance:

None, Typo error as determined by review of STS.

3. Anticipated Resolution:

Correct Typo, investigate possibility of changing wording of 3.1.3.2.C.4 to add clarification.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 155

Priority: 2C

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.1.3.5.a.1

Problem Title: Replace "Alternate control rod Position Indicator"

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Alternate method more general and allows more methods to determine position.

2. Safety Significance:

None, enhancement being considered for operational convention. Change may not be justified.

3. Anticipated Resolution:

Revise Spec to say "Determine the position of the control rod by and alternate method.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

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\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 156

Priority: 2C

\_\_\_\_\_  
/\_\_\_\_\_  
Identified By                                  Date

\_\_\_\_\_  
Responsible Supervisor

Tech Spec Reference: 3.1.5

Problem Title: Standby Liquid Control Sys Loops vrs. Components

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The action statement refers to specific components and cause a loop to be shutdown with other than a pump on valve inoperable. Allow specific components being inop if one operable system exists without entering action statement a.2.

2. Safety Significance:

NONE

3. Anticipated Resolution:

Reword AI to include more than pumps and valve, investigate.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                                  Date                                  Time

5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

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/\_\_\_\_\_  
Date                                  Time

cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 157

Priority: 2C

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.1.5 & 3.2.3

Problem Title: Illegible Figure 3.1.5-1

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

1) Figure 3.1.5-1 is not legible. 2) Curves in 3.2.3 need to be of better quality and indicate acceptable/unacceptable regions.

2. Safety Significance:

None, enhancement only.

3. Anticipated Resolution:

Provide a legible figure for 3.1.5-1 and new curves for 3.2.3.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 158

Priority: 2C

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.2.2

Problem Title: APRM Setpoint "T" statement conflicts with STS

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
The "T" statement is in conflict with the STS. It should be "T is applied when less than or equal to 1".

2. Safety Significance:  
None, enhancement only. Change may not be justified.

3. Anticipated Resolution:  
Reword "T" statement to conform to STS statement.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_  
\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 159  
Rod Brown /

Priority: 2C

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.4.3.1.a

Problem Title: Elimination of Drywell Atmosphere Particulates Monitoring

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Elimination of drywell atmosphere particulates monitoring subsystem for monitoring reactor coolant system leakage for additional operational consideration. Change made to Limerick Tech Spec.

2. Safety Significance:

None: Enhancement only for operation considerations.

3. Anticipated Resolution:

Safety to obtain Limerick Tech Spec change and SER for proposed Tech Spec Change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 160

Priority: 2D

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.4.6.1 Figure 3.4.6.1-1 & Bases 3/4.4.6

Problem Title: Reactor Pressure vs. Metal Temp

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Figure 3.4.6.1-1 has only 3 lines on curve, but bases is worded as if there are 6 lines.

2. Safety Significance:

None, enhancement only

3. Anticipated Resolution:

Investigate the problem to see if either bases or curve or both need to be changed.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION

Item Number: 161

Priority: 2D

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.4.9.2 Bases 3/4.4.9

Problem Title: Shutdown Cooling Bases Inaccurate on Coolant Mixing

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

To be inaccurate with respect to shutdown cooling resulting in coolant mixing to provide accurate temperature indicator.

2. Safety Significance:

None, clarification needed in bases. Check further to see if required.

3. Anticipated Resolution:

Investigate problem to determine if bases are inaccurate.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 162

Priority: 2C

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3.5.1 Action b.3 Note

Problem Title: Note on RHR Subsystems Referenced in Wrong Place

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The note is confusing because with 2 of the 3 RHR loops inoperable, the other loop may be one of the shutdown cooling loops, therefore, the note should also be referenced in action statement A3.

2. Safety Significance:

None, enhancement only.

3. Anticipated Resolution:

Correct reference location.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 163 Priority: 2C

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Identified By \_\_\_\_\_ / \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Date

Tech Spec Reference: 3.5.3.b

Problem Title: Suppression Pool Action Statement

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
3.5.3.b should also reference 4.5.3.2, since it is an action statement.

2. Safety Significance:  
None, enhancement only.

3. Anticipated Resolution:  
Provide correct reference.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: _____	/	_____	_____
Individual Notified		Date	Time

5. Disposition: \_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

_____	/	_____
Date		Time

cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 164

Priority: 2A

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3.6.1.3.b, 3.6.1.1, 3.6.1.6

Problem Title: Primary Contain Integrity Definition Inconsistency

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
The three Tech Specs have various time limits on restoration of primary containment integrity, and should be consistent.

2. Safety Significance:  
Specs as is are confusing to operators.

3. Anticipated Resolution:  
Revise Specs to be consistent, or change bases to justify different times and provide instructions on how to use these Specs.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 165

Priority: 2C

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 4.6.1.4.d

Problem Title: Channel Check on MSIV Leakage Control System

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Instrumentation is always pegged high (measure), so as a channel check on pressure instrumentation has no purpose. Also, same problem may exist on flow instrumentation.

2. Safety Significance:

None, enhancement only.

3. Anticipated Resolution:

Delete channel check on pressure, reword Statement d. Investigate problem with flow to determine if it should also be deleted.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 166

Priority: 3B

NRC/CSB / \_\_\_\_\_

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 4.6.1.5

Problem Title: Feedwater Leakage Control System Interlocks

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Spec does not contain a requirement to test the system interlocks.

REFERENCE: October 18, 1983 Telecon, Guy Cesare and NRC.

2. Safety Significance:

Licensing issue.

3. Anticipated Resolution:

There is currently an open design issue on feedwater leakage control system that may require a Tech Spec change and testing the FO65's (FW check valves) during an outage. Licensing to resolve this issue.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 167

Priority: 2A

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.6.2.1 & 3.6.2.3.b

Problem Title: Drywell Integrity Restoration

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Drywell integrity restoration times conflict in these Specs.

2. Safety Significance:

Specs as is are confusing to operators.

3. Anticipated Resolution:

Revise Specs to be consistent, or change bases to justify times and provide instructions on how to use these Specs.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 168

Priority: 1B

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.6.3.1

Problem Title: Suppression Pool Temp Require in OP condition 3

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Spec is confusing because it doesn't contain any action statements for op condition 3, although the operator is instructed to go to (old shutdown).

2. Safety Significance:

Operating instructions are confusing.

3. Anticipated Resolution:

Investigate to see if statement needs rewording or if CCO should be applicable in op condition 3 as well as 1 and 2.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 169

Priority: 2C

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.6.3.2

Problem Title: Containment Spray

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec should include spray spargers to be consistent with STS.

2. Safety Significance:

No enhancement only.

3. Anticipated Resolution:

Revise Tech Spec to include spray spargers.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 170

Priority: 2D

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

Responsible Supervisor

Tech Spec Reference: 3.6.4

Problem Title: Table 3.6.4-1 Bases Definitions

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The bases/definitions should include definitions of the terms "manual", "auto", and "other isolator valves".

2. Safety Significance:

None, enhancement only.

3. Anticipated Resolution:

Reword bases to include these terms.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 171

Priority: 2C

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.6.4

Problem Title: Table 3.6.4-1 E61 FC56 an FC57 Valve Description

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The valve description for these valves are incorrect.

2. Safety Significance:

None, enhancement.

3. Anticipated Resolution:

Change words in tables to "Purge Filter Train ISOL."

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 172

Priority: 2A

\_\_\_\_\_  
Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.6.2.2 (FSAR 6.2.6.5.1)

Problem Title: Definition of A/k

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Tech Spec does not define what A/ k is Tech Spec makes reference to the drywell bypass leakage as acceptable if leakage is less than or equal to 10% of A/ k of 9 ft. This is confusing since leakage should be as SCFM.

2. Safety Significance:

A/ k terms is not defined surveillance procedures includes provisions for determining leakage.

3. Anticipated Resolution:

Change acceptance leakage to less than 3500 SCFM.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 173

Priority: 2C

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.7.1.1.b

Problem Title: SSW Spec refers to ECCS Pump Room Seal Cooler

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Spec refers to ECCS pump room seal coolers and associated coolers and pump heat exchangers. Spec needs to be revised to address specific plant equipment for operability.

2. Safety Significance:

None, provide clarification

3. Anticipated Resolution:

Revise Spec to properly reference correct ECCS equipment.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

	Date	Time
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5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 174

Priority: 2D

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.8.1.1.b & 3.8.1.2.b

Problem Title: DG Useable Fuel Number

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Spec does not indicate that the numbers for fuel tank capacity are for "useable" fuel.

2. Safety Significance:  
None, enhancement only.

3. Anticipated Resolution:  
Change bases to indicate that the numbers provided are for "useable" fuel.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
  Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
    R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 175

Priority: 3B

\_\_\_\_\_ / \_\_\_\_\_

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3.8.1.1

Problem Title: Action Statements Performing Sur Require on D/G

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Time limits on action statements a,b,d,e and f are too constrictive, all DG's cannot be tested within allowable time limits.

2. Safety Significance:

Ops cannot meet time constraints.

3. Anticipated Resolution:

Investigate problem. Will need to change test methods or time constraints.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 176

Priority: 2A

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.8.1.2

Problem Title: Change Operation Above Containment Pool

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Spec Action Statement a should include the suspension of operator of the crane above the upper containment pool as it does for the spent fuel pool crane.

2. Safety Significance:

None, enhancement only.

3. Anticipated Resolution:

Revise action statement to include this action.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 177

Priority: 2C

\_\_\_\_\_  
Identified By

\_\_\_\_\_  
Date

\_\_\_\_\_  
Responsible Supervisor

Tech Spec Reference: 3.8.2.1

Problem Title: Required Changes Inoperable Statement Inconsistent

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Action Statement C on 3.8.2.2.c is not worded consistently with action statements on 3.8.2.1.c.

2. Safety Significance:

None

3. Anticipated Resolution:

Revise statements to be consistent.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date

\_\_\_\_\_  
Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 178

Priority: 2C

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3.8.4.2 (3/4.8.4.2)

Problem Title: Mov Thermal Overload Protection Spec.

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The above spec appears to have been formed by combining 2 of the STS specification together. This spec is now confusing.

2. Safety Significance:

Interpretation of Spec confusing.

3. Anticipated Resolution:

Investigate and determine appropriate changes.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 179

Priority: 2C

Identified By \_\_\_\_\_ / \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.8.4.2 (Table 3.8.4.2-1)

Problem Title: Table 3.8.4.2-1 Valve on Turbine Designation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The term "valve on turbine" is confusing because more than one valve fits this description.

2. Safety Significance:

None, enhancement only.

3. Anticipated Resolution:

Reword table description to properly identify valve.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 180

Priority: 1D

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 4.8.4.3

Problem Title: BPS Undervoltage Setpoint

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The under-voltage setpoint should be changed to reflect a more accurate operating range.

2. Safety Significance:

None, enhancement only.

3. Anticipated Resolution:

Revise setpoint.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 181

Priority: 2E

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.8.4.3

Problem Title: RPS Elect. Power Monitor-Switch from IMG to Alter.

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

1) Compliance with action statement a. could cause a full scram. 2) No bases to Spec. Full scram could be caused by transferring to alternate supply if half scram was present from say instrumentation sections. Add note to specific transfer, etc. does not have to take place for some time frame if it will result in full scram.

2. Safety Significance:

None, provide clarification.

3. Anticipated Resolution:

Provide bases and correct action statements, on delete spec. Investigation of options is required.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 182

Priority: 2C

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3.9.1.b.1

Problem Title: "All Rods In" vs "One Rod Out" Interlock

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

This is a BWR4 term and is not used on a BWR6 - Should be "Continuous Withdrawal".

2. Safety Significance:

None, enhancement only.

3. Anticipated Resolution:

Change spec wording.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 183

Priority: 2C

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Identified By                      Date

\_\_\_\_\_  
Responsible Supervisor

Tech Spec Reference: 4.10.2.a

Problem Title: Rod Pattern Control System Surveillance Procedure

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

STS and Spec Surveillance a. are not worded the same and should be consistent.

2. Safety Significance:

None: Enhancement.

3. Anticipated Resolution:

Reword 4.10.2.a to match STS.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 184

Priority: 2C

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: T.S. 3.10.3.c

Problem Title: "Rod-Out-Notch Override" is a BWR 4 Term

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

This is a BWR4 Term and is not used on a BWR6.

2. Safety Significance:

None: Enhancement only.

3. Anticipated Resolution:

Reword Spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84





TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 186 \_\_\_\_\_

Priority: 2A \_\_\_\_\_

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4

Problem Title: Review AECM 80/0026 for Tech Spec Changes (TMI).

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Need to Review AECM 80/0026 for required Tech Spec changes from implementing TMI items 1-10 of I&E Bulletin 79-80 and other TMI related items. See Attachment.

2. Safety Significance:

The safety significance will have to be determined during the review of letter AECM 80/0026 Item by Item.

3. Anticipated Resolution:

Review letter AECM 80/0026.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. R. Cross

R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 187

Priority: 2F

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.7 (New Spec)

Problem Title: Annual Sampling of Instrument Air Quality

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Submit proposed Tech Spec and design for commitments in AECM 82/490, 510 and 528 four months prior to startup after first refueling. MP&L commits to annually sample for instrument air quality commencing at first operating outage. Sample will be taken down-stream of the Air dryer of & ch filtering and analyzed for dew point particle size and oil content.

2. Safety Significance:

3. Anticipated Resolution:

Prepare Tech Spec changes.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 188

Priority: 3B

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.1.3-1; Tables: 3.3.1-2; 3.3.6-1; 3.3.6-2

Problem Title: SDV Tech Spec Changes

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

NRC in MAEC 80/260 proposed improvement or changes of the Tech Spec for the purpose of strengthening the provisions for assuring continued operability of the CRD System. Change proposed to sections for Control Rods, RPS Response Times; Control Rod Withdrawal Blocks.

2. Safety Significance:

None: Enhancement to improve operability.

3. Anticipated Resolution:

Review AECM 80/260 and determine if associated changes are adequately covered.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 189

Priority: 2G

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 2.2.1; 3.3.4.2 Table:2.2.1-1; 3.3.4.2-1; 3.3.4.2-2

Problem Title: Turbine Stop Vlv. & Control Vlv. Final Setpoints

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Any required change to Turbine Stop Valve or Control Valve setpoints in Tech Spec Tables 2.2.1-1 and 3.3.9.2-2 will be submitted to the NRC within 90 days of test completion.

2. Safety Significance:

None

3. Anticipated Resolution:

Ensure Startup test program addresses setpoint changes and submittal to NRC upon completion of test.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 190

Priority: 2C

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.11.1.2 & Table 3.12.1-2

Problem Title: Drinking Water Report

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

1) Drinking water may be taken from the Mississippi River within 3 mile of Plant discharge, which would require an appropriate analysis in the Spec if true, as noted in the PWR STS 3.11.1.2. 2) Table 3.12.1-2 will need to be changed to  $3 \times 10^4$  if 1) shows no drinking water being taken within 3 mile.

2. Safety Significance:

None

3. Anticipated Resolution:

Investigate to see if drinking water is being taken 3 miles from Plant Discharge.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 191

Priority: 1B

Rod Brown / \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.11.2.1.b

Problem Title: Dose Rate From All Radioiodines

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
ODCM only addresses 2 Radioiodines, but Spec states all radioiodines.

2. Safety Significance:  
Could show that we are operating non-conservatively need to investigate.

3. Anticipated Resolution:  
Investigate - may need to change Spec and bases.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 192

Priority: 2C

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 4.11.2.4

Problem Title: Operability - Gaseous Radwaste Treatment

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Present Tech Spec Surveillance requires that the Offgas System components be demonstrated operable by operating components for at least 30 minutes during last 52 days. The standard RETS (PWR) says that operability can be met by just meeting release limits of 3.11.2.1, 3.11.2.2, and 3.11.2.3.

2. Safety Significance:

None: Enhancement

3. Anticipated Resolution:

Tech Spec Change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 193

Priority: 2C

\_\_\_\_\_  
 Identified By                          Date                          Responsible Supervisor

Tech Spec Reference: 3.11.2.6

Problem Title: Applicability Change for Explosive Gas Mixture

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 3.11.2.6 concerning concentration of hydrogen in the main condenser offgas treatment system lists the applicability as "at all times" while Tech Spec table 3.3.7.12-1.2.a lists the applicability as "During main condenser offgas treatment system operation."

2. Safety Significance:

None

3. Anticipated Resolution:

Change Tech Spec 3.11.2.6 applicability to agree with Tech Spec Table 3.3.7.12-1.2.a.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                          Date                          Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
 Date                          Time

cc: J. E. Cross  
 R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 194

Priority: 2C

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.12.1

Problem Title: Food Products Sampling

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec and sampling frequency required by Table 3.12.1-1 are misleading. Table requires sampling products monthly when available. This implies that when products are not available then no sampling is required. This conflicts with the associated LCO action statement which requires a special report. The table need revised to delete "when available".

2. Safety Significance:

None: Clarification only.

3. Anticipated Resolution:

Review and change as necessary.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 195

Priority: 3A

R. Keeton / \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.6.1.4

Problem Title: MSIV-LCS Operability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
MSIV-LCS Tech Spec does not ensure that the MSIV-LCS is operable (Ops Superintendent Concern). Surveillance Procedure may be inadequate.

2. Safety Significance:  
None Tech Spec Enhancement

3. Anticipated Resolution:  
Resolve with identifier.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 196

Priority: 2A

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: Table 3.3.7.3-1

Problem Title: Meteorological System Instrumentation Operability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Change "Minimum InstruMents Operable" to "Min Channels Oper", one instrument does not provide information required by Tech Specs should say channel.

2. Safety Significance:  
None, provide clarification to existing criteria.

3. Anticipated Resolution:  
Submit Tech Spec Change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 197

Priority: 3/A

\_\_\_\_\_  
Identified By \_\_\_\_\_ Date \_\_\_\_\_

\_\_\_\_\_  
Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.3.6-1

Problem Title: Rod Block Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Rod blocks from refueling platform hoist loaded & over core not included in table, Tech Spec 4.9.1 checks interlocks which include rod blocks.

2. Safety Significance:

None, all requirements satisfied but should add clarification.

3. Anticipated Resolution:

Discuss with NRC for interpretation.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_  
\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 198

Priority: 1B

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: Table 3.3.7.1-1 items 6,7,8,9

Problem Title: Radiation Monitor

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Min channels required operable for items 6,7,8,9 not the same even though logics are the same.

2. Safety Significance:

Investigation required if 4 minimum channels are required and since presently we only require 3 channels operable, we could be operating unconcernatively in relation to FSAR analysis.

3. Anticipated Resolution:

Investigate; propose Tech Spec change if required.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 199

Priority: 1B

Steve Logan /

Identified By

Date

Responsible Supervisor

Tech Spec Reference: Table 3.3.6-1.5

Problem Title: Scram Discharge Volume Bypass

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Difference between STS and our Tech Specs. GGNS Tech Spec Table 3.3.6-1 does not contain Item 5B as in the STS. Should the GGNS Tech Spec contain 5B? If so, what are the min op chan/trip function, applicable conditions, actions Tech Spec Table 4.3.6-1 should be revised to include surveillance requirements.

2. Safety Significance:

Interlock being verified in Surveillances every 18 months during LSFT, a discrepancy does exist in which non-conservative operation of the plant might occur.

3. Anticipated Resolution:

Submit Tech Spec change to incorporate Scram Discharge Volume bypass so as to conform with Standard.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date

Time

cc: J. E. Cross

R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 200

Priority: 2F

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3/4.3.4

Problem Title: ATWS-RPT System

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
ATWS Recirc Pump Trip System changing logic. Add "manual initiation" to ATWS trips.

2. Safety Significance:  
None, DCP will have safety evaluation completed.

3. Anticipated Resolution:  
DCP incorporation will add manual initiation switch, NRC approval will be required prior to installation, via Tech Spec change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 201

Priority: 2A

\_\_\_\_\_  
Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

\_\_\_\_\_  
Responsible Supervisor

Tech Spec Reference: Table 3.3.2-1.2.e

Problem Title: Secondary Isolation (Containment) Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Manual initiation switches (1B21-HS-M630 A, B, C, D) does not trip mechanical vacuum pumps. Delete note (f) from item (e).

2. Safety Significance:

None until problem researched.

3. Anticipated Resolution:

Investigate problem further submit Tech Spec change if required.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 202

Priority: 1B

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: Table 3.3.7.5-1.4

Problem Title: Post Accident Monitoring Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
12 channels of suppression pool temp noted in FSAR Tech Spec only reference 6. Should "6, 1/sector" be changed to "12, 2/sector" under "required number of channels"?

2. Safety Significance:  
FSAR presently lists 12, Tech Specs could be non-conservative in relation to FSAR.

3. Anticipated Resolution:  
Further investigation required; submit Tech Spec Change if required.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 203

Priority: 2C

\_\_\_\_\_ / \_\_\_\_\_

\_\_\_\_\_

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3.7.6.2.c.3

Problem Title: Control Building Spray/Sprinkler System

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Elevation 93 - N1P64D140 should be NSP64D140, since the sprinkler is shared.

2. Safety Significance:

None: Enhancement only.

3. Anticipated Resolution:

Change Spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 204

Priority: 2G

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Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.3.2-2

Problem Title: Final Setpoint Change During Startup Program

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Submit Tech Spec changes associated with setpoint changes within 90 days of test completion.

2. Safety Significance:

None; setpoints from operation of plant could cause operating at restrictive limits until change received.

3. Anticipated Resolution:

Submit table changes to NRC upon completion of tests.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 205 / \_\_\_\_\_ Priority: 2G \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: Table 4.3.3.1-1 items C.1.b & C.1.f

Problem Title: HPCS Wide-Range-Level Trans Non-Density Compens.

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Table C.1.b and f note ## must be changed to resolve HPCS wide-range-level instrument non-density compensation. First refueling outage note.

2. Safety Significance:

3. Anticipated Resolution:  
Resolve note and change spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 206

Priority: 2F

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: Table 4.3.3.1-1 note (c) & (d)

Problem Title: LPCI/LPCS Injection Valve Interlocks

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Future design change to the LPCI and LPCS injection valve interlocks will require change to notes (c) & (d) of Table 4.3.3.1-1.

2. Safety Significance:

3. Anticipated Resolution:

Submit Tech Spec change prior to design change incorporation, which is not expected until after first refuel outage.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 207  
\_\_\_\_\_ / \_\_\_\_\_

Priority: 2G  
\_\_\_\_\_

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3.3.7.1-1

Problem Title: Radiation Monitors Final Setpoints

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Submit Tech Spec change to provide final setpoint changes to the NRC within 90 days from completion of startup tests. (See item 204)

2. Safety Significance:

None, will not be allowed to operate in excess of existing Tech Spec limit could cause restrictive operations.

3. Anticipated Resolution:

Submit changes upon completion of tests.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 208

Priority: 2G

\_\_\_\_\_ / \_\_\_\_\_

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 4.7.1.1.a.2

Problem Title: SSW Spent Fuel Storage Pool Coolers Iso. Valves

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Need to submit Tech Spec change to reflect the installation of new SSW pumps. Tech Spec 4.7.1.1.a.2 was incorporated to reflect the old undersized SSW pumps.

2. Safety Significance:

3. Anticipated Resolution:

Delete Tech Spec 4.7.1.1.a.2 upon installation of the new SSW pumps.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 209

Priority: 2G

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.6.5

Problem Title: Drywell Post-LOCA Vacuum Breakers

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Ensure that the new design to be installed at the first refueling outage is adequately covered by spec 3/4.6.5 because of expiration of exception on vacuum breaker position indication.

2. Safety Significance:

3. Anticipated Resolution:

Review new design change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 210

Priority: 2F

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3.3.1-1;9

Problem Title: Future Insta. of Add. Scram Disch. Vol. Lev. Trip

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Need to submit Tech Spec change to reflect DCP 81/5007. This design change will install additional scram discharge volume level trip switches.

2. Safety Significance:

3. Anticipated Resolution:

Provide Tech Spec change upon design change completion.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 211

Priority: 2A

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: Table 3.3.2-1 (note (e) & Table 3.3.7-1 note (h)

Problem Title: Radiation Monitoring MOC Notes

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Table 3.3.2-1 note (e) and Table 3.3.7-1 note (h) states "Two upscale Hi-Hi-, one upscale-Hi Hi and one downscale, or two downscale signals from the same trip system...". This statement is incorrect. Downscale trip only actuates an alarm. Statement should read, "Two upscale-Hi Hi, one upscale-Hi Hi and one Inop, or two Inop signals from the same trip system actuate the trip system...".

2. Safety Significance:

Could be Confusing.

3. Anticipated Resolution:

Change notes in Tech Specs to reflect correct plant design.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 212

Priority: 2C

Bechtel /

Identified For \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.3.2-1

Problem Title: Trip Channels vrs. Trip System

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Action statements in table are overly restrictive. Spec 3.3.1 and 3.3.3 action statements, generally allow the trip channel to be placed in the trip condition, rather than the entire trip system. It would be beneficial to allow tripping only the inoperable channel rather than the tyrip system, since not all channels close all isolation valves.

2. Safety Significance:

Plant Enhancement.

3. Anticipated Resolution:

Requires further analysis.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 213

Priority: 1B

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3.3.3-1

Problem Title: Incorp. Represent. of ECCS Man. Init. Logic

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec Table 3.3.3-1 Items 3.3.3-1.A.2.9 and 3.3.3-1.B.2.f list minimum operable channels per trip system as 1/valve per plant design this should be 2/system.

2. Safety Significance:

Since ECCS Manual Initiation Logic (ADS) requires two manual initiation inputs, the plant could be operated in a non-conservative condition.

3. Anticipated Resolution:

Change Tech Spec Table 3.3.3-1 to reflect as built design of plant.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 214

Priority: 3B

Jim McMahan (QA) /

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 4.1.3.3.b.2

Problem Title: Reactivity Control Sys. Acceptance Criteria

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Spec does not agree with STS and does not list any acceptance criteria for the control rod scram accumulator. Check valve pressure check.

2. Safety Significance:

QA does not think this spec meets requirements of 10CFR50.36. Tech support does not agree and does not feel a change is justified.

3. Anticipated Resolution:

Investigate and rewrite spec, if necessary.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 215

Priority: 3B

Identified By \_\_\_\_\_

Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.2.2

Problem Title: APRM Setpoints - Thermal Power Time Constant

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Proof and review comment from RSB - the time constant for the thermal power monitor need to be included in the LCO's and surveillance requirements.

2. Safety Significance:

None: Item already included.

3. Anticipated Resolution:

This item is already in Table 4.3.1.1-1 (note 1). Provide a response to the NAC.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 216

Priority: 3B

Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3/4.3.7.5 & 6.8.3

Problem Title: Post Accident Monitoring TMI Proposed Changes

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Provide Tech Spec changes according to recommendations in memorandum from R. Mattson to D. Eisenhurt (October 12, 1983) to address operability of post accident monitoring instrumentation required under NUREG-0737, Supplement 1. Proof and review comment from ICSB, October 31, 1983.

2. Safety Significance:

None: Enhancement only.

3. Anticipated Resolution:

Determine what recommended changes are and evaluate to see if the Tech Specs should be changed.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: _____	/	_____
Individual Notified	Date	Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

_____	/	_____
Date		Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 217

Priority: 3B

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.3.1

Problem Title: Actuation Logic and Relay Tests

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

GGNS Tech Spec does not include Actuation Logic Test and Master Relay and Slave Relay Tests as part of the instrumentation surveillance requirements, as does the Westinghouse STS. Proof and review comment from ICSB, October 31, 1983.

2. Safety Significance:

None: See anticipated resolution.

3. Anticipated Resolution:

Send response to NRC stating this change is not justified.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 218

Priority: 3A

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.3

Problem Title: Study on Protective Trip Setpoint-Allowable Vlvs.

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Revise Tech Spec's trip setpoints and allowable value to include only the drift associated with that portion of the instrument channel tested at 31 day intervals. Proof and review comment from ICSR, October 31, 1983.

2. Safety Significance:

See anticipated resolution.

3. Anticipated Resolution:

Tech Spec change will be provided in accordance with the final results of the BWROG Review.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

\_\_\_\_\_  
\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 219

Priority: 1B

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: Figure 3.4.6.1-1

Problem Title: Pressure/Temperature Limit Curves

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The pressure-temperature limit curves do not comply with the closure flame pressure-temperature safety margins in Program IV.A.2 of App. G to 10CFR50. Proof and review comment from MEB, December 15, 1983.

2. Safety Significance:

3. Anticipated Resolution:

Evaluate to determine if the figure will require revision.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 220 Priority: 3A

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Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.7.6 (Add New Section)

Problem Title: Portable Fire Extinguisher & Emergency Lighting

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Recommendation that Tech Specs be provided for portable fire extinguishers and emergency lights. Proof and review comment from CER, November 7, 1983.

2. Safety Significance:

3. Anticipated Resolution:

Submit justification to NRR for not changing Tech Specs.

4. NRC Response to Item (NRR/IE) \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 221

Priority: 2C

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3.4.3.2.(c)

Problem Title: Reactor Coolant System Leakage

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

According to NRC (Division of Systems Integration) letter dated Oct. 26, 1983, Tech Spec 3.4.3.2.(c) should read: "25 gpm total leakage (averaged over any 24-hour period)". Presently Tech Spec 3.4.3.2.(c) reads: "30 gpm total leakage..".

2. Safety Significance:

None: Measurement at any time is more conservative than average over any 24 hour period.

3. Anticipated Resolution:

Change spec for added flexibility and clarity.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

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Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 222

Priority: 3B

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.7.2

Problem Title: Control Room Emergency Filtration System

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The following statement should be added to surveillance requirements: "At least once per 12 hours by verifying that the control room air temperature is less than or equal to 120°F." Proof and review comment for OSI, Oct. 26, 1983.

2. Safety Significance:

3. Anticipated Resolution:

Send a response to NRR stating that requirement is satisfied by Spec 3.7.8.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 223

Priority: 2A

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 4.7.6.1.1.C

Problem Title: Add Surveillance Require to Fire Suppresion Sys

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

CED requested addition of following surveillance requirement to paragraph 4.7.6.1.1.C. "At least once per 6 months by performance of a system flow test."

Proof and review comment from CEB, Nov. 7, 1983.

2. Safety Significance:

The request from CEB is not clear.

3. Anticipated Resolution:

Determine what the request is.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 224

Priority: 3B

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.7.6.4

Problem Title: Add Surveillance Require to Halon Sys Stor Tank

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The CEB requested that the following be added to the surveillance requirement:  
"At least once per 6 months by verifying that the Halon Storage Tank weight to be at least 95% of full change weight and pressure to be at least 90% of full change pressure."

Proof and review comment from CEB, Nov. 7, 1983.

2. Safety Significance:

None

3. Anticipated Resolution:

Send response to NRR stating this change is unnecessary. The requirements are in Tech Spec 3.7.6.4 for operability.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 225

Priority: 2C

NRC (I&E plus NRR)                      /1/24/84

Identified By

Date

Responsible Supervisor

Tech Spec Reference: Fig. 5.1.3-1

Problem Title: Illegible Figure Unrestricted Area Boundary

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The associated Tech Spec figure showing the unrestricted area boundary for gaseous effluents and for liquid effluents is illegible.

2. Safety Significance:

None: Enhancement

3. Anticipated Resolution:

Submit new figure.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 226

Priority: 3A

NRC / 2/24/84

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.8.4.3.b

Problem Title: Necessary for RPS Electric Power Monitoring Spec

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The NRC is of the opinion that this spec is probably not required and the necessity for the spec should be reviewed. If in our opinion, however, the spec is justified and required, that the NRC believes that the associated time delays should also be included in the surveillance requirements.

2. Safety Significance:

None; FPA's were added per a request from NRC and only voltage (over & under) and under frequency were requested. This spec is per STS and the time delays have no interaction on plant transient response.

3. Anticipated Resolution:

Investigate deleting spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 227

Priority: 3B

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 4.8.2.1.C.4

Problem Title: Battery Charger Surveillance Requirement

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Region II inspector indicated during exit that Tech Spec 4.8.2.1.C.4 was incorrect and should require 140 VDC instead of 105 VDC. The exit meeting was on Feb. 24, 1984.

2. Safety Significance:

The only critical value is current. Current must stay at a maximum while voltage follows some profile. The Tech Spec is adequate. The NRC inspector is incorrect in assuming that chargers are tested wrong.

3. Anticipated Resolution:

None

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

	Date	Time
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5. Disposition: \_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 228

Priority: 3B

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 3/4.8.4.2 (Table 3.8.4.2-1)

Problem Title: Valve Identifier Nomenclature

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec Table 3.8.4.2-1 list valve as QSP41-F189, NRC thinks valves should be listed as QSP41-F189B.

2. Safety Significance:

None: Correct valve MP&L identified is QSP41-F189 the "B" on the end of the MP&L relates to the divisional power supply and should not be included.

3. Anticipated Resolution:

None

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 229

Priority: 2A

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 3/4.6.1.4

Problem Title: MSIV LCS Spec.

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec requires two independent MSIV LCS. Surveillance 4.6.1.4.C.2 requires checking heater operability on each subsystem, however only the board MSIV-LCS has heaters.

2. Safety Significance:

Tech Spec does not agree with plant design.

3. Anticipated Resolution:

Review Tech Spec and determine if change should be implemented.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 230

Priority: CLOSED

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 4.6.1.1.a

Problem Title: Primary Containment Integrity

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec 4.6.1.1.a reads "After each closing of each penetration subject to type B testing . . . by leak rate testing the equipment hatch seals . . ."; "The equipment hatch seals" should be "The penetration seals". Type B testing does not relate to just equipment hatches.

2. Safety Significance:

None: The change will properly address the surveillance requirements.

3. Anticipated Resolution:

Submit Tech Spec change "The penetration seals".

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: Duplicate of 144.

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 231

Priority: 3B

NRC / 2/24/84

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.6.1.1.b

Problem Title: Containment Penetrations

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec states that "containment penetrations not capable of being closed by operable containment isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in position; NRC thinks "secured in position" should be changed to read "secured closed."

2. Safety Significance:

None: Change not justified.

3. Anticipated Resolution:

Resolve with NRC that no change is required.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 232 Priority: 3B

NRC (I&E) 12/24/84

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.6.4

Problem Title: Tech Spec Table 3.6.4.1 Automatic Isolation Valves

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

NRC (I&E) suggest that we delete all the valves in the Table 3.6.4-1 which are automatic isolation valves. In addition 3/4.6 would have to be revised to delete reference to other valves.

2. Safety Significance:

None: Enhancement GGNS Tech Spec per SIS.

3. Anticipated Resolution:

Review and close item with no action.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 233

Priority: 3B

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Identified By	Date	Responsible Supervisor
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Tech Spec Reference: 4.6.3.2.b, 4.5.1.b

Problem Title: RHR Flows for Containment Spray Mode

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Various Tech Spec sections (4.5.1.b, 4.6.3.2.b and 4.0.5) reference various RHR flows. NRC inspector had concerns regarding testing RHR for various flows.

2. Safety Significance:

Resolve with NRC resident inspection.

3. Anticipated Resolution:

Present surveillance test RHR pumps for high test flows, which pump could meet lower flow requirements.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
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5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

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Date	Time
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cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 234

Priority: 3A

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Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: 4.3.7.5-1.3, 4.5.3.1, 4.6.3.1.C

Problem Title: Suppression Pool Water Level

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

NRC inspector on 2/23/84 identified that our suppression pool level instrumentation does not meet Tech Specs and that FSAR is incorrect (6.2.7.5). His greatest concern was how we identify which channels of Supp Pool level instr. meet each tech spec.

2. Safety Significance:

Tech Spec clarification.

3. Anticipated Resolution:

Revise FSAR, ensure procedures are clearer as to which instruments meet which tech specs.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

/

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Date                      Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 235

Priority: 1B

Ron Davis /

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 4.6.1.3.a

Problem Title: Containment Air Locks

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The 72 hour time frame given is a 10CFR50 Appendix J requirement A # sign should be added to this time frame since 4.0.2 can not apply.

2. Safety Significance:

3. Anticipated Resolution:

Add # sign to 72 hours, submit Tech Spec change.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 236

Priority: 2A

Bill Brown /

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 4.4.1.2

Problem Title: Jet Pump Operability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec requires "Flow control valves to be in same position" if pump performance changes or resistance increases in one loop then flow controllers will change valve position(s) to maintain equal flow in each loop so normal condition will be flows equal-value positions equal - which will make flows different - we can run surveillance just as well with equal flows and differing valve position-proposed amendment avoids need to put flow loops in manual and adjust loop flows.

2. Safety Significance:

3. Anticipated Resolution:

Change Tech Spec to read "when both re-circulating flows are equal within accuracy of loop flow measurements.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 237

Priority: 2C

Ron Davis / \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: Table 3.3.6-2.2.C

Problem Title: Control Rod Block Instrumentation Setpoints

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The nominal trip setpoint listed in Tech Specs for APRM downscale should be 4% of rated thermal power not 5%. (The G.E. design spec data sheet 22A3739AE Sh. 11 Rev. 4 states 4%).

2. Safety Significance:

None: Present setpoint is conservative to that given in G.E. design documents.

3. Anticipated Resolution:

Change Tech Spec.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 238 Priority: 2C

Ron Davis / \_\_\_\_\_

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3.3.2.3 Note # (pg. 3/4.3-19)

Problem Title: Isolation System Instrumentation Response Time

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The # note at end of table references the wrong table should be 3.6.6.2-1 not 3.6.5.2-1.

2. Safety Significance:

None: clarification (typo).

3. Anticipated Resolution:

Change Tech Specs.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

\_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_

Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 239

Priority: 2C

Ron Davis / \_\_\_\_\_

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.3.8

Problem Title: Plant System Actuation Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Add cross reference to action b to LCO such that you can adequately get to containment spray spec.

2. Safety Significance:

None: Clarification

3. Anticipated Resolution:

Investigate possibility of adding reference to Tech Spec section 3/4.6.3.2.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 240

Priority: 2C

Ron Davis / \_\_\_\_\_

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 3/4.6.7

Problem Title: Containment & Drywell Hydrogen Recombiners

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

There is only a Containment Hydrogen Recombiner which also is used for the Drywell (indirectly) via Drywell Purge. Should Spec be rewritten to be more exact?

2. Safety Significance:

None: Clarification

3. Anticipated Resolution:

Research further. Decide.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 241

Priority: 2C

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.1.3.1

Problem Title: Control Rod Operability Clarification

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Action 3.1.3.1.a.1.c should also reference surveillance 4.1.3.1.2.b.

2. Safety Significance:

Add clarity to ensure compliance with action statement.

3. Anticipated Resolution:

Revise Tech Specs.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date                      Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 242

Priority: 3B

NRC (Gilinsky) / 3/8/84

Identified By

Date

Responsible Supervisor

Tech Spec Reference: B2.1.2-2

Problem Title: Thermal Power Discrepancy

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Bases Table B2.1.2-2 lists nominal Thermal Power as 3323 MWT. Definition 1.33 states rated Thermal Power is 3833 MWT. FSAR Figures 1.1-1 and 4.4-1 state rated is 3833 MWT.

2. Safety Significance:

None

3. Anticipated Resolution:

Consulted Joe Frey (G.E. - San Jose - Licensing): Both values are correct; B2.1.2-2 lists the value used in the standard model for BWR Fuel Clad Analysis, the value is not supposed to be plant specific; the analysis is sensitive to # of bundles, not plant specific thermal power.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 243

Priority: 1B

Identified By \_\_\_\_\_ Date \_\_\_\_\_ Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 4.4.7, 4.6.4.3

Problem Title: MSIU Stroke Time Definition

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
ASME code IWV-3413 defines full stroke "from initiation of the actuating signal to the end of the actuating cycle." G.E. design spec states "valve closing time (100% valve travel)" FSAR (large pipe break analysis) gives 5.5 sec. (.5 sec. for actuation delay) for MSIV closure. Should valve timing include actuation delays (i.e., time from handswitch in CLOSE until valve starts CLOSED.)

2. Safety Significance:

3. Anticipated Resolution:  
Resolve definition of valve stroke (ASME IWV-3413 vs. G.E. design spec vs. FSAR).

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 244

Priority: 2A

Identified By

Date

Responsible Supervisor

Tech Spec Reference: Tech Spec 4.7.6.4.a

Problem Title: Halon System Valve Positions

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Above Surveillance requires verifying that each valve in the flow path is in its correct position. However, there are no valves in the 20 PGCC Halon System and there are no valves in the Computer Room Halon System with external indicator. The valves are actuated pneumatically by N<sub>2</sub> cylinders.

2. Safety Significance:

3. Anticipated Resolution:

Review and determine appropriate changes.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 245

Priority: 2A

/

Identified By	Date	Responsible Supervisor
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Tech Spec Reference: Tech Spec 4.7.6.2.c.2

Problem Title: Sprinkler Systems Visual Inspection Dry Pipe.

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Above surveillance requires visual inspection of the dry pipe spray and sprinkler headers. N1P64D142A,B,C are the only dry pipe sprinkler systems listed in Tech Spec 3.7.6.2. All others are wet pipe systems.

2. Safety Significance:

3. Anticipated Resolution:

Review and determine appropriate changes.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
---------------------	------	------

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

/

Date	Time
------	------

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 246

Priority: 2A

/

Identified By	Date	Responsible Supervisor
---------------	------	------------------------

Tech Spec Reference: Tech Spec 4.7.6.2.c.1

Problem Title: Sprinkler Systems Automatic Valves

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Above surveillance requires verifying that automatic valves in the flow path actuate to correct position for a system functional test. N1P64D154 and 155 have no automatic valves in the flow path. Sprinklers' discharge occurs on opening of fused sprinkler heads. N1P64D150, 151, 152, 153, 158, 159, 154, 155, 136A,B and 14D: Simulated automatic actuation of system consists of opening inspector's test drain valve.

2. Safety Significance:

3. Anticipated Resolution:

Review and determine appropriate changes if any.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified	Date	Time
---------------------	------	------

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

/

Date	Time
------	------

cc: J. E. Cross  
R. F. Rogers

Rev. 2, 3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 247

Priority: 2A

\_\_\_\_\_  
Identified By                      Date                      Responsible Supervisor

Tech Spec Reference: Tech Spec 4.7.6.4.C.1

Problem Title: Halon System Initiation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Above Surveillance requires verifying that the system including associated ventilation system fire damper logic actuates automatically upon receipt of a simulated actuation signal. There are, however, no fire dampers associated with the 20 PGCC Halon Systems.

2. Safety Significance:

3. Anticipated Resolution:

Review and determine appropriate changes.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified                      Date                      Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date                      Time

cc: J. E. Cross  
R. F. Rogers



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 248

Priority: 2D

Quadrex ; API-82/1345 / 8/13/82

Identified By

Date

Responsible Supervisor

Tech Spec Reference: B.3/4.11.2.5

Problem Title: Misnumbered Tech Spec Bases

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

This Tech Spec bases has been misnumbered as "3/4.11.2.5" ; should be "3/4.11.2.6".

Also, "3/4.11.2.4" should be "3/4.11.2.4 and 3/4.11.2.5".

2. Safety Significance:

None

3. Anticipated Resolution:

Correct Tech Spec Bases.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_

Individual Notified

Date

Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross

R. F. Rogers

Rev. 2, 3/10/84



ATTACHMENT 4

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 021

Priority: 3X/B

\_\_\_\_\_  
Identified By / Date

\_\_\_\_\_  
Responsible Supervisor

Tech Spec Reference: 3/4.7.4

Problem Title:

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other)

New snubber, Q1E51G180R01, added per DCP 82/0546. (See item #21)

2. Safety Significance: DCP requires a Tech Spec change. Snubber may affect operability of RCIC.

3. Anticipated Resolution: Add new snubber to Tech Spec list of snubbers. *Only required because Tech Spec requires new snubbers to be addressed in next Amendment request.*

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date / Time

cc: J. E. Cross  
R. F. Rogers

SUBJECT: Technical Specification Table 3.7.4-2, page 3/4 7-24.

No. -021

DISCUSSION: Technical Specification Table 3.7.4-2 contains a listing of Safety Related Mechanical Snubbers. The purpose of this change is to add new snubber Q1E51G180R01 Area 8 Elevation 97.

JUSTIFICATION: The new snubber is added to Technical Specification Table 3.7.4-2 as a result of a design change to the Reactor Core Isolation Cooling System (RCIC). The design change installs a bypass line around the RCIC turbine steam shutoff valve to limit flow during automatic starts and thus prevent turbine overspeed trips. The new snubber is added to the new bypass line and provides support to meet system design requirements.

Technical Specification Table 3.7.4-2 allows the addition of snubbers to safety related systems without prior license amendment provided that a revision to Table 3.7.4-2 is included with the next license amendment request.

SIGNIFICANT HAZARDS CONSIDERATION:

The addition of the new snubber is an administrative change to the Technical Specifications to reflect changes to the design of the plant. This change does not involve the reduction of safety margins and no significant increase in the probability or consequences of an accident previously evaluated is involved nor is the possibility of a new or different kind of accident from any accident previously evaluated created. Thus the proposed change to the Technical Specification does not involve a significant hazards consideration.

TABLE 3.7.4-2 (Continued)  
SAFETY RELATED MECHANICAL SNUBBERS\*

<u>SNUBBER NO.</u>	<u>AREA</u>	<u>ELEVATION</u>	<u>SNUBBER NO.</u>	<u>AREA</u>	<u>ELEVATION</u>
<u>RCIC SYSTEM (Continued)</u>			<u>RWCU SYSTEM (Continued)</u>		
Q1E51G004R07	8	97	Q1G33G002R10	11	102
Q1E51G004R07	8	97	Q1G33G002R10	11	102
Q1E51G004R08	11	164	Q1G33G002R11	11	102
Q1E51G004R08	11	164	Q1G33G002R12	11	102
Q1E51G004R11	8	97	Q1G33G002R13	11	102
Q1E51G004R13	11	167	Q1G33G002R13	11	102
Q1E51G004R13	11	167	Q1G33G002R14	11	102
Q1E51G004R14	11	152	Q1G33G002R14	11	102
Q1E51G004R14	11	152	Q1G33G002R16	11	112
Q1E51G158R03	11	143	Q1G33G002R17	8	125
Q1E51G158R03	11	143	Q1G33G002R17	8	125
Q1E51G180R01	8	97	Q1G33G002R18	8	116
k. <u>COMBUSTIBLE GAS CONTROL SYSTEM</u>			Q1G33G002R19	8	116
Q1E61G001R07	11	189	Q1G33G002R21	11	102
			Q1G33G002R21	11	102
			Q1G33G002R22	11	102
l. <u>RWCU SYSTEM</u>			Q1G33G002R24	11	
Q1G33G002C03	11	113	Q1G33G011R01	11	140
Q1G33G002C03	11	113	Q1G33G011R03	11	145
Q1G33G002R03	8	136	Q1G33G011R03	11	145
Q1G33G002R03	8	136	Q1G33G012R01	11	142
Q1G33G002R05	11	140	Q1G33G012R01	11	142
Q1G33G002R05	11	140	Q1G33G012R02	11	152
Q1G33G002R08	11	102	Q1G33G105R01	11	103
Q1G33G002R08	11	102	Q1G33G105R01	11	103
Q1G33G002R09	11	102	Q1G33G105R01	11	103
Q1G33G002R09	11	102			
Q1G33G002R09	11	102			

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 033

Priority: 1A

\_\_\_\_\_  
Identified By / Date

\_\_\_\_\_  
Responsible Supervisor

Tech Spec Reference: Table 3.3.8-2

Problem Title: Containment Spray/Plant Actuation Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):  
Change containment spray timer setpoint and allowable value according to GE specs. Pending resolution of Containment Spray Timer issue.

2. Safety Significance: Plant Staff has administratively increased the surveillance frequency to insure the instrument stays within the tech spec allowable value.

3. Anticipated Resolution: Provide analysis of containment pressure for transient/accident which results in worst containment pressure. NPE has MNCR for action. Resolve timer requirements. *Programmatically increase surveillance frequency. For vendor agreed.*

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_  
Individual Notified / Date / Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date / Time

cc: J. E. Cross  
R. F. Rogers

Identified By \_\_\_\_\_ Date 1

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: ~~3.3.8~~ Table 3.3.8-2

Problem Title: Containment Spray / Plant Activation Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other): \_\_\_\_\_

Change Containment spray timer setpoint  
and allowable value according to GE specs.  
Pending resolution of Containment Spray Time  
issue.

2. Safety Significance: Plant staff has administratively increased

the surveillance frequency to insure the instrument  
stays within the tech spec allowable value.

3. Anticipated Resolution: Provide analysis of containment pressure

for transient/accident which results in worst containment  
pressure. NPE has MNCR for action. Resolve timer requirements

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date 1 Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

"TECH SPEC PRIORITY"

Punchlist Item # 033

Tech Spec Table 3.3.8-2

Priority 1A

TO: Manager of Nuclear Plant Engineering

FROM: Chairman, Prioritization and Disposit on Chairman

SUBJECT: Technical Specifications Punchlist Item # 033

PDTS:84/ 0003

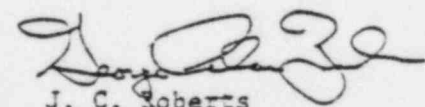
DATE: 3/10/84

The subject Tech Spec item has been determined by the Disposition Committee to require Engineering support.

DETAILS: Resolve time requirements.

Please contact Joe Hendry at Extension 2670<sup>5</sup> for further information.

Please refer to the Tech Spec Punchlist item number in your response. Forward your response to George Zinke.

  
J. C. Roberts  
for Chairman

- LLJ/JCR:swb  
cc: Mr. C. L. Tyrone  
Mr. J. E. Cross  
Mr. D. Stonestreet  
Mr. A. S. McCurdy  
Mr. S. Hutchins  
Mr. J. Hendry  
File (Tech Spec Records)



"TECH SPEC PRIORITY"

Punchlist Item # See ATTACHED

Tech Spec See ATTACHED

Priority See ATTACHED

TO: Manager of Nuclear Plant Engineering

FROM: Chairman, Prioritization and Disposition Chairman

SUBJECT: Technical Specifications Punchlist Item # see ATTACHED

PDTS: 84/ 0014

DATE: 3/10/84

The subject Tech Spec item has been determined by the Disposition Committee to require Engineering support.

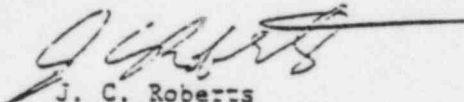
DETAILS: This letter identifies requested response dates for the following Tech Spec problems:

# 199 Letter No. PDTS 84/0001	# 015 Letter No. PDTS 84/0007
# 180 Letter NO. PDTS 84/0002	# 198 Letter NO. PDTS 84/0008
# 033 Letter NO. PDTS 84/0003	# 202 Letter NO. PDTS 84/0009
# 054 Letter NO. PDTS 84/0004	# 213 Letter NO. PDTS 84/0010
# 001 Letter NO. PDTS 84/0005	# 219 Letter NO. PDTS 84/0011
# 016 Letter NO. PDTS 84/0006	# 118 Letter NO. PDTS 84/0012

It is requested that the responses to the above items be completed by 3/13/84

Please contact Jerry Roberts at Extension 2695 for further information.

Please refer to the Tech Spec Punchlist item number in your response. Forward your response to George Zinke

  
J. C. Roberts  
Chairman

LLJ/JCR:swb

cc: Mr. C. L. Tyrone  
Mr. J. E. Cross  
Mr. D. Stonestreet  
Mr. A. S. McCurdy  
Mr. S. Hutchins  
Mr. J. Handry  
File (Tech Spec Records)

A4/61swb1



Tech Spec Problem No.

Tech Spec

Priority

199

Table 3.3.6-1.5

1B

180

4.8.4.3

1D

~~033~~

Table 3.3.8-2

1B

~~054~~

3/4.3.8

1B

~~001~~

3/4.5.1

1B

016

3/4.3.8

1B

015

3/4.3.2

1D

~~198~~

3/4.3.7

1B

202

3/4.3.7

~~1B~~

213

3/4.3.3

~~1B~~

219

Figure 3.4.6.1-1

1B

168

3.6.3.1

1B

"TECH SPEC PRIORITY"

MEMO TO: J. F. Pinto, Manager of Nuclear Plant Engineering

FROM: C. L. Tyrone, Project Manager

SUBJECT: Handling of Tech Spec Review Items

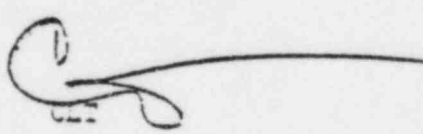
TSRO: 84/0001

DATE: March 11, 1984

This memorandum confirms our conversation of March 10, 1984. At that time, your assistance was requested in resolving discrepancies on eleven priority 1 items. Since then two items have been added. These items are all previously identified items which require early resolution with the NRC. A response is needed on these items by 12:30 PM on March 11, 1984. A list is attached.

Furthermore, all items of any priority identified (or previously known) which are being handled on this program require expeditious handling. This includes areas where requests are originated from other interfacing organizations such as the Plant or Nuclear Services. In any case where conflicts regarding highest priority is not clear, I am available to provide clarification.

It is suggested that you arrange 7 day a week support in this area as it is needed and arrange for all NPE personnel who will be involved in this effort to be available (or on call) in a manner that will support the Tech Spec Review program.



SHH:sad  
Attachment

cc: J. B. Richard (w/a)  
J. P. McGaughey (w/a)  
J. F. Pinto (w/a)  
J. E. Cross (w/a)  
T. H. Cloninger (w/a)  
H. J. Green (w/a)  
R. C. Fron (w/a)  
D. W. Stonestreet (w/a)

~~\_\_\_\_\_~~  
T. E. Reaves, Jr. (w/a)  
S. M. Feith (w/a)  
J. G. Cesare (w/a)  
G. W. Smith (w/a)  
L. R. McKay (w/a)  
L. C. Burgess (w/a)  
File (Tech Spec Records) (w/a)

LIST OF CURRENT PRIORITY 1  
ITEMS REQUIRING NPE SUPPORT

PDTS:84/	P/L #	Date Sent
001	199	3/10/84
002	180	3/10/84
003	033	3/10/84
004	054	3/10/84
005	001	3/10/84
006	016	3/10/84
007	015	3/10/84
008	198	3/10/84
009	202	3/10/84
010	213	3/10/84
011	219	3/10/84
012	083	3/10/84
013	168	3/10/84

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 034

Priority: 1/A

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3

Problem Title: Generic Instrumentation Problems

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

NRR I&C Branch mandated Tech Spec review of Tech Spec Instrumentation. Review includes submitting definitions of channels, trip systems, trip functions. Also included is review of action statements, minimum operable channels, LCO's and single failure criteria.

2. Safety Significance: Tech Spec difficult to understand and implement. Could lead to non-conservative system operations.

3. Anticipated Resolution: Review of Instrumentation Tech Spec with Tech Spec change pending. *GOES AWAY WHEN REVIEW IS SUBMITTED (15 MARCH) SPECIFIC PROBLEMS WILL BE TRACKED SEPARATELY.*

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date / \_\_\_\_\_ Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Identified By \_\_\_\_\_ Date 1

Responsible Supervisor \_\_\_\_\_ 1A

Tech Spec Reference: 3/4.3

Problem Title: Generic Instrumentation Problems

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other): \_\_\_\_\_

NRC T&C Branch mandated Tech Spec review of Tech Spec Instrumentation  
Review includes sketching definitions of channels, trip systems trip functions,  
plus included is review of action statements, minimum acceptable channels,  
LCD's and single failure criteria.

2. Safety Significance: Tech Spec difficult to understand and implement. Could

lead to non-consecutive system operations.

3. Anticipated Resolution: Review of Instrumentation Tech Spec with

Tech Spec change package.

4. NRC Response to Item (NRR/IE): Mandated review of Instrumentation Tech Spec.

NRC Notified: \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date 1 Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ 1  
Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

File (LCTS)



MISSISSIPPI POWER & LIGHT COMPANY

Helping Build Mississippi

P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

October 14, 1983

NUCLEAR PRODUCTION DEPARTMENT

34

U. S. Nuclear Regulatory Commission  
Office of Nuclear Reactor Regulation  
Washington, D. C. 20555

Attention: Mr. Harold R. Denton, Director

Dear Mr. Denton:

SUBJECT: Grand Gulf Nuclear Station  
Unit 1  
Docket No. 50-416  
License No. NPF-13  
File 0260/L-860.0  
Request for Additional  
Information, Technical  
Specifications  
AECM-83/0640

REFERENCE: AECM-83/0519, letter to NRC, dated September 12, 1983, concerning  
Technical Specification Terminology, ICSB Concerns.

This letter addresses the Nuclear Regulatory Commission (NRC) request for additional information sent to Mississippi Power & Light Company (MP&L) on September 12, 1983. Responses to the requests for additional information are attached. As a result of the reviews conducted to provide this information to the NRC, MP&L has identified a need to modify plant procedures, the Technical Specifications and the Final Safety Analysis Report (FSAR). The changes are identified in the attached pages. MP&L proposes to make these changes on the following schedule:

- a. Revise plant procedures for temporary jumper concerns by November 30, 1983.
- b. Submit changes to the Technical Specifications to address trip unit calibration frequency by December 15, 1983.
- c. Include corrections to Table 7.3-10 of the FSAR in the next annual update in June 1984.

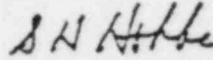
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~~8310190037~~



## MISSISSIPPI POWER &amp; LIGHT COMPANY

Please contact this office if additional information is desired.

Yours truly,



for L. F. Dale  
Manager of Nuclear Services

WJH/JOF/SHH:sap  
Attachment

cc: Mr. J. B. Richard (w/o)  
Mr. R. B. McGehee (w/o)  
Mr. T. B. Conner (w/c)  
Mr. G. B. Taylor (w/o)

Mr. Richard C. DeYoung, Director (w/a)  
Office of Inspection & Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

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File (LCTS) (w/2)  
File (Plant) (w/a)  
File (Project) (w/a) [7]

RESPONSE TO SEPTEMBER 12, 1983 REQUEST  
FOR ADDITIONAL INFORMATION

1. CONCERN:

Table 3.3.2-1 of the Grand Gulf Technical Specifications delineates the minimum number of operable channel requirements for the isolation actuation instrumentation. From a comparison of this table to Table 7.3-10 of the Grand Gulf FSAR the NRC staff has found that in most cases the technical specifications include only one half of the total number of channels provided. Confirm that the single failure criterion can be satisfied for each case where the minimum number of operable channels requirement is less than that of the total number of channels provided for each reactor protection system trip function.

RESPONSE:

The number of sensor channels shown in FSAR Table 7.3-10 is actually the total number installed in the plant. FSAR Table 7.3-10 states incorrectly that the number of sensor channels provided is the number per trip system. This point will be corrected in the annual FSAR update scheduled for submittal in June 1984. For example, for two trip systems the number of channels listed in the FSAR would typically be divided in half and then this number used as the number of channels per trip system in the Technical Specifications.

MP&L has conducted a review of Technical Specification Tables 3.3.1-1, 3.3.2-1, and 3.3.3-1 for the Reactor Protection System, Isolation Actuation and Emergency Core Cooling Actuation Instrumentation. This review was conducted to determine where the minimum number of operable channels requirement is less than the total number of instruments in the trip system and to verify that single failure criterion is satisfied in these cases. The only instruments found to be in this category are the Intermediate Range Monitors (IRM's) and the Average Power Range Monitors (APRM's) listed as items 1 and 2 in Technical Specification Table 3.3.1-1. For these instruments the minimum operable channels per trip system is listed as 3 in Technical Specification Table 3.3.1-1, but the number of channels per trip system installed in the plant is 4. Any one of the 4 channels per trip system installed in the plant for IRM's or APRM's will actuate its associated trip system and initiate a half scram. Therefore, to allow for a single failure, any 2 of the 4 channels provided per trip system (for a total of 4 of the 8 for both trip systems) is all that is required. The 3 (or 2 in Operational Conditions 3 or 4 for IRM's) channels listed in Technical Specification Table 3.3.1-1 are adequate to meet single failure criterion.

2. CONCERN:

Certain technical specification action statements for the isolation actuation instrumentation permit continued operation with inoperable components provided an inoperable instrument channel is placed in the tripped condition. From discussions with MP&L representatives the NRC staff believes that in some cases placing a channel in the tripped condition includes lifting leads and the use of temporary jumpers. Confirm that for each case where placing a reactor protection system

channel in the tripped condition requires the lifting of leads or the use of temporary jumpers the licensing criteria is satisfied. For example, confirm that the cable separation criteria is satisfied with the jumper installed and confirm that following a seismic event the system remains operable with the jumper installed. In addition, confirm that for each case where a reactor protection system technical specification action statement permits continued operation without requiring that the instrument channel be tripped that the single failure criterion is met or the protective action is initiated.

RESPONSE:

The responses to these concerns are provided as follows:

- a. MP&L does not normally use lifted leads or temporary jumpers to place reactor protection system instrumentation channels in the tripped condition. The referenced letter specifies four methods that are preferred for placing a channel in the tripped condition. However, lifted leads or temporary jumpers may be used to place a channel in the tripped condition by following the Grand Gulf plant procedure for temporary alterations. Temporary alterations used to place reactor protection system instrumentation channels in the tripped condition will be accomplished using an engineering work package (Maintenance Work Order) which will assure that seismic, environmental and separation criteria are considered. Plant procedures will be reviewed and revised as necessary by November 30, 1983 to accomplish this requirement.
- b. MP&L has reviewed Technical Specification Tables 3.3.1-1, 3.3.2-1 and 3.3.3-1 for the reactor protection system instrumentation to confirm that for each case where a technical specification action statement permits continued operation without requiring that the instrument channel be tripped that the single failure criterion is met or the protective action is initiated. The results of this review are as follows:
  - 1) ACTION 6 on Table 3.3.1-1 applies to the Turbine Stop Valve-Closure and Turbine Control Valve Fast Closure. Although ACTION 6 does not require the instrument channel to be tripped, it does require a reduction of thermal power within 15 minutes and reduction of turbine first stage pressure to less than the automatic bypass setpoint within 2 hours. Reactor protection is provided in this case since the turbine first stage pressure is reduced to less than the automatic bypass setpoint thus removing the necessity for the turbine stop valve-closure and turbine control valve fast closure trips.
  - 2) ACTION 32 on Table 3.3.3-1 applies to the manual initiation functions for the Emergency Core Cooling Systems Actuation Instrumentation. ACTION 32 does not require the associated systems to be declared inoperable until 8 hours has elapsed. This 8 hour time frame is not considered excessive since the automatic initiation functions for the systems are still operable and will function as designed.



GRAND GULF NUCLEAR STATION UNIT 1  
MISSISSIPPI POWER & LIGHT COMPANY  
BECHTEL CONTINUING SERVICES JOB NO. 15026

DESIGN CHANGE ASSISTANCE  
MP&L NUCLEAR PLANT ENGINEERING  
(CONTINUATION SHEET)

DCP NO. \_\_\_\_\_ } Request No. 1  
PROJECT NO. NPE-3-205  
SHEET 1 OF \_\_\_\_\_

Bechtel Response #1

Attached are the tables requested by this DCA. These tables include the parameters measured, device numbers of trip units for each parameter, total number of channels provided, number of channels provided per trip function or trip system, minimum operable channels per trip system (per Tech Specs tables), and the method to accomplish the actions required by the Tech Specs. If the minimum number of operable channels per trip system is less than the number of channels provided per trip system, then a justification or recommendation for Tech Spec change has been provided. The last column also provides recommendations on what actions should be taken when the number of operable channels is less than the minimum required and when the Tech Specs provide options. The information provided was based on Tech Spec tables 3.3.1-1 (RPS), 3.3.2-1 (Isolation) and 3.3.3-1 (ECCS) through license amendment ~~11~~ 11.

GPO-3387-C1 1/83

DATE: 11/20/83 BY: TLT  
NPE-205(1) BECHTEL RESP. 1





GRAND GULF NUCLEAR STATION UNIT 1  
 MISSISSIPPI POWER & LIGHT COMPANY  
 BECHTEL CONTINUING SERVICES JOB NO. 15026

DESIGN CHANGE ASSISTANCE  
 MP&L NUCLEAR PLANT ENGINEERING  
 (CONTINUATION SHEET)

DCP NO. \_\_\_\_\_  
 PROJECT NO. NPE-3-205 } Request No. 1  
 SHEET 2 OF \_\_\_\_\_

In regards to item 3 of the NRC's request for additional information, attached to their letter of 9-12-83, it appears that the NRC is concerned that the setpoints will be verified only once every 18 months. However in order to address this concern, the NRC has asked a rather open-ended question on setpoint methodology. It is recommended that MP&L address the NRC's concern directly - i.e. setpoints are verified much more frequently than every 18 months and on an interval consistent with other GE BWR's. On the overall issue of setpoint methodology, the response to the NRC should point out that MP&L is participating with other BWR owners and GE to resolve this issue. MP&L should submit a setpoint methodology position to the NRC on a schedule consistent with the generic resolution, and this issue should in no way impact full power licensing.

Attached is a copy of Tech Spec tables 4.3.1.1-1 (RPS), 4.3.2.1-1 (Isolation) and 4.3.3.1-3 (ECCS) showing the channel calibration frequencies for the BWR-5 Standard Tech Specs (NUREG 0123) and annotated with clarifying notes and recommendations for changes.

15/11  
 NPE-3-20501



GRAND GULF NUCLEAR STATION UNIT 1  
 MISSISSIPPI POWER & LIGHT COMPANY  
 BECHTEL CONTINUING SERVICES JOB NO. 15026

DESIGN CHANGE ASSISTANCE  
 MP&L NUCLEAR PLANT ENGINEERING  
 (CONTINUATION SHEET)

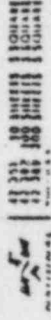
DCP NO. \_\_\_\_\_  
 PROJECT NO. NYE-3-205 Request No. 1

SHEET 3 OF \_\_\_\_\_

List of Attached Documents

1. Special table based on Tech Spec table 3.3.1-1 (4 pages)
2. Special table based on Tech Spec table 3.3.2-1 (6 pages)
3. Special table based on Tech Spec table 3.3.3-1 (5 pages)
4. Tech Spec table 4.3.1.1-1 (Mark-up) (2 pages)
5. Tech Spec table 4.3.2.1-1 (Mark-up) (5 pages)
6. Tech Spec table 4.3.3.1-1 (Mark-up) (2 pages)
7. Tech Spec Calibration Frequencies. Notes (1 page)

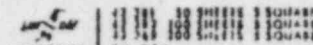
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NYE-3-2050



TECH. SPEC. TABLE 3.3.1-1

PARAMETER	TRIP UNITS	TOTAL NO CHANNELS	NO CHANNELS PER TRIP SYSTEM	NO OPER. CHANNELS IN TRIP SYSTEM	PROCEDURE TO ACCOMPLISH ACTION
1. I.P.M:					
1a. Neutron Flux - High	ICS1-NY - K601A-D (K14) ICS1-NY - K601E-H (K24)	8	4	3-Note 1	Note 5
1b. Inoperative	ICS1-NY - K601A-D (K11) ICS1-NY - K601E-H (K21)	8	4	3-Note 1	Note 7
2. A.P.R.T.:					
2a. Neutron Flux - High, Sella	ICS1-2405 (K4)	8	4	3-Note 1	Note 6
2b. Flow Biased Simulated Thermal Flux - High	ICS1-2409 (K1)	8	4	3-Note 1	Note 6
2c. Neutron Flux - High	ICS1-2401 (K1)	8	4	3-Note 1	Note 6
2d. Temperature	ICS1-2401 (K2)	8	4	3-Note 1	Note 7
3. REACTOR VESSEL STEAM DOME PRESSURE - High	1B21-P15-N678A,B,C,D	4	2	2	Note 8
4. REACTOR VESSEL WATER LEVEL - Low, LEVEL 3	1B21-L15-N680A,B,C,D	4	2	2	Note 8
5. REACTOR VESSEL WATER LEVEL - High, LEVEL 2	1B21-L5-N683A,B,C,D	4	2	2	Note 8
6. MAIN STEAM LINE ISOLATION VALVE CLOSURE	1B21-25-N101A,B,C,D 1B21-25-N102A,B,C,D	8 8	4 4	4-Note 2 4-Note 2	Place appropriate switch IC71-115-H605A,B,C,D or H606A,B,C,D in test or test 2 position as appropriate.
7. MAIN STEAM LINE ISOLATION - High	1B17-R15-K610A,B,C,D	4	2	2	Note 9





PARAMETER	TRIP UNITS	TOTAL # CHANNELS	# CHANNELS PER TRIP SYSTEM	MIN. OPER CHANNELS PER TRIP SYSTEM	PROCEDURE TO ACCOMPLISH ACTION
8. <u>TRIPWELL PRESSURE - HIGH</u>	1C71-PIS-NGSDA,B,C,D	4	2	2	Note 8
9. <u>SCRAM DISCHARGE VOLUME WATER LEVEL - HIGH</u>	1C11-LIS-NG01A,B,C,D	4 - Note 3	2 - Note 3	2 - Note 3	Note 8
10. <u>TURBINE STOP VALVE - CLOSURE</u>	1C71-PIS-NG06A,B,C,D,E,F,G,H	8	4	4	Note 8
11. <u>TURBINE CONTROL VALVE FAST CLOSURE, VALVE TRIP SYSTEM OIL PRESSURE - Loc</u>	1C71-PIS-NG05A,B,C,D	4	2	2	Note 8
12. <u>REACTOR MODE SWITCH - Sitelbound Position</u>	1C71-HSS-M1602	1	1 - Note 4	1 - Note 4	Note 4
13. <u>MANUAL SCRAM</u>	1C71-HS-M1600A,B,C,D	4	2	2	Note 10

1. In the IRM and APRM logic for RPS, any 1 of the 4 channels provided per trip system will activate that trip system and initiate a 1/2 screen. Therefore, to allow for a single failure, any 2 of the 4 channels provided per trip system (for a total of 4 of the 8 for both trip systems) is all that is required. The 3 (or 2 in Modes 3 & 4) channels listed in Tech Spec Table 3.3.1-1 are adequate to meet single failure criterion.
2. There are 8 channels per trip function for the inboard MSIV's as well as 8 channels per trip function for the outboard MSIV's. Actuation of either an inboard MSIV trip system or an outboard MSIV trip system will cause the trip function to occur. Therefore, the minimum number of operable channels per trip system should be addressed for both the inboard values and the outboard values. In both cases, the minimum number of channels required per trip system is 4.
3. At present, only IC11-LIS-NO1A, B, C, D are installed. DCP-31/5007 has been issued to add redundant 4 diverse Level switches (IC11-LIS-NO13A-D). When this DCP is implemented, the total # of channels will become 8, the # channels per trip system will become 4, and the minimum operable channels required to meet the single failure criterion would be 2. (Only one channel is needed to trip the trip system + on

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(DC)

ATTACH 1, 3 OF 4

## NOTES

additional channel to account for a single failure = 2.) However should the NRC require the single failure criterion to be met for each diverse <sup>level sensing</sup> method independently, then the minimum number of operable channels will be 4.

4. There is only one reactor mode switch which operates relays in both trip systems. The mode switch and all its associated relays should be verified operable in the shutdown position during periodic channel checks. Otherwise, the mode switch should be declared inoperable, and the appropriate action (1, 3 or 7) carried out.
5. Place the SI-selector switch for the MSV-WRM for the affected IRM channel in the "STANDBY" position. Do not bypass the IRM channel to be placed in the tripped condition.
6. Place the SI selector switch for the affected APRM channel in a position other than "operate". Do not bypass the APRM channel to be placed in the tripped condition.
7. If the number of operable neutron monitoring inoperative channels is less than the minimum operable channels per trip system requirement, then the trip system should be placed in the tripped condition.
8. Rosemount trip units (which trip on open circuit) can be placed in the tripped condition by using the calibration station. This can only be done for one trip unit within a card file, which then limits the ability to calibrate or functionally test other units in the same card file. An alternate acceptable method is removing the trip unit.
9. Place the SI selector switch for the affected radiation in a position other than "OPERATE."
10. If a manual scram switch is inoperative, then the trip system should be placed in a tripped condition using the other manual scram switch in the trip system.

TECH SPEC TABLE 3.3.2-1  
Isolation Actuation Instrumentation

Parameter	Trip Units	Total # of channels	# of channels per trip system	minimum operable channels per trip system	Procedure to accomplish action
<b>1. Primary Containment Isolation</b>					
a. RPV Level-L2	1B21-LS-N682A,B,C,D	4	2	2	Note 1
b. RPV Level-L2 (ECCS)	1B21-LIS-N673C,G,L,R	4	4	4	Note 2
c. RPV Level-L1 (ECCS)	1B21-LIS-N691A,B,E,F	4	2	2	Note 3
d. Drywell Pressure	1C71-PIS-N650A,B,C,D	4	2	2	Note 1
e. Drywell Pressure(ECCS-1,2)	1B21-PIS-N694A,B,E,F	4	2	2	Note 3
f. Drywell Pressure(ECCS-3)	1B21-PIS-N667C,G,L,R	4	4	4	Note 2
g. Cont & dowl vent exhaust radiation	1D17-RITS-K609A,B,C,D	4	2	2(e)-Note 4	Note 1
h. Manual initiation	1B21-IIS-M630A,B,C,D	4	2	2	Note 5
<b>2. Main Steam Line Isolation</b>					
a. RPV Level-L1	1B21-LIS-N681A,B,C,D	4	2	2	Note 1
b. MSL radiation	1D17-RITS-K610A,B,C,D	4	2	2	Note 1
c. MSL pressure	1B21-PIS-N676A,B,C,D	4	2	2	Note 1
d. MSL flow	1E31-FIS-N686A,B,C,D	4/line	2/line	2(g)- Note 6	Note 1
	1E31-FIS-N687A,B,C,D				
	1E31-FIS-N688A,B,C,D				
	1E31-FIS-N689A,B,C,D				
e. Condenser vacuum	1B21-PIS-N675A,B,C,D	4	2	2	Note 1
f. MSL tunnel temp	1E31-TS-N604A,B,C,D	4	2	2	Note 1
g. MSL tunnel atemp	1E31-TDS-N605A,B,C,D	4	2	2	Note 1
h. Manual initiation	— Same as 1d —	4	2	2	Note 5



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TECH SPEC TITLE 33  
Isolation Actuation Instrumentation

ATTACH 2 2 OF 6

Parameter	Trip Units	Total # of channels	# of channels per trip system	minimum operable channels per trip system	Procedure to accomplish action
<b>3. Secondary Containment Isolation</b>					
a. RPV Level - L2	— Same as 1a —	4	2	2	Note 1
b. Drywell pressure	— Same as 1b —	4	2	2	Note 1
c. FIA vent exp radiation	ID17-RITS-KG17A,B,C,D	4	2	2	Note 7
d. FIA pool sweep exh radiation	ID17-RITS-KG18A,B,C,D	4	2	2	Note 7
e. Manual initiation	— Same as 1d —	4	2	2	Note 5
<b>4. Reactor Water Cleanup System Isolation</b>					
a. RWCU flow	IE31-FS-N609A,B	2	1	1	Note 7
b. RWCU flow timer	IE31-KIS-R615A,B	2	1	1	Note 7
c. RWCU area temp	IE31-TS-N620A,B	2/room	1/room	1/room	Note 7
	IE31-TS-N621A,B				
	IE31-TS-N622A,B				
	IE31-TS-N623A,B				
	IE31-TS-N624A,B				
	IE31-TS-N625A,B				
	IE31-TS-N626A,B				
	IE31-TS-N627A,B				
d. RWCU area & Temp	IE31-TDS-N612A,B	2/room	1/room	1/room	Note 7
	IE31-TDS-N613A,B				
	IE31-TDS-N614A,B				
	IE31-TDS-N615A,B				
	IE31-TDS-N616A,B				
	IE31-TDS-N617A,B				
	IE31-TDS-N618A,B				
	IE31-TDS-N619A,B				

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TECH SPEC TABLE 3.3.2-1

ATTACH. 2, 3 OF 6

Parameter	Trip Units	Instrumentation			Procedure to accomplish action
		Isolation	Actuation	Instrumentation	
		Total # of channels	# of channels per trip system	minimum operable channels per trip system	
<b>4. Reactor Water Cleanup System Isolation (Continued)</b>					
e. RPV Level-L2	Same as 1a	4	2	2	Note 1
f. MSL tunnel temp	IE31-TS-N604A,B	2	1	1	Note 1
g. MSL tunnel Δtemp	IE31-TDS-N605A,B	2	1	1	Note 1
h. SICS initiation	CA11-HS-M601A,B	2	1	NA	Note 7
i. Manual initiation	Same as 1d	4	2	2	Note 5
<b>5. Reactor Core Isolation Cooling System Isolation</b>					
a. RCIC atm line flow	IE31-PDIS-N683A,B	2	1	1	Note 7
b. RCIC atm supply pressure	IE31-PIS-N685A,B	2	1	1	Note 7
c. RCIC turb exh diaph press.	IE51-PIS-N655A,B,E,F	4	2	2	Note 7
d. RCIC room temp	IE31-TS-N602A,B	2	1	1	Note 7
e. RCIC room Δtemp	IE31-TDS-N603A,B	2	1	1	Note 7
f. MSL tunnel temp	IE31-TS-N604C,F	2	1	1	Note 1
g. MSL tunnel Δtemp	IE31-TDS-N605E,F	2	1	1	Note 1
h. MSL tunnel temp timer	IE31-KIS-R617E,F	2	1	1	Note 7
i. RHR room temp	IE31-TS-N608A,B	2/room	1/room	1/room	Note 7
	IE31-TS-N610A,B				
j. RHR room Δtemp	IE31-TDS-N600A,B	2/room	1/room	1/room	Note 7
	IE31-TDS-N611A,B				
k. RHR/RCIC atm line flow	IE31-PDIS-N684A,B	2	1	1	Note 7
l. Manual Initiation	IE51-HS-N627	1	1-Div 1 only	1 - Note 8	Note 5
m. Drywell Pressure (ECCS)	IB21-PIS-N694A,B	2	1	1	Note 3

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TECH SPEC TABLE 3.3.2-1

Isolation Activation	Minimum Isolation	# of channels per trip system	minimum operable channels per trip system
Trip Units	Total # of channels		
Same as 5j	2/room	1/room	1/room
Same as 5j	2/room	1/room	1/room
IB21-L15-N680A, B, C, D	4	2	2
IB21-P15-N679A, B, C, D	4	2	2
Same as 1b	4	2	2
Same as 1d	4	2	2

Parameter

6. RIR System Isolation

- a. RIR room temp
- b. RIR room 4 temp
- c. RIR Level - L3
- d. RIR Pressure high
- e. RIR Pressure
- f. Manual initiation

Procedure to accomplish action

Note 7  
 Note 7  
 Note 7  
 Note 7  
 Note 7  
 Note 5



## Tech Spec Table 3.3.1-1 Notes

1. The trip system can be placed in the tripped condition by operating the isolation switch (1B21-HS-M630A, B, C, or D) associated with the inoperable channel, for one-out-of-two twice logic (MSIV's). For 2 out of 2 logic, use action statement on page 3/4 3-14.

2. Since there is only one trip system for Division 3 ECCS, Action 29 should be used.

3. Rather than placing the trip system in the tripped condition (i.e. initiating ECCS), Action 27 or 29 should be used.

4. Footnote (e) states that channel trips on hi-hi or downscale. Trip units actually trip on hi-hi or inoperative; downscale only causes alarm.

5. If the manual initiation function is inoperable, then it will not be possible to place the trip system in the tripped condition using the inoperable switch. The trip system should be tripped using the other manual initiation switch, or Actions 22, 25, or 26 should be used as appropriate.

6. An instrument channel should be defined as one measurement channel only, from sensor through to trip unit (or trip relay). Note (a) appears confusing as written and should be removed from the Tech Specs. Also, steam flow in each of the four lines is an independent parameter, and each steam line should have enough instrument channels available to meet the single failure criterion. The correct value for this column is: 2/line.

7. Action 25, 27, or 28 should be used rather than placing the entire trip system in the tripped condition.

ATTACH. 2, 6 of 6Tech Spec Table 3.3.2-1 Notes

8. There is an isolation switch only for division 1. The correct value for this column is "1-Trip System A; 0-Trip System B."

9. The action statements in Table 3.3.2-1 are overly restrictive.

In Tech Spec sections 3.3.1 and 3.3.3, action statements generally allow the trip channel to be placed in the tripped condition, rather than the entire trip system. (For one-out-of-two-twice logic, placing the trip channel in the tripped condition does trip the trip system.). It would be beneficial to allow tripping only the inoperable channel, rather than the trip system, since not all channels (parameters) close all isolation valves.

Emergency Core Cooling System Activation Instrumentation

Parameter	Trip Unit	Total # of channels	# of Channels per trip function	Min. Redundant Channels per Trip function	Procedure To Accomplish Action
1. Division 1 Trip System					
1. RHR-A (LPCI Mode) & LPCS System					
a. RPV Level - LI	1B21-LIS-N691A,E	2	2	2 (b)	NOTE 1
b. DH Pressure - Risk	1B21-PI5-N694A,E	2	2	2 (b)	NOTE 1
c. LPCI Pump A Start Time Delay Relay	1E12-R70A - NOTE 2	1	1	1	
d. Manual Initiation	1E21-UIS-N613	1	1	1/system	
2. ADS Trip System "A"					
a. RPV Level - LI	1B21-LIS-N691A,E	2	2	2 (b)	NOTE 1
b. DH Pressure - Risk	1B21-PI5-N694A,E	2	2	2 (b)	NOTE 1
c. ADS Timer	1B31-K5A	1	1	1	
d. RPV Level - LI	1B41-LIS-N695A	1	1	1	
e. LPCS Pump Disch. Pressure - Risk	1E21-PI5-N662 & N653	2	2	2 - NOTE 3	NOTE 1
f. LPCI Pump A Disch. Pressure - Risk	1E12-PI5-N655A & N656A	2	2	2 - NOTE 3	NOTE 1
g. Manual Initiation	1B21-UIS-N629A,E	2	2	1/2/1/2 - NOTE 4	NOTE 1

TABLE 3.3.3-1

Emergency Core Cooling System Actuation Instrumentation

Requirement	Trip Unit	Total # of channels	# of channels per trip function	Min. Operable Channels per trip function	Procedure To Accomplish Action
B. Division 2 Trip System					
1. RHR BIC (LPCI Mode)					
a. RPV Level - LI	IB21-LIS-N691 B,F	2	2	2 (W)	NOTE 1
b. DW Pressure - High	IB21-PI5-N694 B,F	2	2	2 (W)	NOTE 1
c. LPCI Pump B Start Time Delay Relay	IE12-K70 B - NOTE 2	1	1	1	
d. Manual Initiation	IE12-HE-M617	1	1	1/system	
2. ADS Trip System - B <sup>0</sup>					
a. RPV Level - LI	IB21-LIS-N691 B,F	2	2	2 (W)	NOTE 1
b. DW Pressure - High	IB21-PI5-N694 B,F	2	2	2 (W)	NOTE 1
c. ADS Timmer	IB21-K5 B	1	1	1	NOTE 1
d. RPV Level - LI	IB21-LIS-N695 B	1	1	1	NOTE 1
e. LPCI Pump B & C Inlet Pressure - High	IE12-PI5-N655 B, C & N656 B, C	4	4	2/pump - NOTE 3	NOTE 1
f. Manual Initiation	IB21-MS-M629 B, F	2	2	1/relv2 - NOTE 4	



TABLE 3-2.2-1

Emergency Core Cooling System Activation Instrumentation

Parameter	Trip Unit	Total # of channels	# of channels per trip function	Min. Operable channels per trip function	Procedure To Accomplish Action
C. Division 3 Trip System					
1. HPCS System					
a. RPV Level - L2	IB21-LIS-N673 C, G, L, R	4	4	4 (W)	NOTE 1
b. DRI Pressure - High	IB21-PIS-N667 C, G, L, R	4	4	4 (W)	NOTE 1
c. RPV Level - L8	IB21-LIS-N674 C, L	2	2	2 (C)	NOTE 1
d. Cond. Storage Tank Level - Low	IE22-LIS-N654 C, G	2	2	2 (d)	NOTE 1
e. SUPP. Pool Motor Level - High	IE22-LIS-N655 C, G	2	2	2 (d)	NOTE 1
f. Manual Initiation	IE22-HIS-M616	1	1	VSystem	
D. Loss of Power					
1. Divisions 1 and 2					
a. 416 KV Bus Undervoltage (Loss of Voltage)	Bistables (4) - PT1, 2	4	4	4	NOTE 5
b. 416 KV Bus Undervoltage (BOP Load Shed)	Bistables (4) - PT1, 2	4	4	4	NOTE 5
c. 416 KV Bus Undervoltage (Beyond Voltage)	Bistables (4) - PT1, 2	4	4	4	NOTE 5
2. Division 3					
a. 416 KV Bus Undervoltage (Loss of Voltage)	27 S1, S2, S3 and S4	4	4	4	NOTE 5
Note: Division 1, 15 AA Bus Located on 1122-P331					
Division 2, 16 A Bus Located on 1122-P332					

TECH SPEC TABLE 3.3.3-1, NOTES

1. Rosemount trip units (which do not trip on open circuit) can be placed in the tripped condition by using the calibration station. This can only be done for one trip unit within a card file, which then limits the ability to calibrate or functionally test other units in the same card file. An alternate acceptable method is by inserting a trip unit which has been modified to be in the trip condition at all times.
2. THE LPCI TIME DELAY FUNCTION IS ALSO PROVIDED BY TIMING CIRCUITS IN THE LOAD SHEDDING AND SEQUENCING PANELS.
3. ACTION 31 IN THIS CASE IS TOO RESTRICTIVE; ACTION 30 SHOULD BE USED, WHICH WILL ALLOW ONE OF THE CHANNELS TO BE PLACED IN TRIPPED CONDITION INSTEAD OF TRIPPING THE WHOLE SYSTEM. Disagree
4. THE MANUAL INITIATION FUNCTION IN THE INSTRUMENT TECH SPECS NORMALLY APPLIES TO SYSTEM LEVEL MANUAL INITIATION AND NOT THE INDIVIDUAL COMPONENT CONTROL SWITCHES. THERE ARE TWO SYSTEM LEVEL MANUAL INITIATION SWITCHES PER ADS SYSTEM, BOTH OF WHICH MUST BE OPERATED TO CAUSE THE TRIP FUNCTION TO OCCUR. THEREFORE THE CORRECT VALUE FOR THIS COLUMN IS 2.
5. ACTION 30 OF TECHNICAL SPECIFICATION SHOULD BE MODIFIED FOR THE LOSS OF POWER FUNCTION. IF AN UNDERVOLTAGE CHANNEL IS INOPERABLE, IT MAY NOT BE ABLE TO BE PLACED IN



NOTES: CONTD.

THE TRIPPED CONDITION PER ACTION 30.  
HOWEVER THE EQUIVALENT ACTION (I.E. CHANGE  
THE LOGIC FROM ONE-OUT-OF-TWO-TWO TO ONE-OUT-  
OF-TWO) CAN BE ACCOMPLISHED BY REMOVING THE  
POTENTIAL TRANSFORMER FUSES FOR THE UNDERVOLTAGE  
CHANNEL IN THE SAME ONE-OUT-TWO LOGIC  
AS THE INOPERABLE CHANNEL. THUS, THE STILL OPERABLE  
CHANNEL WILL SENSE NO VOLTAGE AND WILL TRIP.

Wrong

TABLE 4.3.1.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION (a)	OPERATIONAL CONDITIONS FOR WATCH SURVEILLANCE REQUIRED	NOTES
1. Intermediate Range Monitors:					
a. Neutron Flux - High	S/U,S, S	S/U(c), W W	R R	2 3, 4, 5	R R
b. Inoperative	NA	W	NA	2, 3, 4, 5	NA
2. Average Power Range Monitor: (f)					
a. Neutron Flux - High, Setdown	S/U,S, S	S/U(c), W W	SA SA	2 3, 5	SA SA
b. Flow Biased Simulated Thermal Power - High	S, D(h)	S/U(c), W	W(d)(e), SA, R(1)	1	w
c. Neutron Flux - High	S	S/U(c), W	W(d), SA	1	w
d. Inoperative	NA	W	NA	1, 2, 3, 5	NA
3. Reactor Vessel Steam Dome Pressure - High	S	N	R(g)	1, 2	R 1
4. Reactor Vessel Water Level - Low, Level 3	S	N	R(g)	1, 2	R 1
5. Reactor Vessel Water Level - High, Level B	S	H	R(g)	1	
6. Main Steam Line Isolation Valve - Closure	NA	H	R	1	R
7. Main Steam Line Radiation - High	S	H	R	1, 2	R
8. Drywell Pressure - High	S	H	R(g)	1, 2	R

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NPE-205(h)(B21)

TABLE 4.3.1.1-1 (Continued)

ATTACH. 4, PAGE 2 OF 2

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR MITCH SURVEILLANCE REQUIRED	NOTES
9. Scram Discharge Volume Water Level High	S	M	R(g)	1, 2, 5	1 R
10. Turbine Stop Valve - Closure	S	M	R(g)	1	1 R
11. Turbine Control Valve Fast Closure Valve Trip System Oil Pressure - Low	S	M	R(g)	1	1 Q
12. Reactor Mode Switch Shutdown Position	NA	R	NA	1, 2, 3, 4, 5	NA
13. Manual Scram	NA	M	NA	1, 2, 3, 4, 5	NA

- (a) Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (b) The IRM and SRM channels shall be determined to overlap for at least 1/2 decade during each startup after entering OPERATIONAL CONDITION 2 and the IRM and APRM channels shall be determined to overlap for at least 1/2 decade during each controlled shutdown, if not performed within the previous 7 days.
- (c) Within 24 hours prior to startup, if not performed within the previous 7 days.
- (d) This calibration shall consist of the adjustment of the APRM channel to conform to the power values calculated by a heat balance during OPERATIONAL CONDITION 1 when THERMAL POWER > 25% of RATED THERMAL POWER. Adjust the APRM channel if the absolute difference is greater than 2% of RATED THERMAL POWER. Any APRM channel gain adjustment made in compliance with Specification 3.2.2 shall not be included in determining the absolute difference.
- (e) This calibration shall consist of the adjustment of the APRM flow biased channel to conform to a calibrated flow signal.
- (f) The LPRMs shall be calibrated at least once per 1000 effective full power hours (EFPH) using the TIP system.
- (g) Calibrate trip unit at least once per 31 days.
- (h) Verify measured drive flow to be less than or equal to established drive flow at the existing flow control valve position.
- (i) This calibration shall consist of verifying the  $6 \pm 1$  second simulated thermal power time constant.

GRAND GULF UNIT 1

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Amendment No. 7, 9

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>		<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>	
<b>1. PRIMARY CONTAINMENT ISOLATION</b>						
a. Reactor Vessel Water Level - Low Low, Level 2	S	M	R	2	1, 2, 3 and #	R
b. Reactor Vessel Water Level - Low Low, Level 2 (ECCS - Division 3)	S	M	R(c)	7	1, 2, 3 and #	R
c. Reactor Vessel Water Level - Low Low Low, Level 1 (ECCS - Division 1 and Division 2)	S	M	R(c)	7	1, 2, 3 and #	R
d. Drywell Pressure - High	S	M	R	2	1, 2, 3	Q
e. Drywell Pressure - High (ECCS - Division 1 and Division 2)	S	M	R(c)	7	1, 2, 3	Q
f. Drywell Pressure - High (ECCS - Division 3)	S	M	R(c)	7	1, 2, 3	Q
g. Containment and Drywell Ventilation Exhaust Radiation - High High	S	M	R		1, 2, 3 and *	R
h. Manual Initiation	NA	M(a)	NA		1, 2, 3 and **	NA
<b>2. MAIN STEAM LINE ISOLATION</b>						
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	M	R	2	1, 2, 3	R
b. Main Steam Line Radiation - High	S	M	R		1, 2, 3	R
c. Main Steam Line Pressure - Low	S	M	R	2	1	Q
d. Main Steam Line Flow - High	S	M	R	2	1, 2, 3	R
e. Condenser Vacuum - Low	S	M	R	2	1, 2**, 3**	Q

GRAND GULF-UNIT 1

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Amendment No. 7, 9

NPE-31205 (XDC1)

ATTACH. 5, 2 OF 5

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED
<u>2. MAIN STEAM LINE ISOLATION (Continued)</u>				
f. Main Steam Line Tunnel Temperature - High	S	H	R	3 1, 2, 3
g. Main Steam Line Tunnel Δ Temp. - High	S	H	R	3 1, 2, 3
h. Manual Initiation	NA	H(a)	NA	1, 2, 3 NA
<u>3. SECONDARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level - Low Low, Level 2	S	H	R	2 1, 2, 3 and #
b. Drywell Pressure - High	S	H	R	2 1, 2, 3
c. Fuel Handling Area Ventilation Exhaust Radiation - High High	S	H	R	1, 2, 3 and *
d. Fuel Handling Area Pool Sweep Exhaust Radiation - High High	S	H	R	1, 2, 3 and *
e. Manual Initiation	NA	H(a)	NA	1, 2, 3 and * NA
<u>4. REACTOR WATER CLEANUP SYSTEM ISOLATION</u>				
a. Δ Flow - High	S	H	R	1, 2, 3 R
b. Δ Flow Timer	NA	H	Q	1, 2, 3 Q
c. Equipment Area Temperature - High	S	H	R	3 1, 2, 3
d. Equipment Area Ventilation Δ Temp. - High High	S	H	R	3 1, 2, 3
e. Reactor Vessel Water Level - Low Low, Level 2	S	H	R	2 1, 2, 3

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TABLE 4.3.2.1-1 (Continued)

ATTACH. 5, 3 DE 5

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	*CHANNEL CALIBRATION	OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED	NOTES
<u>4. REACTOR WATER CLEANUP SYSTEM ISOLATION (Continued)</u>					
f. Main Steam Line Tunnel Ambient Temperature - High	S	H	R	1, 2, 3	Q
g. Main Steam Line Tunnel Δ Temp. - High	S	H	R	1, 2, 3	Q
h. SLCS Initiation	NA	M <sup>(b)</sup>	NA	1, 2, 3	NA
i. Manual Initiation	NA	M <sup>(a)</sup>	NA	1, 2, 3	NA
<u>5. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>					
a. RCIC Steam Line Flow - High	S	H	R	1, 2, 3	Q
b. RCIC Steam Supply Pressure - Low	S	H	R	1, 2, 3	Q
c. RCIC Turbine Exhaust Diaphragm Pressure - High	S	H	R	1, 2, 3	Q
d. RCIC Equipment Room Ambient Temperature - High	S	H	R	1, 2, 3	Q
e. RCIC Equipment Room Δ Temp. - High	S	H	R	1, 2, 3	Q
f. Main Steam Line Tunnel Ambient Temperature - High	S	H	R	1, 2, 3	Q
g. Main Steam Line Tunnel Δ Temp. - High	S	H	R	1, 2, 3	Q



NPE-3-205 (i) (BR)

ATTACH. 5, 4 DFS

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED
<b>5. REACTOR CORE ISOLATION-COOLING-SYSTEM ISOLATION (Continued)</b>				
h. Main Steam Line Tunnel Temperature Timer	NA	H	Q	1, 2, 3
i. RHR Equipment Room Ambient Temperature - High	S	H	R	1, 2, 3
j. RHR Equipment Room Δ Temp. - High	S	H	R	1, 2, 3
k. RHR/RHIC Steam Line Flow - High	S	H	R	1, 2, 3
l. Manual Initiation	NA	H(a)	NA	NA
m. Drywell Pressure-High (CCS Division 1 and Division 2)	S	H	R(c)	1, 2, 3
<b>6. RHR SYSTEM ISOLATION</b>				
a. RHR Equipment Room Ambient Temperature - High	S	H	R	1, 2, 3
b. RHR Equipment Room Δ Temp. - High	S	H	R	1, 2, 3
c. Reactor Vessel Water Level - Low, Level 3	S	H	R	1, 2, 3
d. Reactor Vessel (RHR) Cut-in Permissive Pressure - High	S	H	R	1, 2, 3

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TABLE 4.3.2.1-1 (Continued)

ATTACH. 5, SOFS

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED
6. <u>RHR SYSTEM ISOLATION (Continued)</u>				
e. Drywell Pressure <sup>High</sup> High	S	H <sup>(a)</sup>	R <sub>1</sub>	1, 2, 3
f. Manual Initiation	NA	H <sup>(a)</sup>	NA	1, 2, 3

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NA

- \*When handling irradiated fuel in the primary or secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.
- \*\*When reactor steam pressure  $\geq$  1045 psig and/or any turbine stop valve is open.
- #During CORE ALTERATION and operations with a potential for draining the reactor vessel.
- (a) Manual initiation switches shall be tested at least once per 18 months during shutdown. All other circuitry associated with manual initiation shall receive a CHANNEL FUNCTIONAL TEST at least once per 31 days as part of circuitry required to be tested for automatic system isolation.
- (b) Each train or logic channel shall be tested at least every other 31 days.
- (c) Calibrate trip unit at least once per 31 days.

GRAND GULF-UNIT 1

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Amendment No. 9

TABLE 4.3.3.1-1

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED	
<b>A. DIVISION 1 TRIP SYSTEM</b>					
<b>1. RHR-A (LPCI MODE) AND LPCS SYSTEM</b>					
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	H	4	R(a)	1, 2, 3, 4*, 5* R
b. Drywell Pressure - High	S	H	4	R(a)	1, 2, 3 R
c. LPCI Pump A Start Time Delay Relay	NA	H		Q(d)	1, 2, 3, 4*, 5* Q
d. Manual Initiation	HA	R(b)(c)	5	Q(d)	1, 2, 3, 4*, 5* Q
<b>2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM</b>					
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	M	4	R(a)	1, 2, 3 R
b. Drywell Pressure - High	S	H	4	R(a)	1, 2, 3 R
c. ADS Timer	NA	H		Q	1, 2, 3 R
d. Reactor Vessel Water Level - Low, Level 3	S	H	4	R(a)	1, 2, 3 R
e. LPCS Pump Discharge Pressure - High	S	M	6	R	1, 2, 3 Q
f. LPCI Pump A Discharge Pressure - High	S	H	4	R(a)	1, 2, 3 Q
g. Manual Initiation	NA	R(b)	1	NA	1, 2, 3 NA
<b>B. DIVISION 2 TRIP SYSTEM</b>					
<b>1. RHR B AND C (LPCI MODE)</b>					
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	H	4	R(a)	1, 2, 3, 4*, 5* R
b. Drywell Pressure - High	S	H	4	R(a)	1, 2, 3 R
c. LPCI Pump B Start Time Delay Relay	NA	H		Q(d)	1, 2, 3, 4*, 5* Q
d. Manual Initiation	HA	R(b)(c)	5	Q(d)	1, 2, 3, 4*, 5* Q

GRAND GULF UNIT

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Amendment No. 8

TABLE 4.3.3.1-1 (Cont Inued)  
EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
<b>B. DIVISION 2 TRIP SYSTEM (Cont Inued)</b>				
<b>2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "B" #</b>				
a. Reactor Vessel Water Level - Low Low, Level 1	S	H	4 R(a)	1, 2, 3 R
b. Drywell Pressure-High	S	H	4 R(a)	1, 2, 3 R
c. ADS Timer	NA	H	Q	1, 2, 3 R
d. Reactor Vessel Water Level - Low, Level 3	S	H	4 R(a)	1, 2, 3 R
e. LPCI Pump B and C Discharge Pressure-High	S	H(b)	4 R(a)	1, 2, 3 R
f. Manual Initiation	NA	R	NA	1, 2, 3 R
<b>C. DIVISION 3 TRIP SYSTEM</b>				
<b>1. LCS SYSTEM</b>				
a. Reactor Vessel Water Level - Low Low, Level 2	S	H	4 R(a)	1, 2, 3, 4 <sup>A</sup> , 5 <sup>A</sup> R
b. Drywell Pressure-High #/ #	S	H	4 R(a)	1, 2, 3, 4 <sup>A</sup> , 5 <sup>A</sup> R
c. Reactor Vessel Water Level-High, Level B	S	H	4 R(a)	1, 2, 3, 4 <sup>A</sup> , 5 <sup>A</sup> R
d. Condensate Storage Tank Level - Low	S	H	4 R(a)	1, 2, 3, 4 <sup>A</sup> , 5 <sup>A</sup> R
e. Suppression Pool Water Level - High	S	H(b)	4 R(a)	1, 2, 3, 4 <sup>A</sup> , 5 <sup>A</sup> R
f. Manual Initiation #/ #	NA	R	NA	1, 2, 3, 4 <sup>A</sup> , 5 <sup>A</sup> R
<b>D. LOSS OF POWER</b>				
<b>1. Division 1 and 2</b>				
a. 4.16 kV Bus Undervoltage (Loss of Voltage)	NA	H(e)	R	1, 2, 3, 4 <sup>AA</sup> , 5 <sup>AA</sup> R
b. 4.16 kV Bus Undervoltage (BOP Load Shed)	NA	H(z)	R	1, 2, 3, 4 <sup>AA</sup> , 5 <sup>AA</sup> R
c. 4.16 kV Bus Undervoltage (Degraded Voltage)	NA	H(e)	R	1, 2, 3, 4 <sup>AA</sup> , 5 <sup>AA</sup> R
<b>2. Division 3</b>				
a. 4.16 kV Bus Undervoltage (Loss of Voltage)	NA	H(e)	R	1, 2, 3, 4 <sup>AA</sup> , 5 <sup>AA</sup> R

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Amendment No. 6, 10

Temperature Control for the primary and secondary pumps and also for the instrumentation

Tech. Spec. Calibration Frequencies - Notes

1. Footnote (g) in the Tech Specs specifies trip unit calibration every 31 days. This exceeds BWR-5 STS requirements.
2. To be consistent with the RPS and ECCS instrument surveillance requirements, Rosemount trip units for the isolation function should be calibrated monthly. This will exceed BWR-5 STS requirements. It is recommended that a footnote be added to the Tech Specs for this item.
3. BWR-5 STS require channel calibration quarterly, whereas Grand Gulf Tech Specs require it on an 18 month cycle. Some of the thermocouples are in inaccessible areas and it is impractical and unnecessary to calibrate them quarterly. However it is recommended that the Tech Specs be modified to require quarterly or monthly calibration of the temperature switches. Not valid justification
4. Footnote (a) in the Tech Specs specifies trip unit calibration every 31 days. This exceeds BWR-5 STS requirements.
5. Channel calibration for manual initiation should be NA. This change has already been submitted to the NRC.
6. LPCS pump discharge pressure should have footnote (a) applied to be consistent with other parameters in this table.
7. Footnote (c) in The Tech Specs specifies trip unit calibration every 31 days. This exceeds BWR-5 STS requirements.



*File (LCS)*



MISSISSIPPI POWER & LIGHT COMPANY

*Helping Build Mississippi*

P. O. BOX 1840, JACKSON, MISSISSIPPI 39205

December 15, 1983

NUCLEAR PRODUCTION DEPARTMENT

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U. S. Nuclear Regulatory Commission  
Office of Nuclear Reactor Regulation  
Washington, D. C. 20555

Attention: Mr. Harold R. Denton, Director

Dear Mr. Denton:

SUBJECT: Grand Gulf Nuclear Station  
Unit 1  
Docket No. 50-416  
License No. NPF-13  
File 0260/L-860.0  
Technical Specification  
Terminology, ICSB Concerns  
AECM-83/0764

- REFERENCE: 1) AECM-83/0519, letter to NRC, dated September 12, 1983, concerning Technical Specification Terminology, ICSB Concerns.
- 2) Letter from A. Schwencer to J. P. McGaughey, September 12, 1983.

Mississippi Power & Light (MP&L) committed in Reference 1 to provide the Nuclear Regulatory Commission (NRC) with certain Technical Specification definitions by December 15, 1983. Definitions for the terms "channel", "trip system", and "trip function" are provided as an attachment to this letter both as generic definitions and specifically for the instrumentation in Technical Specification Table 3.3.2-1, Isolation Actuation Instrumentation. These definitions for the remaining instrumentation in the Technical Specifications will be provided by February 15, 1984. Also, identification of Technical Specification changes, if any, will be provided by February 15, 1984.

Please contact this office if additional information is desired.

Yours truly,

L. F. Dale  
Manager of Nuclear Services

WJR/SRH:sap  
Attachment

cc: (See Next Page)



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File (Project) (w/a) [18]

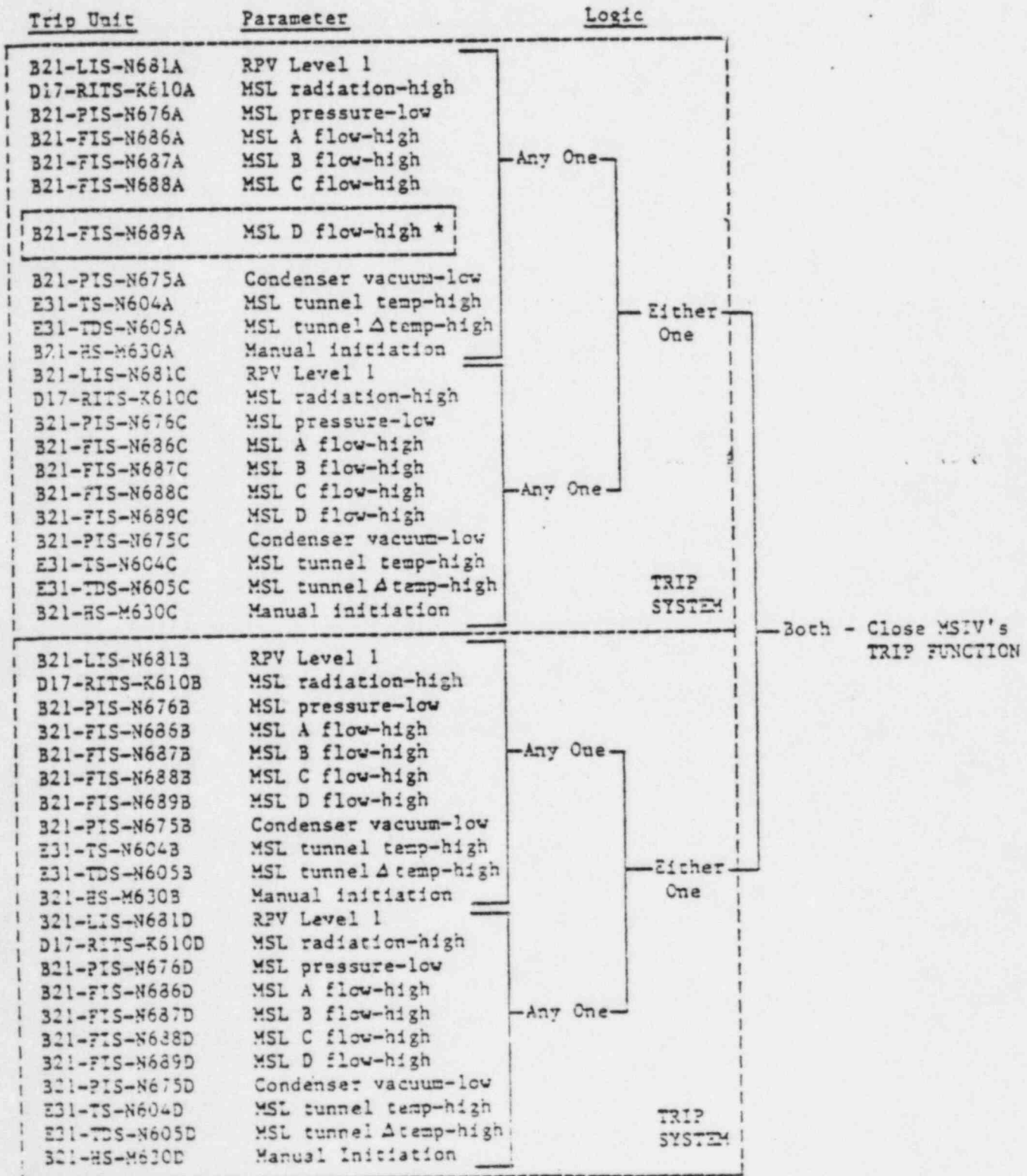
DEFINITIONS FOR THE TERMS  
"CHANNEL", "TRIP SYSTEM", AND "TRIP FUNCTION"  
FOR INSTRUMENTATION IN TECHNICAL SPECIFICATION  
TABLE 3.3.2-1

The definitions presented in this attachment are provided as both generic and instrumentation specific definitions. The generic definitions apply to all cases where the terms "Channel", "Trip System" and "Trip Functions" are used in the Technical Specifications. Specific definitions are provided in tabular form for each valve group in Technical Specification Table 3.3.2-1, Isolation Actuation Instrumentation.

Generic Definitions

- 1) Channel - A group of devices which is sufficient to produce only one trip signal for only one parameter. A channel consists of all components from sensor through trip unit (or trip relay, when provided).
- 2) Trip System - A group of devices including instrument channels and logic components, which is required to produce a trip of an actuation device or group of actuation devices. The tripping of a trip system may or may not produce a trip function.
- 3) Trip Function - The initiation of actuation devices and equipment necessary to produce the system response required by plant conditions.

DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 1 ISOLATION (MSIV CLOSURE)



\* One Channel (typical of 44 shown on this page)

DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 1 ISOLATION (MSL DRAIN CLOSURE)

Trip Unit	Parameter	Logic	
B21-LIS-N681A	RPV Level 1	Any One	Close outboard MSL drain valves TRIP FUNCTION
D17-RITS-K610A	MSL radiation-high		
B21-PIS-N676A	MSL pressure-low		
B21-FIS-N686A	MSL A flow-high		
B21-FIS-N687A	MSL B flow-high		
B21-FIS-N688A	MSL C flow-high		
B21-FIS-N689A	MSL D flow-high *	Both	
B21-PIS-N675A	Condenser vacuum-low	Any One	
E31-TS-N604A	MSL tunnel temp-high		
E31-TDS-N605A	MSL tunnel Δtemp-high		
B21-HS-M630A	Manual initiation		
B21-LIS-N681D	RPV Level 1		
D17-RITS-K610D	MSL radiation-high		
B21-PIS-N676D	MSL pressure-low		
B21-FIS-N686D	MSL A flow-high		
B21-FIS-N687D	MSL B flow-high		
B21-FIS-N688D	MSL C flow-high		
B21-FIS-N689D	MSL D flow-high		
B21-PIS-N675D	Condenser vacuum-low	TRIP SYSTEM	
E31-TS-N604D	MSL tunnel temp-high		
E31-TDS-N605D	MSL tunnel Δtemp-high		
B21-HS-M630D	Manual initiation		
B21-LIS-N681B	RPV Level 1	Any One	Close inboard MSL drain valves TRIP FUNCTION
D17-RITS-K610B	MSL radiation-high		
B21-PIS-N676B	MSL pressure-low		
B21-FIS-N686B	MSL A flow-high		
B21-FIS-N687B	MSL B flow-high		
B21-FIS-N688B	MSL C flow-high		
B21-FIS-N689B	MSL D flow-high		
B21-PIS-N675B	Condenser vacuum-low	Any One	
E31-TS-N604B	MSL tunnel temp-high		
E31-TDS-N605B	MSL tunnel Δtemp-high		
B21-HS-M630B	Manual initiation		
B21-LIS-N681C	RPV Level 1		
D17-RITS-K610C	MSL radiation-high		
B21-PIS-N676C	MSL pressure-low		
B21-FIS-N686C	MSL A flow-high		
B21-FIS-N687C	MSL B flow-high		
B21-FIS-N688C	MSL C flow-high		
B21-FIS-N689C	MSL D flow-high		
B21-PIS-N675C	Condenser vacuum-low	TRIP SYSTEM	
E31-TS-N604C	MSL tunnel temp-high		
E31-TDS-N605C	MSL tunnel Δtemp-high		
B21-HS-M630C	Manual Initiation		

\* One Channel (typical of 44 shown on this page)

DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 3 ISOLATION (EXCEPT E12-F037A AND B)

Trip Unit	Parameter	Logic		
E31-TS-N608A	RHR Area #1 ambient temp-high	Two out of two	Any One	Close outboard Group 3 valves TRIP FUNCTION
E31-TS-N610A	RHR Area #2 ambient temp-high			
E31-TDS-N600A	RHR Area #1 $\Delta$ temp-high			
E31-TDS-N611A	RHR Area #2 $\Delta$ temp-high			
B21-LIS-N680A	RPV Level 3 *	Two out of two	Any One	Close outboard Group 3 valves TRIP FUNCTION
B21-LIS-N680D	RPV Level 3			
B21-PS-N679A	Reactor pressure-high	Two out of two	TRIP SYSTEM	
B21-PS-N679D	Reactor pressure-high			
B21-HS-M630A	Manual initiation			
B21-HS-M630D	Manual initiation			
E31-TS-N608B	RHR Area #1 ambient temp-high	Two out of two	Any One	Close inboard Group 3 valves TRIP FUNCTION
E31-TS-N610B	RHR Area #2 ambient temp-high			
E31-TDS-N600B	RHR Area #1 $\Delta$ temp-high			
E31-TDS-N611B	RHR Area #2 $\Delta$ temp-high			
B21-LIS-N680B	RPV Level 3	Two out of two	Any One	Close inboard Group 3 valves TRIP FUNCTION
B21-LIS-N680C	RPV Level 3			
B21-PS-N679B	Reactor pressure-high	Two out of two	TRIP SYSTEM	
B21-PS-N679C	Reactor pressure-high			
B21-HS-M630B	Manual initiation			
B21-HS-M630C	Manual initiation			

\* One Channel (typical of 20 shown on this page)



DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 3 ISOLATION (E12-F037A AND B ONLY)

Trip Unit	Parameter	Logic	
E31-TS-N608A	RHR Area #1 ambient temp-high		
E31-TS-N610A	RHR Area #2 ambient temp-high		
E31-TDS-N600A	RHR Area #1 $\Delta$ temp-high		
E31-TDS-N611A	RHR Area #2 $\Delta$ temp-high		
B21-LIS-N680A	RPV Level 3 *	One-out-of-two	Any One
B21-LIS-N680D	RPV Level 3		
C71-PIS-N650A	Drywell pressure-high	Two out of two	TRIP SYSTEM
C71-PIS-N650D	Drywell pressure-high		
B21-HS-M630A	Manual initiation	Two out of two	TRIP SYSTEM
B21-HS-M630D	Manual initiation		
E31-TS-N608B	RHR Area #1 ambient temp-high		
E31-TS-N610B	RHR Area #2 ambient temp-high		
E31-TDS-N600B	RHR Area #1 $\Delta$ temp-high		
E31-TDS-N611B	RHR Area #2 $\Delta$ temp-high		
B21-LIS-N680B	RPV Level 3	One-out-of-two	Any One
B21-LIS-N680C	RPV Level 3		
C71-PIS-N650B	Drywell pressure-high	Two out of two	TRIP SYSTEM
C71-PIS-N650C	Drywell pressure-high		
B21-HS-M630B	Manual initiation	Two out of two	TRIP SYSTEM
B21-HS-M630C	Manual initiation		

\* One Channel (typical of 20 shown on this page)

DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 4 ISOLATION

Trip Unit	Parameter	Logic	
E31-FIS-N683A	RCIC steam line flow-high *		
E31-PIS-N685A	RCIC steam supply pressure-low		
E31-PIS-N655A	RCIC turb exh diaph pressure-high	Two out of two	
E31-PIS-N655E	RCIC turb exh diaph pressure-high		
E31-TS-N602A	RCIC room ambient temp-high		
E31-TDS-N603A	RCIC room Δtemp-high		
E31-TS-N604E	MSL tunnel ambient temp-high	Either one	Two out of two
E31-TDS-N605E	MSL tunnel Δtemp-high		
E31-KIS-R617E	MSL tunnel temp timer		Any one
E31-TS-N608A	RHR Area #1 ambient temp-high		Close outboard Group 4 valves TRIP FUNCTION
E31-TS-N610A	RHR Area #2 ambient temp-high		
E31-TDS-N600A	RHR Area #1 Δtemp-high		
E31-TDS-N611A	RHR Area #2 Δtemp-high		
E31-FIS-N684A	RHR/RCIC steam line flow-high		
E31-RS-M627	Manual initiation (Note k)		
			TRIP SYSTEM
E31-FIS-N683B	RCIC steam line flow-high		
E31-PIS-N685B	RCIC steam supply pressure-low		
E31-PIS-N655B	RCIC turb exh diaph pressure-high	Two out of two	
E31-PIS-N655F	RCIC turb exh diaph pressure-high		
E31-TS-N602B	RCIC room ambient temp-high		
E31-TDS-N603B	RCIC room Δtemp-high		
E31-TS-N604F	MSL tunnel ambient temp-high	Either one	Two out of two
E31-TDS-N605F	MSL tunnel Δtemp-high		
E31-KIS-R617F	MSL tunnel temp timer		Any one
E31-TS-N608B	RHR Area #1 ambient temp-high		Close inboard Group 4 valves TRIP FUNCTION
E31-TS-N610B	RHR Area #2 ambient temp-high		
E31-TDS-N600B	RHR Area #1 Δtemp-high		
E31-TDS-N611B	RHR Area #2 Δtemp-high		
E31-FIS-N684B	RHR/RCIC steam line flow-high		

\* One Channel (typical of 29 shown on this page)

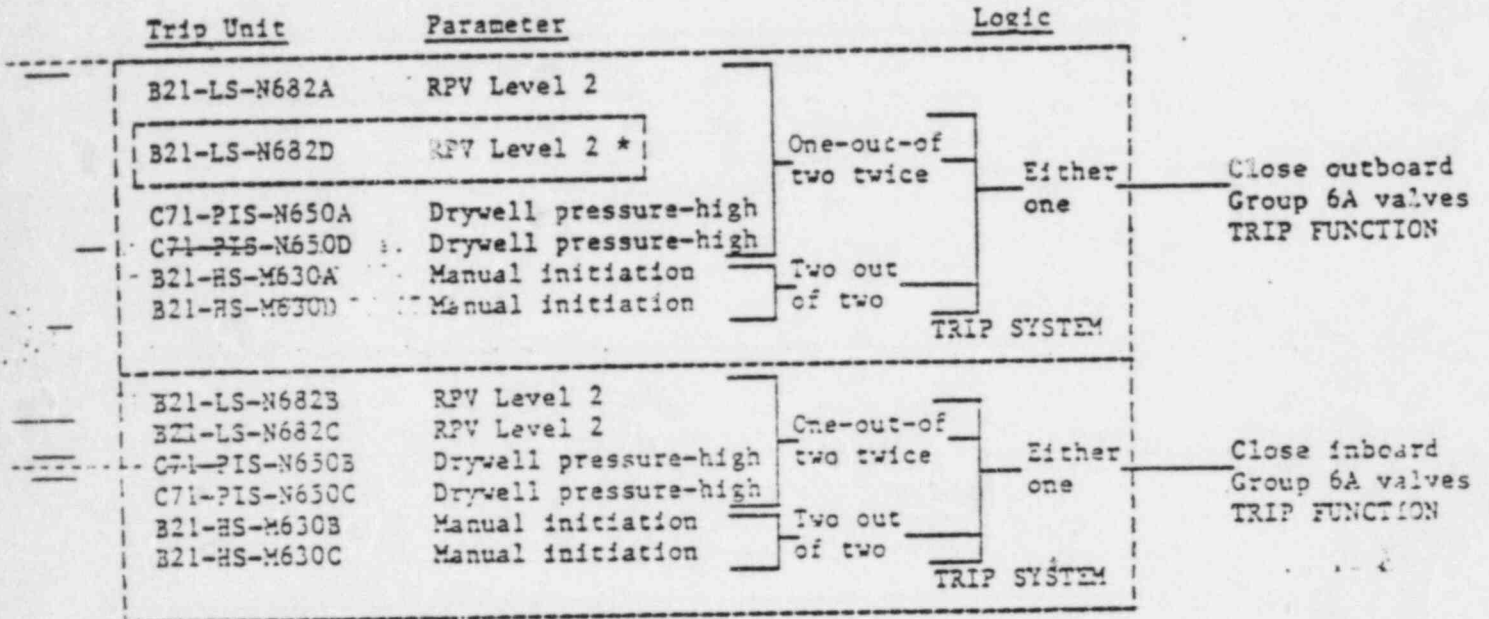
Note k - A concurrent RCIC initiation signal is required for isolation to occur.

DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 5 ISOLATION

<u>Trip Unit</u>	<u>Parameter</u>	<u>Logic</u>	
B21-LIS-N691A	RPV Level 1	One-out-of two twice TRIP SYSTEM	Close outboard Group 5 valves TRIP FUNCTION
B21-LIS-N691E	RPV Level 1		
B21-PIS-N694A	Drywell pressure-high		
B21-PIS-N694E	Drywell pressure-high		
B21-LIS-N691B *	RPV Level 1 *	One-out-of two twice TRIP SYSTEM	Close inboard Group 5 valves TRIP FUNCTION
B21-LIS-N691F	RPV Level 1		
B21-PIS-N694B	Drywell pressure-high		
B21-PIS-N694F	Drywell pressure-high		

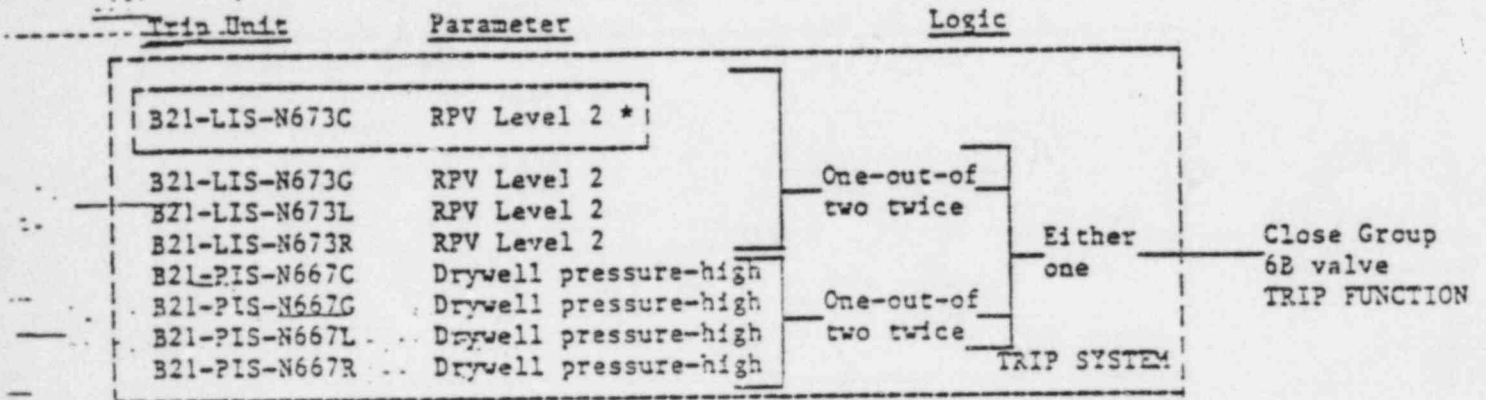
\* One Channel (typical of 8 shown on this page)

DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 6A ISOLATION



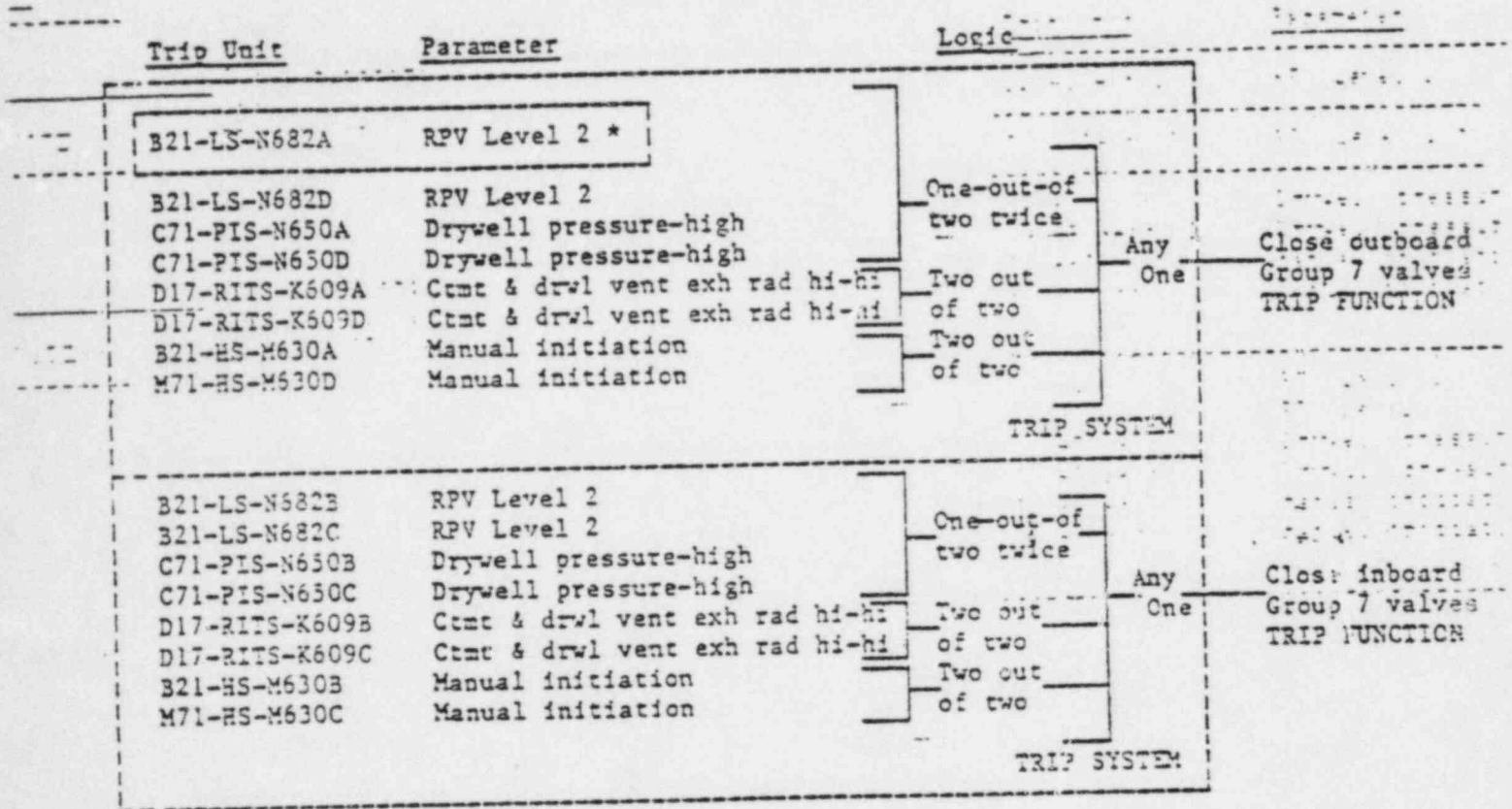
\* One Channel (typical of 10 shown on this page)

DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 6B ISOLATION



\* One Channel (typical of 8 shown on this page)

DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 7 ISOLATION



\* One Channel (typical of 16 shown on this page)



DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 8 ISOLATION

Trip Unit	Parameter	Logic	
E31-FS-N609A	RWCU $\Delta$ flow-high	Two out of two	
E31-KIS-R615A	RWCU $\Delta$ flow-timer		
E31-TS-N620A	RWCU area temp-high	Any One	Close outboard Group 8 valves TRIP FUNCTION
E31-TS-N621A	RWCU area temp-high		
E31-TS-N622A	RWCU area temp-high *		
E31-TS-N623A	RWCU area temp-high		
E31-TS-N624A	RWCU area temp-high		
E31-TS-N625A	RWCU area temp-high		
E31-TS-N626A	RWCU area temp-high		
E31-TS-N627A	RWCU area temp-high		
E31-TDS-N612A	RWCU area $\Delta$ temp-high		
E31-TDS-N613A	RWCU area $\Delta$ temp-high		
E31-TDS-N614A	RWCU area $\Delta$ temp-high	Two out of two	
E31-TDS-N615A	RWCU area $\Delta$ temp-high		
E31-TDS-N616A	RWCU area $\Delta$ temp-high	Two out of two	TRIP SYSTEM
E31-TDS-N617A	RWCU area $\Delta$ temp-high		
E31-TDS-N618A	RWCU area $\Delta$ temp-high		
E31-TDS-N619A	RWCU area $\Delta$ temp-high		
B21-LS-N682A	RPV Level 2		
B21-LS-N682D	RPV Level 2		
E31-TS-N604A	MSL tunnel temp-high		
E31-TDS-N605A	MSL tunnel $\Delta$ temp-high		
C41-HS-M601A	SLCS initiation (Note 1)		
B21-HS-M630A	Manual initiation		
B21-HS-M630D	Manual initiation		

\* One Channel (typical of 25 shown on this page)  
 Note 1 - Closes only valve G33-F004

DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 8 ISOLATION (CONTINUED)

Trip Unit	Parameter	Logic	
E31-FS-N609B	RWCU $\Delta$ flow-high	Two out of two	
E31-KIS-R615B	RWCU $\Delta$ flow-timer		
E31-TS-N620B	RWCU area temp-high		
E31-TS-N621B	RWCU area temp-high		
E31-TS-N622B	RWCU area temp-high *		
E31-TS-N623B	RWCU area temp-high	Any One	Close inboard Group 8 valves TRIP FUNCTION
E31-TS-N624B	RWCU area temp-high		
E31-TS-N625B	RWCU area temp-high		
E31-TS-N626B	RWCU area temp-high		
E31-TS-N627B	RWCU area temp-high		
E31-TDS-N612B	RWCU area $\Delta$ temp-high		
E31-TDS-N613B	RWCU area $\Delta$ temp-high		
E31-TDS-N614B	RWCU area $\Delta$ temp-high		
E31-TDS-N615B	RWCU area $\Delta$ temp-high		
E31-TDS-N616B	RWCU area $\Delta$ temp-high		
E31-TDS-N617B	RWCU area $\Delta$ temp-high		
E31-TDS-N613B	RWCU area $\Delta$ temp-high		
E31-TDS-N619B	RWCU area $\Delta$ temp-high		
B21-LS-N5823	RPV Level 2	Two out of two	
B21-LS-N682C	RPV Level 2		
E31-TS-N604B	MSL tunnel temp-high	Two out of two	
E31-TDS-N605B	MSL tunnel $\Delta$ temp-high		
C41-HS-M601B	SLCS initiation (Note 1)		
B21-HS-M630B	Manual initiation	Two out of two	TRIP SYSTEM
B21-HS-M630C	Manual initiation		

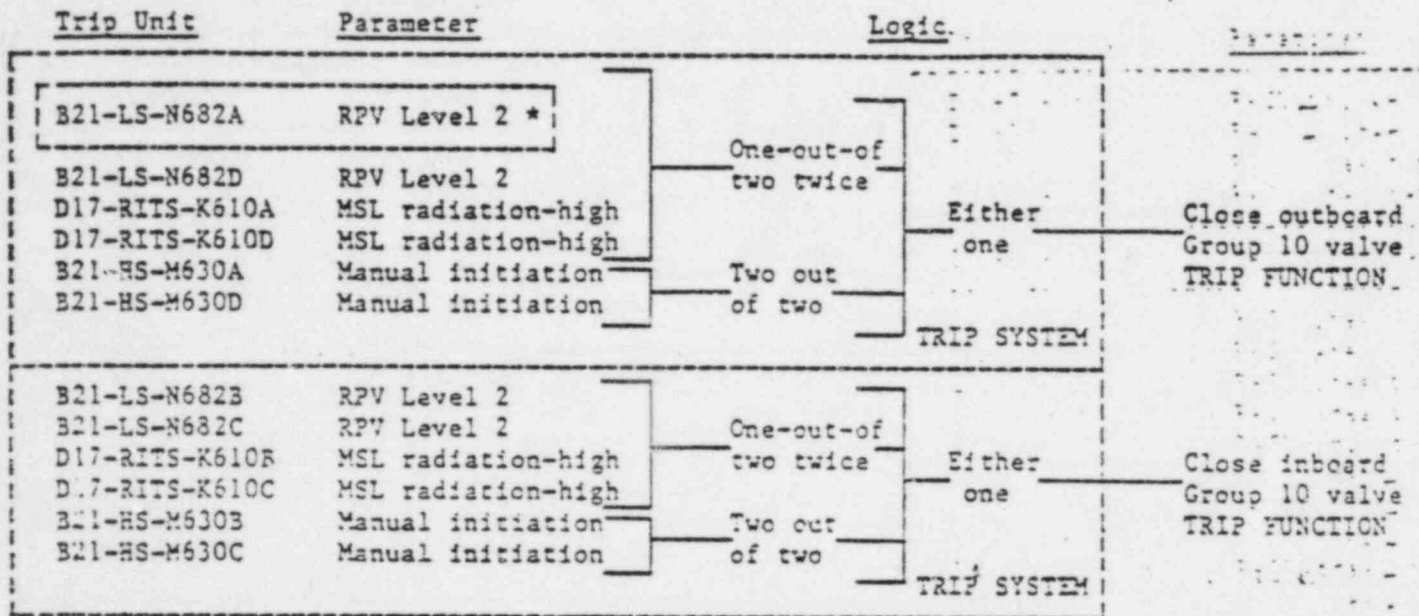
\* One Channel (typical of 25 shown on this page)  
 Note 1 - Closes only valve G33-F001 & F251

DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 9 ISOLATION

<u>Trip Unit</u>	<u>Parameter</u>	<u>Logic</u>	
E31-PIS-N685A	RCIC steam supply pressure-low *	Two out of two TRIP SYSTEM	Close outboard Group 9 valve TRIP FUNCTION
B21-PIS-N694A	Drywell pressure-high		
E31-PIS-N685B	RCIC steam supply pressure-low	Two out of two TRIP SYSTEM	Close inboard Group 9 valve TRIP FUNCTION
B21-PIS-N694B	Drywell pressure-high		

\* One Channel (typical of 4 shown on this page)

DEFINITIONS FOR  
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"  
 FOR GROUP 10 ISOLATION



\* One Channel (typical of 12 shown on this page)

File (LCTS)



MISSISSIPPI POWER & LIGHT COMPANY

Helping Build Mississippi

P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

September 12, 1983

NUCLEAR PRODUCTION DEPARTMENT

34

U. S. Nuclear Regulatory Commission  
Office of Nuclear Reactor Regulation  
Washington, D. C. 20555

Attention: Mr. Harold R. Denton, Director

Dear Mr. Denton:

SUBJECT: Grand Gulf Nuclear Station -  
Unit 1  
Docket No. 50-416  
License No. NPF-13  
File 0260/L-860.0  
Technical Specification  
Terminology, ICSB Concerns  
AECM-83/0519

Recent discussions have been held between Mississippi Power & Light (MP&L) representatives and the Nuclear Regulatory Commission (NRC) Instrumentation and Control Systems Branch (ICSB) reviewer pertaining to the Grand Gulf Nuclear Station (GGNS) Technical Specifications, with particular emphasis on MP&L definitions and applications of certain commonly used terminology. The terms "channel", "trip system", and "trip function" are not considered by the NRC staff to be clearly defined in either the GGNS Technical Specifications or the General Electric Standard Technical Specifications (3WR/6). The NRC Staff also questioned whether the single failure criterion was considered in arriving at the minimum number of channels required to be operable for each reactor protection system trip function. In addition, the NRC Staff requested that MP&L provide the methods that will be used to place an inoperable instrument channel in the tripped condition as permitted by certain Technical Specification action statements.

The attachment to this letter addresses each of the concerns stated above. MP&L will perform a thorough review in a timely manner to clearly define the subject Technical Specification terms and to provide verification that the single failure criterion has been properly applied. The results of the review effort will be submitted to the NRC on the schedule presented in the attachment to this letter. A preliminary review of approximately 130 instrumentation items indicated that only one necessitates a Technical Specification change. The proposed change will be submitted within 30 days of this letter.

Please contact this office if additional information is desired.

Yours truly,

A handwritten signature in dark ink, appearing to read 'L. F. Dale', with a stylized flourish at the end.

L. F. Dale  
Manager of Nuclear Services

MJC/JGC:sap  
cc: (See Next Page)

MISSISSIPPI POWER & LIGHT COMPANY

cc: Mr. J. B. Richard (w/a)  
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Mr. T. B. Conner (w/o)  
Mr. G. B. Taylor (w/o)

Mr. Richard C. DeYoung, Director (w/a)  
Office of Inspection & Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

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Mr. A. G. Wagner (w/a) -  
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Mr. J. F. Pinto (w/a)  
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Mr. A. S. McCurdy (w/o)  
Middle South Services  
Nuclear Activities (w/a)  
File (LCTS) (w/2)  
File (Plant) (w/a)  
File (Project) (w/a) [ 7 ]

The following items describe areas of discussion between NRC and Mississippi Power & Light Company (MP&L). Proposed schedules for completion of input to the NRC are provided where applicable.

1. Concern: The Instrumentation and Control Systems Branch (ICSB) of the NRC indicated that the terms "channel", "trip system" and "trip function" are used throughout the Technical Specifications without being clearly defined.

Response: The definition of these terms is complicated by their use within individual systems or the plant as a whole. A "channel" may mean one or possibly more instruments whose logic input then determines the "trip system" output. MP&L proposes to provide, in a timely manner, definitions for these terms used in the Technical Specifications. Since the initial thrust has been towards the isolation valve instrumentation, definitions will be provided by December 15, 1983, for this instrumentation. Definitions for other instrumentation in the Technical Specifications will be provided by February 15, 1984. Also, FSAR changes where applicable, as a result of this work effort will be identified by February 15, 1984, and incorporated into the FSAR via the initial update required by 10 CFR 50.72. This update will be submitted by June, 1984.

2. Concern: The NRC questioned whether or not single failure criterion was considered in determining the number of channels required to be operable for all cases in the Technical Specifications.

Response: The present Technical Specifications list Minimum Number of Channels per Trip System or Minimum Number of Channels per Trip Function for various instrumentation. As a part of the review effort required to provide the definitions of "channel", "trip system," and "trip function", MP&L will provide verification that the single failure criterion has been properly applied where applicable. Verification for isolation actuation instrumentation will be provided by December 15, 1983 and for the remaining instrumentation by February 15, 1984. To provide assurance to the NRC that present requirements for number of channels, trip systems, and trip functions are adequate for plant startup and operations in the interim, a preliminary review of the instrumentation requirements in the Grand Gulf Technical Specifications has been completed.

The preliminary review included the following nine Technical Specification Tables and two Specifications:

- a. Table 3.3.1-1, Reactor Protection System Instrumentation
- b. Table 3.3.2-1, Isolation Actuation Instrumentation
- c. Table 3.3.3-1, Emergency Core Cooling Actuation Instrumentation
- d. Table 3.3.4.1-1, ATWS Recirculation Pump Trip System Instrumentation
- e. Table 3.3.4.2-1, End-of-Cycle Recirculation Pump Trip System Instrumentation
- f. Table 3.3.5-1, Reactor Core Isolation Cooling System Actuation Instrumentation
- g. Table 3.3.6-1, Control Rod Block Instrumentation
- h. Table 3.3.7.1-1, Radiation Monitoring Instrumentation
- i. Table 3.3.8-1, Plant Systems Actuation Instrumentation
- j. Technical Specification 3.4.2.1, Safety/Relief Valves
- k. Technical Specification 3.4.2.2, Safety/Relief Valves Low-Low Set Function

Of the approximately 130 items reviewed, only two items were identified as potential problems with respect to the concerns identified by ICSB. As discussed below, only one of these necessitates a Technical Specification change.

- a) Table 3.3.5-1 lists the actuation instrumentation for the Reactor Core Isolation Cooling (RCIC) System. Presently the table requires two minimum operable channels per trip system for Reactor Vessel Water Level - Low Low, Level 2. RCIC initiates on one-out-of-two-twice low level logic. To ensure a success path for initiation, all four low level channels are required operable. The RCIC system is not considered a safety related system, and consequently, single failure criteria is not required. However, because RCIC is redundant to HPCS for the safe shutdown function, and may be required under certain abnormal conditions (FSAR Section 7.4.1.1.1.1), MP&L will submit necessary Technical Specification changes within thirty days of this letter. The proposed change will consist primarily of increasing the number of minimum operable channels to four.
  - b) Table 3.3.8-1 contains Containment Spray System Actuation Instrumentation. According to FSAR Section 7.3.2.4.3.1.2, single failure criteria is met with the present minimum of one operable channel per trip system for Containment Spray Actuation Instrumentation because redundancy in equipment and control logic circuitry is provided. The worst case single failure would be limited to the disabling of the initiation of only one loop. Therefore, it is not credible that the complete containment spray system would be rendered inoperative following a single failure. MP&L will evaluate increasing the minimum operable channels for actuation instrumentation from the present one per trip system to two per trip system. This evaluation is being pursued not as a safety concern but to enhance system reliability since single failure criteria is met with the present Technical Specification requirements.
3. Concern: Problems or concerns from other plants have prompted NRC questions on the methods used by Grand Gulf to bypass instrumentation. The NRC concern involves cases where reactor protection system instrumentation channels are placed in the tripped condition using lifted leads or temporary jumpers.

Response: MP&L does not normally use lifted leads or temporary jumpers to place reactor protection system instrumentation channels in the tripped condition. However, if jumpers or lifted leads are utilized, the resulting modification is considered a temporary alteration. Administrative controls on the temporary alteration program include a 10 CFR 50.59 review prior to implementation of the alteration.

Technical Specification ACTION statements for instrumentation typically include "place the inoperable channel and/or that trip system in the tripped condition within one hour," or similar requirements. The following are examples of acceptable methods which may be used with the various hardware at GGNS. Concerns about the seismic evaluation of the tripping mechanism have been considered.

Method 1: Rosemount trip units (which trip on open circuit) can be placed in the tripped condition by using the calibration station. This can only be done for one trip unit within a card file, which then limits the ability to calibrate or functionally test other units in the same card file. An alternate acceptable method is removing the trip unit. Removal of cards can also be used for non-Rosemount hardware logic which trips on open circuits.

Method 2: Rosemount trip units (which do not trip on open circuit) can be placed in the tripped condition by using the calibration station. This can only be done for one trip unit within a card file, which then limits the ability to calibrate or functionally test other units in the same card file. An alternate acceptable method is by inserting a trip unit which has been modified to be in the trip condition at all times.

Method 3: The hardware is designed to be in the tripped condition whenever the trip unit switch is taken out of "Operate".

Method 4: The hardware is designed to be in the trip condition when local switches are taken out of their normal position, and the input signal is adjusted to be above/below the trip setpoint. Trip signals for some hardware is not provided by just taking the local switch out of the normal position. For this hardware a trip signal must also be placed in the circuit by adjusting the input signal.



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 037

Priority: 1A<sup>B</sup>

Identified By / Date

Responsible Supervisor

Tech Spec Reference: 3/4.3.2

Problem Title: Riley & Rosemount Trip Units

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Specs 4.3.1.1-1 and 4.3.3.1-1 require monthly cal of Rosemount trip units. Contrary to this, spec 4.3.2 does not require monthly cal of Rosemount trip units. Tech Spec requirement for temp switch cal (Riley) is every 18 monthes. Vendor, however, recommends a yearly cal frequency.

2. Safety Significance:

3. Anticipated Resolution: Change cal frequency for Riley Temp switches from once/18 monthes to once/12 monthes and the Rosemount trip unit cal from once/18 monthes to once/30 days. *Programatically increase surveillance frequency.*

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_  
Individual Notified Date / Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

TRANSMITTAL OF PROPOSED CHANGES  
TO GRAND GULF TECHNICAL SPECIFICATIONS

SUBJECT: NS-83/43

Technical Specification Table 4.3.2.1-1, pages 3/4 3-20 through 3/4 3-23a.

DISCUSSION: Mississippi Power & Light (MP&L) Company committed in letter AECM-83/0640 dated October 14, 1983 to propose certain changes to the Technical Specifications. The two changes proposed herein to Technical Specification Table 4.3.2.1-1; Isolation Actuation Instrumentation Surveillance Requirements, are submitted in satisfaction of that commitment:

- 1) Footnote (c) which requires trip unit calibration at least once per 31 days is added to all Rosemont trip units which do not presently refer to footnote (c). This change involves items 1.a, 1.d, 2.a, 2.c, 2.d, 2.e, 3.a, 3.b, 4.e, 5.a, 5.b, 5.c, 5.k, 6.c, 6.d, and 6.e.
- 2) The present channel calibration frequency for Riley Temperature Switches is at least once per 18 months (R). The proposed change is to at least once per 366 days (A). Items affected by this change are 2.f, 2.g, 4.c, 4.d, 4.f, 4.g, 5.d, 5.e, 5.f, 5.g, 5.i, 5.j, 6.a, and 6.b.

JUSTIFICATION: Technical Specification Tables 4.3.1.1-1 and 4.3.3.1-1 for the Reactor Protection System and the Emergency Core Cooling Systems require monthly calibration frequencies for Rosemont Trip Units which exceeds the six month frequency recommended by the manufacturer. This monthly calibration frequency is not specified for all the Rosemont Trip Units associated with instrumentation in Table 4.3.2.1-1 (Isolation Actuation Instrumentation). Although not required by Tech Specs, all Rosemont Trip Units are calibrated on a monthly frequency. To ensure consistency among the Technical Specification Instrumentation Tables, changes to Table 4.3.2.1-1 are requested to add the monthly calibration of all Rosemont Trip Units to the Tech Specs (i.e., the setpoints will be verified and adjusted as necessary in accordance with Technical Specification requirements).

Technical Specification Table 4.3.2.1-1 for the Isolation Actuation Instrumentation presently requires channel calibration on a refueling frequency for temperature monitoring instrumentation. The present refueling calibration frequency corresponds to the calibration frequency assumed in the setpoint methodology by the Nuclear Steam Supply System (NSSS) vendor. However, the temperature switch manufacturer (Globe) recommends a yearly frequency for temperature switch calibration. MP&L is currently meeting the yearly calibration requirements, however, to provide consistency between the NSSS and the manufacturer's recommendations, the changes proposed herein will require annual calibration.



## SIGNIFICANT HAZARDS CONSIDERATION:

These proposed changes are made to promote consistency among Technical Specification Instrumentation Tables and between manufacturer's recommended calibration frequency and the Technical Specification requirements.

Rosemont Trip Units presently require calibration at least once per 31 days in Technical Specification Tables 4.3.1.1-1 and 4.3.3.1-1. Adding this 31 day calibration frequency to all Rosemont Trip Units associated with instrumentation in Table 4.3.2.1-1 promotes consistency between these tables.

The manufacturer of the temperature switches listed in Table 4.3.2.1-1 recommends a yearly calibration frequency. The proposed change to an annual calibration frequency promotes consistency between manufacturer's recommendations and Technical Specification requirements. The proposed annual calibration frequency is more conservative than the NSSS vendor's assumed setpoint methodology calibration frequency of at least once per 18 months.

These proposed changes increase the surveillance requirements for the Rosemont Trip Units and Riley Temperature Switches listed in Table 4.3.2.1-1. As such, this change does not involve the reduction of safety margins and no significant increase in the probability or consequences of an accident previously evaluated is involved nor is the possibility of a new or different kind of accident from any accident previously evaluated created. Thus the proposed change to the Technical Specification does not involve any significant hazards consideration.

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TABLE 4.3.2.1-1

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<b>1. REACTOR CONTAINMENT ISOLATION</b>				
a. Reactor Vessel Water Level - Low Low, Level 2	S	H	R <sup>(c)</sup>	1, 2, 3 and 7
b. Reactor Vessel Water Level - Low Low, Level 2 (ECCS - Division 3)	S	H	R <sup>(c)</sup>	1, 2, 3 and 7
c. Reactor Vessel Water Level - Low Low Low, Level 1 (ECCS - Division 1 and Division 2)	S	H	R <sup>(c)</sup>	1, 2, 3 and 7
d. Drywell Pressure - High	S	H	R <sup>(c)</sup>	1, 2, 3
e. Drywell Pressure-High (ECCS - Division 1 and Division 2)	S	H	R <sup>(c)</sup>	1, 2, 3
f. Drywell Pressure-High (ECCS - Division 3)	S	H	R <sup>(c)</sup>	1, 2, 3
g. Containment and Drywell Ventilation Exhaust Radiation - High High	S	H	R	1, 2, 3 and *
h. Manual Initiation	NA	H <sup>(a)</sup>	NA	1, 2, 3 and **
<b>2. MAIN STEAM LINE ISOLATION</b>				
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	H	R <sup>(c)</sup>	1, 2, 3
b. Main Steam Line Radiation - High	S	H	R	1, 2, 3
c. Main Steam Line Pressure - Low	S	H	R <sup>(c)</sup>	1
d. Main Steam Line Flow - High	S	H	R <sup>(c)</sup>	1, 2, 3
e. Condenser Vacuum - Low	S	H	R <sup>(c)</sup>	1, 2**, 3**

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE IS REQUIRED
<b>2. PRIMARY CONTAINMENT ISOLATION (Continued)</b>				
a. Fuel Steam Line Tunnel Temperature - High	S	H	-R-A	1, 2, 3
b. Fuel Steam Line Tunnel Δ Temp. - High	S	H	-R-A	1, 2, 3
c. Manual Inflation	NA	H(a)	NA	1, 2, 3
<b>3. SECONDARY CONTAINMENT ISOLATION</b>				
a. Reactor Vessel Water Level - Low Low, Level 2	S	H	R (c)	1, 2, 3 and θ
b. Drywell Pressure - High	S	H	R (c)	1, 2, 3
c. Fuel Handling Area Ventilation Exhaust Radiation - High High	S	H	R	1, 2, 3 and *
d. Fuel Handling Area Pool Sweep Exhaust Radiation - High High	S	H	R	1, 2, 3 and *
e. Manual Inflation	NA	H(a)	NA	1, 2, 3 and *
<b>4. REACTOR WATER CLEANUP SYSTEM ISOLATION</b>				
a. Flow - High	S	H	R	1, 2, 3
b. Flow Timer	NA	H	Q	1, 2, 3
c. Equipment Area Temperature - High	S	H	-R-A	1, 2, 3
d. Equipment Area Ventilation Δ Temp. - High	S	H	-R-A	1, 2, 3
e. Reactor Vessel Water Level - Low Low, Level 2	S	H	R (c)	1, 2, 3

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED
<b>4 REACTOR WATER CLEANUP SYSTEM ISOLATION (Continued)</b>				
4.1 Main Steam Line Tunnel Ambient Temperature - High	S	H	RA	1, 2, 3
4.2 Main Steam Line Tunnel Δ Temp. - High	S	H	RA	1, 2, 3
4.3 SICS Initiation	NA	H <sup>(b)</sup>	NA	1, 2, 3
4.4 Reactor Initiation	NA	H <sup>(a)</sup>	NA	1, 2, 3
<b>5 REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</b>				
5.1 RWC Steam Line Flow - High	S	H	R <sup>(c)</sup>	1, 2, 3
5.2 RWC Steam Supply Pressure - Low	S	H	R <sup>(c)</sup>	1, 2, 3
5.3 RWC Turbine Exhaust Diaphragm Pressure - High	S	H	R <sup>(c)</sup>	1, 2, 3
5.4 RWC Equipment Room Ambient Temperature - High	S	H	RA	1, 2, 3
5.5 RWC Equipment Room Δ Temp. - High	S	H	RA	1, 2, 3
5.6 Main Steam Line Tunnel Ambient Temperature - High	S	H	RA	1, 2, 3
5.7 Main Steam Line Tunnel Δ Temp. - High	S	H	RA	1, 2, 3

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION (Continued)</u>				
1. Main Steam Line Tunnel Temperature Timer	NA	H	Q	1, 2, 3
2. RHR Equipment Room Ambient Temperature - High	S	H	RA	1, 2, 3
3. RHR Equipment Room Δ Temp. - High	S	H	RA	1, 2, 3
4. RHR/PCIC Steam Line Flow - High	S	H	R <sup>(c)</sup>	1, 2, 3
5. Manual Initiation	NA	H <sup>(a)</sup>	NA	1, 2, 3
6. Reactor Pressure-High (SCCS Division 1 and Division 2)	S	H	R <sup>(c)</sup>	1, 2, 3
<u>5. RHR SYSTEM ISOLATION</u>				
1. RHR Equipment Room Ambient Temperature - High	S	H	RA	1, 2, 3
2. RHR Equipment Room Δ Temp. - High	S	H	RA	1, 2, 3
3. Reactor Vessel Water Level - Low, Level 3	S	H	R <sup>(c)</sup>	1, 2, 3
4. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	S	H	R <sup>(c)</sup>	1, 2, 3



TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
5. <u>REACTOR SYSTEM ISOLATION (Continued)</u>				
a) Reactor Pressure - High	S	H	R <sup>(c)</sup>	1, 2, 3
b) Manual Initiation	NA	H <sup>(a)</sup>	NA	1, 2, 3

(a) During handling irradiated fuel in the primary or secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.

(b) When reactor steam pressure  $\geq$  1045 psig and/or any turbine stop valve is open.

(c) During CORE ALTERATION and operations with a potential for draining the reactor vessel.

- a) Reactor initiation switches shall be tested at least once per 18 months during shutdown. All other switches associated with manual initiation shall receive a CHANNEL FUNCTIONAL TEST at least once per 31 days as part of circuitry required to be tested for automatic system isolation.
- b) Trip logic or logic channel shall be tested at least every other 31 days.
- c) Trip unit at least once per 31 days.



TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 038

Priority: 21A<sup>B</sup>

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.7.1-1.5

Problem Title: Carbon Bed Rad Mon Calibration Frequency

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

Tech Spec surveillance Table 4.3.7.1-1.5 list calibration frequency for the carbon Beck Vault Radiation Monitor as 18 months. The vendor recommends a calibration frequency of 12 months.

2. Safety Significance: None: Annual calibration of the mon is administratively controlled through surveillance program.

3. Anticipated Resolution: Tech Spec change to change calibration frequency.  
*Programatically increase surveillance frequency.*

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 041

Priority: 2A  
7-13

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.4.1.1

Problem Title: Recirc. Loop Operability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The surveillance requirements do not meet the requirements for the limiting condition for operation section 3.4.1.1. The surveillance check valve operation not loop operation.

2. Safety Significance: The proposed change establishes better surveillance requirements, therefore it does not involve any significant hazards or safety considerations.

3. Anticipated Resolution: Change 4.4.1.1 to check loop operation and add tech spec requirements to check flow control valves. Obtain GE evaluation to determine if item can be downgraded.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers

Identified By \_\_\_\_\_  
Date 1

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.4.1.1

Problem Title: Recirc Loop Operability

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other): THE SURVEILLANCE REQUIREMENTS DO NOT MEET THE REQUIREMENTS FOR THE LIMITING CONDITION FOR OPERATION SECTION 3.4.1.1. THE SURVEILLANCE CHECK VALVE OPERATION NOT LOOP OPERATION.

2. Safety Significance: THE PROPOSED CHANGE ESTABLISH BETTER SURVEILLANCE REQUIREMENTS, THEREFORE IT DOES NOT INVOLVE ANY SIGNIFICANT HAZARDS OR SAFETY CONSIDERATIONS.

3. Anticipated Resolution: CHANGE 4.4.1.1 TO CHECK LOOP OPERATION AND ADD TECH. SPEC. REQUIREMENTS TO CHECK FLOW CONTROL VALV.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_  
NRC Notified: \_\_\_\_\_  
Individual Notified \_\_\_\_\_ Date 1 Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_  
Date 1 Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers

6. (GCNS - 210)

SUBJECT: Technical Specification 3/4.1.1, page 3/4 4-1.

DISCUSSION: Technical Specification 3.4.1.1 specifies recirculation loop operability requirements for OPERATIONAL CONDITIONS 1 and 2. The corresponding surveillance requirement, 4.4.1.1, does not verify operation of the reactor coolant system recirculation loops, but instead specifies surveillance requirements for the reactor coolant system recirculation loop flow control valves. Recirculation loop operability should be verified at least once per 24 hours when demonstrating jet pump operability per Specification 4.4.1.2.a. Surveillance Requirement 4.4.1.1 for recirculation loop flow control valves along with associated new limiting conditions for operation should be placed in Specification 3/4.1, REACTIVITY CONTROL SYSTEMS, and a new bases section, 3/4.1.6, should be written to describe the recirculation loop flow control valves.

The new Limiting Condition for Operation (LCO), Applicability, and Action statements for the flow control valves are written to comply with the existing Specification 3.4.1.1, for the reactor coolant system recirculation loops. The flow control valves are specified to be operable in OPERATIONAL CONDITIONS 1 and 2 when their associated recirculation loops are required to be in operation. The surveillance requirements for the flow control valves have been changed to require a demonstration of operability prior to returning the valve(s) to service after maintenance, repair, or replacement work is performed on the valve or its associated actuator, hydraulic power unit or control circuit. The surveillance requirements for the flow control valves have also been changed to reflect that the rate of control valve movement is a "maximum" rate rather than the presently stated "average". The "maximum" rather than "average" control valve movement rate of 11% of stroke per second is specified in FSAR Section 15.3.

JUSTIFICATION: The present Surveillance Requirement 4.4.1.1 for recirculation loop flow control valves does not support the corresponding LCO 3.4.1.1, for recirculation loops in operation. The recirculation loops should be verified in operation during performance of jet pump operability surveillance performed at least once per 24 hours per Surveillance Requirement 4.4.1.2.a. The recirculation loop flow control valve position affects reactivity addition to the reactor core since recirculation pump speed is kept constant and loop flow is changed by position of the flow control valve. The increased surveillance requirements on the flow control valves help to ensure their operability.

SIGNIFICANT HAZARDS CONSIDERATION:

The proposed change is administrative in that it establishes better consistency between LCO's and surveillance requirements. Additionally it establishes more stringent requirements for operability of the recirculation flow control valves. Therefore, based on comparison with NRC example 3 (ii) (10 CFR 50, Interim Final Rule, Federal Register, April 6, 1983), amendments that are considered not likely to involve significant hazards considerations, the proposed change to the Technical Specifications does not involve any significant hazards considerations.

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 073

Priority: L1B

Identified By \_\_\_\_\_ / \_\_\_\_\_  
Date

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.7.9

Problem Title: Fire Detection Instrumentation Format Revision

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The Fire Detection Instrumentation Tech Spec has a confusing format which can lead to taking wrong actions. Also all the detections are not included. (ie. PGCC )

2. Safety Significance:

Interim controls may be necessary to ensure the appropriate action, are taken if detectors become inop.

3. Anticipated Resolution:

Final tech spec change proposal submitted on 9-9-83 (AECM-83/0565). Earlier proposal was denied by NRR since our submittal did not match the St. Louis Tech Spec format which the NRR likes.

*Could use Tech spec position statement*

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_ / \_\_\_\_\_  
Individual Notified Date Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_ / \_\_\_\_\_  
Date Time

cc: J. E. Cross  
R. F. Rogers



TRANSMITTAL OF PROPOSED CHANGES  
TO GRAND GULF TECHNICAL SPECIFICATIONS

1. (GGNS - 687) (Resubmittal of Item 1, AECM-83/0411)

SUBJECT: Technical Specification Table 3.3.7.9-1 and Technical Specification 3.3.7.9, pages 3/4 3-76 through 3/4 3-80.

DISCUSSION: Technical Specification Table 3.3.7.9-1 provides a listing of fire detection instrumentation. The present table is ordered by building and room numbers within a building whereas the Specification (3.3.7.9) governing the table is ordered by zones. One of the changes to Table 3.3.7.9-1 is to order the table by building, zone numbers within the building, and room numbers within the zone. The present listing of fire detection instruments does not distinguish them as either early warning or actuation of fire suppression systems devices. The proposed change adds Function A (early warning and notification) and Function B (actuation of fire suppression systems) to the table and to the ACTION statements of Technical Specification 3.3.7.9. Other changes to the Fire Detection Instrumentation Table 3.3.7.9-1 include the following:

1. Design Changes
2. Additions to the Table
3. Corrections to the Table

The changes to the table are discussed in detail in the justification section below.

JUSTIFICATION: The following changes to Table 3.3.7.9-1 are the result of design changes made to the fire detection system:

1. In Zone 1-4 for the Control Building, Area OC201 (stairwell) including one smoke detector was added to the table. Safety related cables pass through this area and appropriate fire detection was added to provide coverage.
2. In Zone 1-6 for the Control Building, OC216 (West Corridor) with two smoke detectors was added. Safety related cables pass through this area and appropriate fire detection was added.
3. In Zone 1-23 for the Control Building one smoke detector was added to provide additional coverage.
4. In Zone 2-9 for the Auxiliary Building, one smoke detector was added to provide additional coverage.

The following changes to Table 3.3.7.9-1 constitute additions of areas/zones and their associated instrumentation which were previously not part of the original instrument.

1. Add Zone 1-23 at Elevation 99' in the Control Building

(Corridor). These areas have safety related cable and should be included in the table. A total of 12 smoke detectors is added with these areas.

2. Add area OC306 (Electrical Chase) in Zone 1-10 at Elevation 133' in the Control Building. Safety related cable passes through these areas.
3. Add area OC410 (Battery Room) in Zone 1-14 at Elevation 148' in the Control Building. Specification 3.3.7.9 covers all instrumentation in each Zone listed in Table 3.3.7.9-1 and this area is added to complete the listing for Zone 1-10. The number of Smoke Detectors for Zone 1-10 increased from 7 to 9 as a result of this change.
4. Add Zone 1-12 at Elevation 133' in the Control Building. This zone includes areas OC304 (Electrical Space) and OC305 (Electrical Space). Two additional smoke detectors are added with this change. Safety related cable passes through this zone.
5. Zone 1-13 at Elevation 133' in the Control Building is added since part of Unit 1 control room HVAC equipment is in this zone. Zone 1-13 includes area OC303 (HVAC Room) and adds 16 smoke detectors.
6. Areas OC401 (Corridor), OC408 (Corridor), and OC409 (Electrical Chase) are added to Zone 1-15 along with four additional smoke detectors. Specification 3.3.7.9 covers all instrumentation in each Zone listed in Table 3.3.7.9-1 and these areas (OC401, 408, 409) were added to complete the listing for Zone 1-15.
7. Zone 1-19 at Elevation 166' in the Control Building is added because safety related cable passes through this area. This zone includes area OC514 (Locker Room) and adds 9 smoke detectors.
8. Zone 1-21 at Elevation 166' in the Control Building is added because safety related cable passes through this area. This zone includes area OC518 (Electrical Chase) and adds two smoke detectors.
9. Zone 1-22 at Elevation 177' in the Control Building is added because safety related cable passes through this area. This zone includes areas OC601 (Viewing Gallery), OC603 (Emergency Dormitory), and OC608 (Technical Support) and also adds sixteen smoke detectors.

Areas OC708 (West Corridor), OC709 (Electrical Chase) and OC712 (HVAC Room) are added to Zone 1-23 at Elevation 169' in the Control Building. Specification 3.3.7.9 covers all instrumentation in each Zone listed in Table 3.3.7.9-1 and these areas are added to complete the listing for Zone

1-23. The number of smoke detectors in Zone 1-23 is increased from 15 to 21 with one being added as a design change and five added with areas OC706, OC709, and OC712.

11. Add areas 1A128 (RHR "A" Heat Ex Room), 1A129 (RHR "B" Heat Ex Room), and 1A223 (Passage) to Zone 2-4 in the Auxiliary Building. Areas 1A128 and 1A129 are separated by grating from 1A102 and 1A106, respectively, and as such smoke detectors in area 1A102 serve 1A128 and in 1A106 serve 1A129. Area 1A223 is an area already served by Zone 2-4 instrumentation and is included to complete the Zone 2-4 listing.
12. Add areas 1A524 (Platform) and 1A529 (IPC and CU Room) to Zone 2-9. These areas are added to complete the listing for Zone 2-9.
13. Add area 1A101 (Passage) to Zone 2-17. This area is currently served by Zone 2-17 instrumentation and should be included.
14. In Zone 2-10 for the Diesel Generator Building, add three smoke detectors due to the addition of the corridor between the Auxiliary Building and Diesel Generator Building.
15. The type of fire protection initiated has been added to the Heat detector column. This administrative change indicates that Halon, CO<sub>2</sub> or Deluge is actuated by the heat detector.
16. The Control Room HVAC Intake Plenum Mounted Detectors have been added since they involve control room habitability.
17. PGCC Halon systems in the Control Building have been added as Section "g" of the Table. The list of Halon systems is broken down by room, elevation and also by Halon panels within the room. The panels are listed as underfloor modules/Halon panel (Example: 1H13-U713/1H13-P913) and the number of detectors associated with each Halon panel is shown.
18. The function designation of A or B (X or Y) is added to the Table to distinguish between those instruments that perform early warning fire detection and notification and those that also actuate fire suppression systems as well as give early warning and notification. The Function A and Function B designation is also added to the ACTION statements of Technical Specifications 11.1.9 to provide consistency between the Technical Specification and its associated Table. The design of the Function A and B designations follows the Standard Technical Specification for Fire Detection Instrumentation design the standard Technical

Specification ACTION statements do not apply. With the addition of function A or B designation, present footnote (2) on page 3/4 3-77 is not needed and is deleted. Present footnote (2) is a duplication of the function A or B requirements.

The following changes to Table 3.3.7.9-1 constitutes correction of errors in the original Technical Specification:

1. Area OC308 (Corridor) at Elevation 133' in the Control Building is moved from Zone 1-10 to Zone 1-11.
2. The number of heat detectors in OC403 - Computer Room of the Control Building is corrected from present 13 to 12. This change does not reflect the deletion of a heat detector from the plant but only a correction to the table.
3. The number of Smoke Detectors in OC503 (Control Room) at Elevation 166' of the Control Building is changed from 17 to 16. This change reflects the temporary split of Unit 1 and Unit 2 Control Rooms. The detector deleted is on the Unit 2 side of the Control Room.
4. Area 1A211 [North Corridor (Partial)] is added to Zone 2-2 of the Auxiliary Building. This area overlaps into Zone 2-2 and also appears in Zone 2-18 as an overlap or interface area.
5. Area 1A314 [South Corridor (Partial)] is added to Zone 2-6 of the Auxiliary Building. This area overlaps into Zone 2-6 and also appears in Zone 2-19 as an overlap or interface area.
6. Area 1A424 [Set Down Area (Partial)] is added to Zone 2-8. This area overlaps into Zone 2-8 and also appears in Zone 2-7 as an overlap or interface area. The number of smoke detectors in Zone 2-7 goes from 12 to 11 and the number in Zone 2-8 goes from 24 to 25 due to Zone assignment of instrumentation.
7. Areas 1A122 [South Corridor (Partial)] and 1A123 [North Corridor (Partial)] are added to Zone 2-14 of the Auxiliary Building. These areas overlap into Zone 2-14 and also appear in Zone 2-17 as an overlap or interface area.
8. Zones 6-9A, 6-9B, and 6-9C for the Diesel Generator Building are corrected to 2-10, 2-11, and 2-12 respectively. Added Corridor between Diesel Generator and Auxiliary Building.
9. Added area number for standby service under Emergency Release.



The format change is proposed so that the Technical Specification will more accurately reflect that the operability of each individual smoke detector affects the entire zone, not just the area in which it is installed.

**SIGNIFICANT HAZARDS CONSIDERATION:**

The changes to the Fire Detection Instrumentation Table 3.3.7.9-1 constitute additions, corrections, and changes due to design changes to the plant. The design changes add additional equipment and enhance fire detection capability. The additions to the table also enhance fire detection capability. The corrections to the table do not decrease fire detection capability but reflect actual plant systems and instrumentation arrangements. This change does not involve a reduction of safety margins and no significant increase in the probability or consequences of an accident previously evaluated is involved nor is the possibility of a new or different kind of accident from any accident previously evaluated created. Thus the proposed change to the Technical Specifications does not involve any significant hazards considerations.

INSTRUMENTATIONFIRE DETECTION INSTRUMENTATIONLIMITING CONDITION FOR OPERATION

3.3.7.9 As a minimum, the fire detection instrumentation for each fire detection zone shown in Table 3.3.7.9-1 shall be OPERABLE.

APPLICABILITY: Whenever equipment protected by the fire detection instrument is required to be OPERABLE.

ACTION:

*Function A. or Function B.*

With the number of OPERABLE fire detection instruments less than the Minimum Instruments OPERABLE requirement of Table 3.3.7.9-1:

a. Within 1 hour, establish a fire watch patrol to inspect the zone(s) with the inoperable instrument(s) at least once per hour, unless the instrument(s) is located inside the containment, ~~or drywell~~, then inspect the primary containment at least once per 8 hours or monitor the containment, ~~and/or drywell~~ air temperature at least once per hour at the locations listed in Specification 4.6.1.8 and 4.6.2.6.

*or steam tunnel*

*or steam tunnel*

b. Restore the minimum number of instruments to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the instrument(s) to OPERABLE status.

c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.7.9.1 Each of the above required fire detection instruments which are accessible during unit operation shall be demonstrated OPERABLE at least once per 6 months by performance of a CHANNEL FUNCTIONAL TEST. Fire detectors which are not accessible during unit operation shall be demonstrated OPERABLE by the performance of a CHANNEL FUNCTIONAL TEST during each COLD SHUTDOWN exceeding 24 hours unless performed in the previous 6 months.

4.3.7.9.2 The NPPA Standard 720 supervised circuits supervision associated with the detector alarms of each of the above required fire detection instruments shall be demonstrated OPERABLE at least once per 6 months.

Rec'd  
9-1-78  
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TABLE 3.3.7.9-1

FIRE DETECTION INSTRUMENTATION

INSTRUMENT LOCATION			MINIMUM INSTRUMENTS OPERABLE*			
			ZONE <sup>(1)</sup>	HEAT <sup>(2)</sup>	FLAME	SMOKE <sup>(3)</sup>
a. Containment Building						
1. Return Duct Mounted Detectors			NA	NA	NA	3
b. Control Building						
ROOM NO.	ELEV.	ROOM NAME				
1. OC202	111'	DIV I SWGR RM	1-4	6	NA	4
2. OC207	111'	DIV I BATTERY RM	1-4	NA	NA	1
3. OC208	111'	DIV II REMOTE SHUTDOWN PANEL ROOM	1-27	1	NA	1
4. OC208A	111'	DIV I REMOTE SHUTDOWN PANEL ROOM	1-27	1	NA	1
5. OC209	111'	DIV III BATTERY RM	1-5	NA	NA	1
6. OC210	111'	DIV III SWGR RM	1-5	4	NA	2
7. OC211	111'	DIV II BATTERY RM	1-6	NA	NA	1
8. OC215	111'	DIV II SWGR RM	1-6	7	NA	4
9. OC307	133'	ELECTRICAL CHASE	1-10	NA	NA	1
10. OC308	133'	ELECTRICAL CHASE	1-10	NA	NA	1
11. OC302	133'	HVAL EQUIP. ROOM	1-11	NA	NA	13
12. OC402	148'	CABLE SPREADING RM	1-15	7	NA	10
13. OC403	148'	COMPUTER ROOM	1-14	13	NA	7
14. OC407	148'	INSTR. MOTOR GEN ROOM	1-15	2	NA	1
15. OC503						
OC504	166'	CONTROL ROOM	1-18	NA	NA	17
16. OC702	189'	CABLE SPREADING RM	1-23	12	NA	14
17. OC703	189'	CONTROL CAB. ROOM	1-24	4	NA	6
18. OC707	189'	INSTR MOTOR GEN. RM	1-23	NA	NA	1

The fire detection instruments located within the primary containment are not required to be OPERABLE during the performance of Type A Containment Leakage tests.

- (1) Zones apply only to smoke detectors.
- (2) Heat detectors provide warning and activation of automatic extinguishing systems.

(3) Fast thermopiles which monitor ambient air temperature will provide early warning capability.

TABLE 3.3.7.9-1 (Continued)  
FIRE DETECTION INSTRUMENTATION

TRUANT LOCATION			MINIMUM INSTRUMENTS OPERABLE*			
ROOM NO.	ELEV.	ROOM NAME	ZONE (1)	HEAT (2)	FLAME	SMOKE (3)
<b>Auxiliary Building</b>						
1. 1A102	93'	RHR 'A' HT EX RM	2-4	NA	NA	1
2. 1A103	93'	RHR 'A' PUMP RM	2-4	NA	NA	2
3. 1A104	93'	RCIC PUMP RM	2-4	NA	NA	2
4. 1A105	93'	RHR 'B' PUMP RM	2-4	NA	NA	2
5. 1A106	93'	RHR 'B' HT EX RM	2-4	NA	NA	1
6. 1A109	93'	HPCS PUMP RM	2-17	NA	NA	2
7. 1A111	93'	PIPING PENETRATION RM	2-17	NA	NA	1
8. 1A114	93'	FAH COIL AREA	2-14	NA	NA	4
9. 1A115	93'	PIPING PENETRATION RM	2-14	NA	NA	1
10. 1A116	93'	PIPING PENETRATION RM	2-14	NA	NA	1
11. 1A117	93'	MISC. EQUIP AREA	2-14	NA	NA	4
12. 1A118	93'	RHR 'C' PUMP ROOM	2-14	NA	NA	2
13. 1A119	93'	LPCS PUMP ROOM	2-14	NA	NA	2
14. 1A120	93'	CCW PUMP AND HX AREA	2-14	NA	NA	3
15. 1A121	103'	EAST CORRIDOR	2-17	NA	NA	5
16. 1A122	103'	SOUTH CORRIDOR	2-17	NA	NA	3
17. 1A123	103'	NORTH CORRIDOR	2-17	NA	NA	5
18. 1A201	119'	EAST CORRIDOR	2-18	NA	NA	6
19. 1A202	119'	RHR 'A' HX RM	2-4	NA	NA	1
20. 1A203	119'	PIPING PENETRATION RM	2-4	NA	NA	2
21. 1A204	119'	PIPING PENETRATION RM	2-4	NA	NA	2
22. 1A205	119'	PIPING PENETRATION RM	2-4	NA	NA	2
23. 1A206	119'	RHR 'B' HX RM	2-4	NA	NA	1
24. 1A207	119'	ELECT. SWGR ROOM	2-4	3	NA	2
25. 1A208	119'	ELECT. SWGR ROOM	2-4	3	NA	2
26. 1A209	115'	RWCU RECIRC PUMP 'A' RM	2-4	NA	NA	1
27. 1A210	115'	RWCU RECIRC PUMP 'B' RM	2-4	NA	NA	1
28. 1A211	115'	NORTH CORRIDOR	2-17	NA	NA	5
29. 1A212	115'	SOUTH CORRIDOR	2-17	NA	NA	3
30. 1A213	119'	ELECT. SWGR RM	2-4	3	NA	2

TABLE 3.3.7.9-1 (continued)

FIRE DETECTION INSTRUMENTATION

INSTRUMENT LOCATION			MINIMUM INSTRUMENTS OPERABLE*				
ROOM NO.	ELEV.	ROOM NAME	ZONE <sup>(1)</sup>	HEAT <sup>(2)</sup>	FLAME	SMOKE <sup>(3)</sup>	
c. Auxiliary Building (Continued)							
31.	1A220	119'	PIPING PENETRATION RM	2-3	NA	NA	1
32.	1A221	119'	ELECT. SWGR RM	2-3	2	NA	2
33.	1A222	119'	WEST CORRIDOR	2-2	NA	NA	18
34.	1A301	139'	NORTHEAST CORRIDOR	2-5	NA	NA	2
35.	1A302	139'	SOUTHEAST CORRIDOR	2-5	NA	NA	1
36.	1A303	139'	RHR 'A' HX RM	2-5	NA	NA	1
37.	1A304	139'	PIPING PENETRATION RM	2-5	NA	NA	1
38.	1A305	139'	STEAM TUNNEL	2-20	NA <sup>(4)</sup>	NA	2 NA
39.	1A306	139'	PIPING PENETRATION RM	2-5	NA	NA	1
40.	1A307	139'	RHR 'B' HX RM	2-5	NA	NA	1
41.	1A308	139'	ELECT. PENETRATION RM	2-5	3	NA	2
42.	1A309	139'	ELECT. PENETRATION RM	2-5	3	NA	2
				2-5			3
43.	1A314	139'	SOUTH CORRIDOR	2-19	NA	NA	3
44.	1A315	139'	NORTH CORRIDOR	2-5	NA	NA	13
45.	1A318	139'	ELECT. PENETRATION RM	2-5	2	NA	2
46.	1A319	139'	R/V INSTR. TEST RM	2-5	NA	NA	1
47.	1A320	139'	ELECT. PENETRATION RM	2-5	2	NA	2
48.	1A321	139'	MCC AREA	2-19	NA	NA	3
49.	1A322	139'	CENTRIFUGAL CHILLER AREA	2-19	NA	NA	4
50.	1A323	139'	SGTS AREA	2-19	NA	NA	1
51.	1A324	139'	HVAC EQUIP AREA	2-19	NA	NA	1
52.	1A326	139'	SGTS AREA	2-19	NA	NA	1
53.	1A401	166'	NORTHEAST CORRIDOR	2-8	NA	NA	2
54.	1A402	166'	STEAM TUNNEL ROOF	2-5	NA	NA	1
55.	1A403	166'	SOUTHEAST CORRIDOR	2-8	NA	NA	2
56.	1A404	166'	UNASSIGNED AREA	2-8	NA	NA	1
57.	1A405	166'	CENTRAL EQUIP R.	2-8	NA	NA	1
58.	1A406	166'	CENTRAL FILTER	2-8	NA	NA	1

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1. (GGNS-887)

TABLE 3.3.7.9-1 (Continued)

FIRE DETECTION INSTRUMENTATION

INSTRUMENT LOCATION				MINIMUM INSTRUMENTS OPERABLE*			
ROOM NO.	ELEV.	ROOM NAME	ZONE <sup>(1)</sup>	HEAT <sup>(2)</sup>	FLAME	SMOKE <sup>(3)</sup>	
Auxiliary Building (Continued)							
59.	1A407	165'	MCC AREA	2-8	2	NA	1
60.	1A410	166'	MCC AREA	2-8	2	NA	1
61.	1A417	166'	NORTH CORRIDOR	2-8	NA	NA	14
62.	1A420	166'	SOUTH CORRIDOR	2-7	NA	NA	4
63.	1A424	166'	SET DOWN AREA	2-7	NA	NA	2
64.	1A428	166'	WEST CORRIDOR	2-7	NA	NA	4
65.	1A432	166'	FPC AND CU PUMP RM	2-7	NA	NA	1
66.	1A434	166'	PASSAGE	2-7	NA	NA	1
67.	1A519	185'	STORAGE AREA	2-9	NA	NA	4
68.	1A527	185'	LOAD CENTER AREA	2-9	NA	NA	5
69.	1A539	185'	CABLE CHASE	2-15	NA	NA	1
70.	1A502	203'10"	STORAGE AREA	2-13	NA	NA	6
71.	1A503	208'10"	PASSAGE	2-13	NA	NA	3
72.	1A504	203'10"	FUEL HANDLING AREA	2-13	NA	NA	13
73.	1A506	245'	HVAC EQUIP AREA	2-13	NA	NA	9
Diesel Generator Building							
1.	Unit 1 El. 158'-0"	HPCS Generator	6-9A	7	6	NA	
2.	Unit 1 El. 158'-0"	Bus B Generator	6-9B	7	6	NA	
3.	Unit 1 El. 158'-0"	Bus A Generator	6-9C	7	6	NA	
Standby Service Water Pump House							
1.	Pump house A		2-1	NA	NA	1	
2.	Valve Room A		2-1	NA	NA	1	
3.	Pump House B		2-1	NA	NA	1	
4.	Valve Room B		2-1	NA	NA	1	
Charcoal Filter Trains							
1.	Standby Gas Treatment System Filter Train	Auxiliary Building	NA	1	NA	NA	
(Allison Thermostat Vitr)							
2.	Standby Gas Treatment System Filter Train	Control Building	NA	1	NA	NA	
3.	Standby Gas Treatment System Filter Train	Control Building	NA	1	NA	NA	
4.	Standby Gas Treatment System Filter Train	Control Building	NA	1	NA	NA	

TABLE 3.3.7.9-1  
FIRE DETECTION INSTRUMENTATION

<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERABLE*</u>		
	<u>HEAT</u> (X/Y)	<u>FLAME(1)</u> (X/Y)	<u>SMOKE (1)</u> (X/Y)
<b>a. <u>CONTAINMENT BUILDING</u> ‡</b>			
1. Return Duct Mounted Detectors			3/0
<u>ROOM</u>	<u>ELEV</u>	<u>ROOM NAME</u>	
<b>b. <u>CONTROL BUILDING</u></b>			
1. Zone 1-3			12/0
OC109	93'	Decontamination Area	
OC115	93'	Corridor	
OC116	93'	Hot Machine Shop	
OC117	93'	Corridor	
2. Zone 1-4			6/0
OC201	111'	Stairwell	
OC202	111'	Div I Swgr Rm	0/6(CO <sub>2</sub> )
OC207	111'	Div I Battery Rm	

\* (X/Y): X - is number of Function A (early warning fire detection and notification only) instruments.  
Y - is number of Function B (actuation of fire suppression systems and early warning and notification) instruments.

‡ The fire detection instruments located within the primary containment are not required to be OPERABLE during the performance of Type A Containment Leakage Rate Tests.

(1) Smoke and flame detectors provide only early warning capability with the exception of:

- (a) Zone 1-27 detectors trip closed the door between the OC206/OC208A Remote Shutdown panel rooms.
- (b) Containment building return duct mounted detectors' trip the containment cooler fans.
- (c) Zone 1-11 and 1-13 detectors initiate the control building purge fan system.
- (d) Control Room HVAC Intake Plenum Detectors trip the control room A/C units unless a control room emergency filtration system filtration mode automatic activation signal is present.



TABLE 3.3.7.9-1  
FIRE DETECTION INSTRUMENTATION

ROOM	ELEV	ROOM NAME	MINIMUM INSTRUMENTS OPERABLE*		
			HEAT (X/Y)	FLAME <sup>(1)</sup> (X/Y)	SMOKE <sup>(1)</sup> (X/Y)
3. Zone 1-5					3/0
OC209	111'	Div III Battery Rm			
OC210	111'	Div III Swgr Rm	0/4(CO <sub>2</sub> )		
4. Zone 1-6					7/0
OC211	111'	Div II Battery Rm			
OC215	111'	Div II Swgr Rm	0/7(CO <sub>2</sub> )		
OC216	111'	West Corridor			
5. Zone 1-10					2/0
OC306	133'	Electrical Chase			
OC307	133'	Electrical Chase			
6. Zone 1-11					13/0
OC302	133'	HVAC Equipment Rm			
OC308	133'	Corridor			
7. Zone 1-12					2/0
OC304	133'	Electrical Space			
OC305	133'	Electrical Space			
8. Zone 1-13					16/0
OC303	133'	HVAC Room			
9. Zone 1-14					9/0
OC403	148'	Computer Room	0/12(Halon)		
OC410	148'	Battery Room			
10. Zone 1-15					15/0
OC401	148'	Corridor			
OC402	148'	Lower Cable Spreading Room	0/7(CO <sub>2</sub> )		
OC407	148'	Instr. Motor Gen Rm	0/2(CO <sub>2</sub> )		
OC408	148'	Corridor			
OC409	148'	Electrical Chase			
11. Zone 1-16					16/0
OC500	166'	U-1 Inst Rack Area			



TABLE 3.3.7.9-1  
FIRE DETECTION INSTRUMENTATION

ROOM	ELEV	ROOM NAME	MINIMUM INSTRUMENTS OPERABLE*		
			HEAT (X/Y)	FLAME <sup>(1)</sup> (X/Y)	SMOKE <sup>(1)</sup> (X/Y)
12. Zone 1-19					9/0
OC514	166'	Locker Room			
13. Zone 1-21					2/0
OC518	166'	Electrical Chase			
14. Zone 1-22					16/0
OC601	177'	Viewing Gallery			
OC603	177'	Emergency Dormitory			
OC608	177'	Technical Support			
15. Zone 1-23					21/0
OC702	189'	Upper Cable Spreading Room	0/12(CO <sub>2</sub> )		
OC706	189'	West Corridor			
OC707	189'	Instr. Motor Gen Rm			
OC709	189'	Electrical Chase			
OC712	189'	EVAC Room			
16. Zone 1-24					6/0
OC703	189'	Control Cabinet Area	4/0(CO <sub>2</sub> )		
17. Zone 1-27					2/0
OC208	111'	Div I Remote Shutdown Panel	0/1(CO <sub>2</sub> )		
OC208A	111'	Div II Remote Shutdown Panel	0/1(CO <sub>2</sub> )		
18. Control Room EVAC Intake Plenum Mounted Detectors					2/0
<b>c. <u>AUXILIARY BUILDING</u></b>					
1. Zone 2-2					23/0
1A211	119'	North Corridor (Partial)			
1A215	119'	South Corridor			
1A222	119'	West Corridor			

TABLE 3.3.7.9-1  
FIRE DETECTION INSTRUMENTATION

ROOM	ELEV	ROOM NAME	MINIMUM INSTRUMENTS OPERABLE*		
			HEAT (X/Y)	FLAME <sup>(1)</sup> (X/Y)	SMOKE <sup>(1)</sup> (X/Y)
2. Zone 2-3					5/0
1A219	119'	Electrical Swgr Rm	0/2(CO <sub>2</sub> )		
1A220	119'	Piping Penetration Rm			
1A221	119'	Electrical Swgr Rm	0/2(CO <sub>2</sub> )		
3. Zone 2-4					22/0
1A102	93'	RHR "A" Heat Ex Rm			
1A103	93'	RHR "A" Pump Rm			
1A104	93'	RCIC Pump Rm			
1A105	93'	RHR "B" Pump Rm			
1A106	93'	RHR "B" Heat Ex Rm			
1A128	108'	RHR "A" Heat Ex Rm			
1A129	108'	RHR "B" Heat Ex Rm			
1A202	119'	RHR "A" Heat Ex Rm			
1A203	119'	Piping Penetration Rm			
1A204	119'	Piping Penetration Rm			
1A205	119'	Piping Penetration Rm			
1A206	119'	RHR "B" Heat Ex Rm			
1A207	119'	Electrical Swgr Rm	0/3(CO <sub>2</sub> )		
1A208	119'	Electrical Swgr Rm	0/3(CO <sub>2</sub> )		
1A209	115'	RWCU Recirc Pump "A" Rm			
1A210	115'	RWCU Recirc Pump "B" Rm			
1A223	128'	Passage			
4. Zone 2-5					5/0
1A318	139'	Electrical Penetration Room	0/2(CO <sub>2</sub> )		
1A319	139'	RPV Instr Test Rm			
1A320	139'	Electrical Penetration Room	0/2(CO <sub>2</sub> )		
5. Zone 2-6					26/0
1A301	139'	East Corridor			
1A302	139'	Southeast Corridor			
1A303	139'	RHR "A" Heat Ex Rm			
1A304	139'	Piping Penetration Rm			
1A304	139'	Piping Penetration Rm			
1A307	139'	RHR "B" Heat Ex Rm			
1A308	139'	Electrical Penetration Room	0/3(CO <sub>2</sub> )		
1A309	139'	Electrical Penetration Room			
1A310	139'	Electrical Penetration Room			
1A311	139'	Electrical Penetration Room			
1A312	139'	Electrical Penetration Room			
1A313	139'	Electrical Penetration Room			
1A314	139'	Electrical Penetration Room			
1A315	139'	Electrical Penetration Room			
1A316	139'	North Corridor			

TABLE 3.3.7.9-1  
FIRE DETECTION INSTRUMENTATION

ROOM	ELEV	ROOM NAME	MINIMUM INSTRUMENTS OPERABLE*		
			HEAT (X/Y)	FLAME <sup>(1)</sup> (X/Y)	SMOKE <sup>(1)</sup> (X/Y)
6. Zone 2-7					11/0
1A420	166'	South Corridor (Partial)			
1A424	166'	Set Down Area (Partial)			
1A428	166'	West Corridor			
1A432	166'	FPC & CU Pump Rm			
1A434	166'	South Passage			
7. Zone 2-8					25/0
1A401	166'	Northeast Corridor			
1A402	166'	Steam Tunnel Roof			
1A403	166'	Southeast Corridor			
1A404	166'	Unassigned Area			
1A405	166'	Containment Vent. Equip Room			
1A406	166'	Containment Exhaust Filter Rm			
1A407	166'	MCC Area	0/2(CO <sub>2</sub> )		
1A410	166'	MCC Area	0/2(CO <sub>2</sub> )		
1A417	166'	North Corridor (Partial)			
1A424	166'	Set Down Area (Partial)			
8. Zone 2-9					10/0
1A519	185'	Storage Area			
1A524	195'	Platform			
1A527	185'	Load Center Area			
1A529	185'	FPC & CU Rm			
9. Zone 2-13					31/0
1A602	208'	Storage Area			
1A603	208'	Passage			
1A604	208'	Fuel Handling Area			
1A606	243'	HVAC Equip Area			
10. Zone 2-14					17/0
1A114	83'	Fan Coil Area			
1A111	81'	Piping Penetration Rm			
1A112	81'	Piping Penetration Rm			
1A113	81'	North Corridor Area			
1A114	81'	FPC & CU Pump Rm			

TABLE 3.3.7.9-1  
FIRE DETECTION INSTRUMENTATION

<u>ROOM</u>	<u>ELEV</u>	<u>ROOM NAME</u>	<u>MINIMUM INSTRUMENTS OPERABLE*</u>		
			<u>HEAT</u> (X/Y)	<u>FLAME</u> <sup>(1)</sup> (X/Y)	<u>SMOKE</u> <sup>(1)</sup> (X/Y)
1A119	93'	LPCS Pump Room			
1A120	93'	CCW Pump & Heat Ex Rm			
1A122	103'	South Corridor (Partial)			
1A123	103'	North Corridor (Partial)			
11. Zone 2-15					1/0
1A539	185'	Cable Chase			
12. Zone 2-17					16/0
1A101	93'	Passage			
1A109	93'	HPCS Pump Rm			
1A111	93'	Piping Penetration Rm			
1A121	103'	East Corridor			
1A122	103'	South Corridor (Partial)			
1A123	103'	North Corridor (Partial)			
13. Zone 2-18					20/0
1A201	119'	East Corridor			
1A211	119'	North Corridor (Partial)			
14. Zone 2-19					13/0
1A314	139'	South Corridor (Partial)			
1A321	139'	MCC Area			
1A322	139'	Centrifugal Chiller Area			
1A323	139'	SGTS Area			
1A324	139'	HVAC Equip Area			
1A326	139'	SGTS Area			
15. Zone 2-20					2/0
1A305	139'	Steam Tunnel			
d. <u>DIESEL GENERATOR BUILDING</u>					
1. Zone 2-10					6/0 3/0
1B301	133'	Corridor			
1B302	133'	Div III Diesel Gen Room			
1B303	133'	Div III Diesel Gen Room			

TABLE 3.3.7.9-1  
FIRE DETECTION INSTRUMENTATION

ROOM	ELEV	ROOM NAME	MINIMUM INSTRUMENTS OPERABLE*		
			HEAT (X/Y)	FLAME <sup>(1)</sup> (X/Y)	SMOKE <sup>(1)</sup> (X/Y)
2. Zone 2-11				6/0	
1D308	133'	Div II Diesel Gen Room			
1D402	158'	Div II Diesel Gen Room	0/7 (Deluge)		
3. Zone 2-12				6/0	
1D310	133'	Div I Diesel Gen Room			
1D403	158'	Div I Diesel Gen Room	0/7 (Deluge)		
e. <u>STANDBY SERVICE WATER PUMP HOUSE</u>					
1. Zone 2-1					4/0
1M110	133'	SSW Pump Rm A			
1M112	133'	SSW Valve Rm A			
2M110	133'	SSW Pump Rm B			
2M112	133'	SSW Valve Rm B			
f. <u>CHARCOAL FILTER TRAINS</u>					
1. Standby Gas Treatment System Filter Train				1/0 (Allison Thermistor Wire)	
Auxiliary Building El. 139'					
2. Control Room Standby Fresh Air System Filter Train				1/0 (Allison Thermistor Wire)	
Control Building El. 133'					
g. <u>CONTROL BUILDING (PGCC HALON SYSTEMS)</u>					
OC503	166'	Control Room (Unit 1 side)			
Module/Halon Panel					
		1H13-U700/1H13-P900	0/10		10/0
		1H13-U701/1H13-P901	0/10		15/0
		1H13-U702/1H13-P902	0/9		14/0
		1H13-U703/1H13-P903	0/11		17/0
		1H13-U704/1H13-P904	0/7		13/0
		SH13-U700/1H13-P900	0/11		12/0
		SH13-U701/1H13-P901	0/11		12/0
		SH13-U702/1H13-P902	0/11		12/0

1. (GUNS-687)

TABLE 3.3.7.9-1  
FIRE DETECTION INSTRUMENTATION

<u>ROOM</u>	<u>ELEV</u>	<u>ROOM NAME</u>	<u>MINIMUM INSTRUMENTS OPERABLE*</u>		
			<u>HEAT</u> (X/Y)	<u>FLAME</u> <sup>(1)</sup> (X/Y)	<u>SMOKE</u> <sup>(1)</sup> (X/Y)
OC504	166'	Unit 1 Instrument Rack Area			
		Module/Halon Panel			
		1H13-U710/1H13-P910	0/8		15/0
		1H13-U711/1H13-P911	0/8		14/0
		1H13-U712/1H13-P912	0/8		9/0
		1H13-U714/1H13-P914	0/8		13/0
		1H13-U732/1H13-P932	0/8		14/0
		1H13-U733/1H13-P933	0/8		13/0
		1H13-U734/1H13-P934	0/8		13/0
		1H13-U735/1H13-P935	0/8		11/0
OC703	189'	Unit 1 Instrument Rack Area			
		Module/Halon Panel			
		1H13-U713/1H13-P913	0/9		15/0
		1H13-U715/1H13-P915	0/8		10/0
		1H13-U717/1H13-P917	0/8		15/0
		1H13-U736/1H13-P936	0/8		14/0
		1H13-U737/1H13-P937	0/8		10/0



Identified By \_\_\_\_\_

Date 1

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3/4.3.7.9Problem Title: FIRE Detection Instrumentation - Format Revision

## 1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other): \_\_\_\_\_

The Fire Detection Instrumentation Tech Spec has a confusing format which can lead to taking wrong actions. Also all the detectors are not included (eg. PCCC Water)

2. Safety Significance: Interim controls may be necessary to ensure the appropriate actions are taken if detectors become inop.3. Anticipated Resolution: Final Tech spec change proposal submitted on 9-9-83 (NRCM-83/0567). Earlier proposal was denied by NRC since our submittal did not match the Structure Tech Spec format which the NRC likes.4. NRC Response to Item (NRR/IE): NRC gave NRR/IE an interim Tech Spec (with original format) amendment 2 on the basis that they wanted time to review our submittal.NRC Notified: Denton (NRCM-83/0567, item 1)

Individual Notified

9-9-83

Date

1

Time

## 5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date

Time

cc: J. E. Cross  
R. F. Rogers



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NOV 7 1983

ENCLOSURE 2

MEMORANDUM FOR:

Standardization and Special Projects Branch  
Division of Licensing

FROM:

Victor Benaroya, Chief  
Chemical Engineering Branch  
Division of Engineering

SUBJECT:

GRAND GULF NUCLEAR STATION UNIT 1 - TECHNICAL SPECIFICATIONS  
(TAC #51479)

Plant Name: Grand Gulf Nuclear Station Unit 1

Docket No.: 50-416

Licensing Stage: OL

Milestone No.: N/A

Responsible Branch & Project Manager: LB #2; D. Houston

Reviewers: J. Stang, F. Witt

Requested Completion Date: October 31, 1983

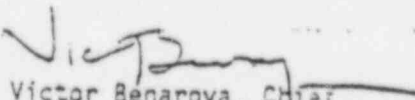
Status: Complete

We have reviewed the plant Technical Specifications and compared them to the Standard Technical Specifications to identify any deviations. Our findings are attached in (Enclosure 1).

In addition, we recommended that emergency lights and portable fire extinguishers also be included in the technical specifications. Proposed technical specifications for both are enclosed. (Enclosure 2).

By letter dated September 9, 1983, the applicant proposed additional changes to the fire detection instrumentation and hose station sections of the fire protection technical specifications. We have reviewed the proposed changes and compared them to the Standard Technical Specifications to identify any deviations. Our findings are included in Enclosure 1.

With the revisions as stated in the enclosures we conclude that these sections of the plant technical specifications are acceptable.

  
Victor Benaroya, Chief  
Chemical Engineering Branch  
Division of Engineering

Enclosures: As stated

cc: See next page

Contacts: F. Witt  
x29360

J. Stang  
x24730

8311140487

By letter dated September 9, 1983, the applicant requested changes to the Fire Detection Instrumentation and Hose Station sections of the Technical Specifications.

Based on our review, we conclude that the changes to the Hose Station section reflect the as-built conditions in the plant and therefore are acceptable. We recommend the following changes to the Fire Detection Instrumentation Section of the proposed technical specification.

1. Page 3/4 3-76, paragraph 3.3.7.9 - Action paragraphs a., b. and c. - these paragraphs should be rewritten as follows:

a. With any, but not more than one-half the total in any fire zone, Function A fire detection instruments shown in Table 3.3.7.9-1 inoperable, restore the inoperable instrument(s) to operable status within 14 days or within the next 1 hour establish a fire watch patrol to inspect the zone(s) with the inoperable instrument(s) at least once per hour, unless the instrument(s) is located inside the containment, drywell, or steam tunnel then inspect that containment zone at least once per 8 hours (or monitor the containment drywell and steam tunnel air temperatures at least once per hour at the locations listed in Specification 4.6.1.6).

b. With more than one-half of the Function A fire detection instruments in any fire zone shown in Table 3.3.7.9-1 inoperable, or with any Function B fire detection instruments shown in Table 3.3.7.9-1 inoperable, or with any two or more adjacent fire detection instruments shown in Table 3.3.7.9-1 inoperable, within 1 hour establish a fire watch patrol to inspect the zone(s) with the inoperable instrument(s) at least once per hour, unless the instrument(s) is located inside the containment, drywell, or steam tunnel then inspect that containment zone at least once per 8 hours (or monitor the containment air temperature drywell and steam tunnel at least once per hour at the locations listed in Specification 4.6.1.6).

respond

073

- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.
2. Page 3/4 3-77, 3/4 3-78, 3/4 3-79, 3/4 3-80, 3/4 3-80a, 3/4 3-80b, 3/4 3-80c and 3/4 3-80d. Table 3.3.7.9.1 - Revise table 3.3.7.9-1 using the following as an illustration:

TABLE 3.3.7.9-1  
FIRE DETECTION INSTRUMENTATION

<u>INSTRUMENT LOCATION</u> (Illustrative)	<u>HEAT</u> (x/y)	<u>TOTAL NUMBER</u> <u>OF INSTRUMENTS</u> **	
		<u>FLAME</u> (x/y)	<u>SMOKE</u> (x/y)
1. Containment #			
a. Zone 1 Elevation _____			
b. Zone 2 Elevation _____			
2. Control Room			
3. Cable Spreading			
a. Zone 1 Elevation _____			
b. Zone 2 Elevation _____			
4. Computer Room			
5. Switchgear Room			
6. Remote Shutdown Panels			
7. Station Battery Rooms			

8. Turbine

- a. Zone 1 Elevation \_\_\_\_
- b. Zone 2 Elevation \_\_\_\_

9. Diesel Generator

- a. Zone 1 Elevation \_\_\_\_
- b. Zone 2 Elevation \_\_\_\_

10. Safety Related Pumps

- a. Zone 1 Elevation \_\_\_\_
- b. Zone 2 Elevation \_\_\_\_

11. Fuel Storage

- a. Zone 1 Elevation \_\_\_\_
- b. Zone 2 Elevation \_\_\_\_

(List all detectors in areas required to ensure the OPERABILITY of safety-related equipment).

\* (x/y): x is number of Function A (early warning fire detection and notification only) instruments.

y is number of Function B (actuation of Fire Suppression Systems and early warning and notification) instruments.

#The fire detection instruments located within the containment are not required to be OPERABLE during the performance of Type A containment leakage rate tests.

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 076

Priority: 1/B

\_\_\_\_\_  
Identified By / Date

\_\_\_\_\_  
Responsible Supervisor

Tech Spec Reference: 3/4.3.3 Table 3.3.3-3.2.a

Problem Title: ECCS Response Time

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The low pressure coolant injection (LPCI) mode of RHR System is noted in Tech Specs as having a pump response time of 45 seconds for trains A and B. In order to consistent with the accident analysis assumptions as noted in FSAR Table 6.3-1 and 6.3-2 the correct response time is 40 seconds.

2. Safety Significance:

The proposed change is somewhat more limiting than the original requirement by its shorter response time and is consistent with the existing accident analysis.

3. Anticipated Resolution:

Change Tech Spec table 3.3.3-3 item 2 to 40 seconds.  
*Incorporated in surveillance procedure.*

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: \_\_\_\_\_  
Individual Notified / Date / Time

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

\_\_\_\_\_  
Date / Time

cc: J. E. Cross  
R. F. Rogers



6. (GCNS - 638)

SUBJECT: Technical Specification 3.3.3, Table 3.3.3-3, page 3/4 3-10.

DISCUSSION: The low pressure coolant injection (LPCI) mode of the ~~RPV~~ System is noted in the subject table as having a pump response time of 45 seconds for trains A and B. In order to be consistent with the accident analysis assumptions as noted in FSAR Tables 6.3-1 and 6.3-2 the correct response time is 40 seconds for all trains.

The proposed change deletes the distinction between trains A and B and C and utilizes the most limiting response time consistent with the accident analysis.

JUSTIFICATION: The proposed change is somewhat more limiting than the original requirement by its shorter response time. This change brings the technical specifications to a basis consistent with the plant design as represented in the accident analyses.

SIGNIFICANT HAZARDS CONSIDERATION:

The proposed change is more restrictive and is consistent with the existing accident analyses. It does not increase the probability or consequences of an accident previously evaluated nor does it create the possibility of a new or different accident from those previously evaluated. It does not constitute a significant hazards consideration.

TABLE B.3.2-3

EMERGENCY CORE COOLING SYSTEM RESPONSE TIMES (SECONDS)

1. LOW PRESSURE CORE SPRAY SYSTEM	≤ 40
2. LOW PRESSURE COOLANT INJECTION MODE OF BHR SYSTEM Pumps A, B and C	≤ 40
<del>a. Pumps A and B</del>	<del>≤ 45</del>
<del>b. Pump C</del>	<del>≤ 40</del>
3. AUTOMATIC DEPRESSURIZATION SYSTEM	DA
4. HIGH PRESSURE CORE SPRAY SYSTEM	≤ 27
5. LOSS OF POWER	DA

Identified By \_\_\_\_\_ Date 1

Responsible Supervisor \_\_\_\_\_

Tech Spec Reference: 3-3.3.3 <sup>Table</sup> 3-3.3-3.2.a

Problem Title: ECCS Response Time

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other): \_\_\_\_\_

The low pressure coolant injection (LPCI) mode of RHR System is noted in Tech Specs as having a pump response time of 45 seconds for trains A and B. In order to be consistent with the accident analysis assumptions as noted in FSAR Table 6.2-1 and 6.2-2 the correct response time is 40 seconds.

2. Safety Significance: The proposed change is conservative more restrictive than the original requirement by its shorter response time and is consistent with the existing accident analysis.

3. Anticipated Resolution: Change Tech Spec table 3.3.3.3 item 2 to 40 seconds.

4. NRC Response to Item (NRR/IE): \_\_\_\_\_

NRC Notified: Dennis (i.e. 183-505, item 6) 4-7-83  
Individual Notified \_\_\_\_\_ Date \_\_\_\_\_ Time \_\_\_\_\_

5. Disposition: \_\_\_\_\_

Items Closed: (How) \_\_\_\_\_

Date \_\_\_\_\_ Time \_\_\_\_\_

cc: J. E. Cross  
R. F. Rogers