



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated June 19, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

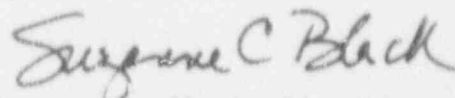
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 56, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Suzanne C. Black, Director
Project Directorate IV-2
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 10, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 56.

UTILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

3/4 3-41
6-7
6-15

INSERT

3/4 3-41
6-7
6-15

TABLE 3.3-6 (Continued)

TABLE NOTATIONS

- *With fuel in the respective fuel storage pool.
- **With irradiated fuel in the fuel storage areas or fuel building.
- #Trip Setpoint concentration value ($\mu\text{Ci}/\text{cm}^3$) is to be established such that the actual submersion dose rate would not exceed 2 mR/h in the control room.
- ##Trip Setpoint concentration value ($\mu\text{Ci}/\text{cm}^3$) is to be established such that the actual submersion dose rate would not exceed 4 mR/h in the fuel building.
- ###Trip Setpoint concentration value ($\mu\text{Ci}/\text{cm}^3$) is to be established such that the actual submersion dose rate would not exceed 9 mR/h in the containment building. The Setpoint value may be increased up to the equivalent limits of Section 3.1 of the ODCM in accordance with the methodology and parameters in the ODCM during containment purge or vent provided the Setpoint value does not exceed twice the maximum concentration activity in the containment determined by the sample analysis performed prior to each release in accordance with Table 3-1 of the ODCM.

ACTION STATEMENTS

- ACTION 26 - With less than the Minimum Channels OPERABLE requirement, operation may continue provided the containment purge valves are maintained closed.
- ACTION 27 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, isolate the Control Room Emergency Ventilation System and initiate operation of the Control Room Emergency Ventilation System in the recirculation mode within 72 hours, or with no OPERABLE channels within 1 hour.
- ACTION 28 - With less than the Minimum Channels OPERABLE requirement, operation may continue for up to 30 days provided an appropriate portable continuous monitor with the same Alarm Setpoint is provided in the fuel area. Restore the inoperable monitors to OPERABLE status within 30 days or suspend all operations involving fuel movement in the fuel building.
- ACTION 29 - Must satisfy the ACTION requirements for Specification 3.4.6.1.
- ACTION 30 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, isolate the Fuel Building Ventilation System and initiate operation of the Emergency Exhaust System to maintain the fuel building at a negative pressure within 72 hours, or with no OPERABLE channels within 1 hour.

WOLF CREEK - UNIT 1

3/4 3-42

TABLE 4.3-3

RADIATION MONITORING INSTRUMENTATION FOR PLANT
OPERATIONS SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
1. Containment				
a. Containment Atmosphere-Gaseous Radioactivity-High (GT-RE-31 & 32)	S	R	M	All
b. Gaseous Radioactivity-RCS Leakage Detection (GI-RE-31 & 32)	S	R	M	1, 2, 3, 4
c. Particulate Radioactivity - RCS Leakage Detection (GT-RE-31 & 32)	S	R	M	1, 2, 3, 4
2. Fuel Building				
a. Fuel Building Exhaust-Gaseous Radioactivity-High (GG-RE-27 & 28)	S	R	M	**
b. Criticality-High Radiation Level				
1) Spent Fuel Pool (SD-RE-37 & 38)	S	R	M	*
2) New Fuel Pool (SD-RE-35 & 36)	S	R	M	*
3. Control Room				
Air Intake-Gaseous Radioactivity-High (GK-RE-04 & 05)	S	R	M	All

*With fuel in the respective fuel storage pool.
 **With irradiated fuel in the fuel storage areas or fuel building.

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATION (Continued)

- b. The position of Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.
- c. The NSRC members shall meet or exceed the requirements of ANSI/ANS 3.1-1981.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager Training and shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1978 with the following exceptions:

- a. The training program for Licensed Operators and Senior Operators shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1981 as endorsed by Regulatory Guide 1.8, Revision 2, and 10 CFR Part 55.
- b. Training shall include familiarization with relevant industry operational experience identified by the ISEG or another plant group.

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY REVIEW COMMITTEE (PSRC)

FUNCTION

6.5.1.1 The PSRC shall function to advise the Director Plant Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PSRC shall be composed of the:

Member:	Manager System Engineering
Member:	Manager Operations
Member:	Manager Technical Support
Member:	Manager Maintenance and Modifications
Member:	Manager Instrumentation and Control
Member:	Supervisor Reactor Engineering
Member:	Manager Radiation Protection
Member:	Manager Chemistry
Member:	Supervisor Results Engineering
Chairman:	Manager Plant Support

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSRC activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The PSRC shall be responsible for:

- a. Review of: (1) all procedures required by Specification 6.8 and changes thereto, (2) all programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Director Plant Operations to affect nuclear safety;
- b. Review of all proposed changes, tests and experiments which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Review of all proposed changes to Technical Specifications or the Operating License;
- d. Review of all safety evaluations performed under the provision of Section 50.59(a)(1), 10 CFR, for changes, tests and experiments;
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President Operations, and to the Nuclear Safety Review Committee (NSRC);
- f. Review of all REPORTABLE EVENTS;
- g. Review of reports of operating abnormalities, deviations from expected performance of plant equipment and of unanticipated deficiencies in the design or operation of structures, systems or components that affect nuclear safety;
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman, NSRC;
- i. Review of the plant Security Plan and implementing procedures and shall submit recommended changes to the NSRC;
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the NSRC;

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

6.8.3 Changes to Procedures

- a. Temporary changes to Major Procedures, of the categories listed in Specification 6.8.1 which do not change the intent or generate an unreviewed safety question of the original or subsequent approved procedure, may be made provided such changes to operating procedures are approved by the Shift Supervisor (SRO licensed) and one of the Call Superintendents. For temporary changes to Major Procedures under the jurisdiction of groups other than Operations (e.g., Maintenance, Instrumentation and Control, Reactor Engineering, Chemistry, Health Physics) which do not change the intent or generate an unreviewed safety question, changes may be made upon approval of the Cognizant Group Leader and a Call Superintendent.
All temporary changes to Major Procedures (made by a Call Superintendent and either a Cognizant Group Leader or the Shift Supervisor) shall subsequently be reviewed by the PSRC and approved by the Director Plant Operations within 14 days, except that temporary changes to Major Procedures made during a refueling outage may be reviewed and approved at any time prior to initial criticality of the reload core. All permanent changes to Major Procedures shall be made in accordance with Specification 6.8.2.a.
- b. All temporary or permanent changes to Minor Operating Procedures (checkoff lists, alarm responses, data sheets, operating instructions, etc.) shall be approved by the Shift Supervisor, and shall be subsequently reviewed and approved by the Operations PSRC Subcommittee. All temporary or permanent changes to other Minor Procedures under the jurisdiction of groups other than Operations (e.g., Maintenance, Instrumentation and Control, Reactor Engineering, Chemistry, Health Physics) shall be approved by a Cognizant Group Leader and shall be subsequently reviewed and approved by the appropriate PSRC Subcommittee.
- c. Temporary changes to Corporate Emergency Plan implementing procedures may be made provided that: (1) the intent of the original procedure is not altered, (2) the change is approved by the Manager Technical Services, and (3) the change is documented, reviewed by appropriate Corporate and plant personnel and approved by the President and Chief Executive Officer within 14 days of the implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

- a. Reactor Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the appropriate portions of the Containment Spray System, Safety Injection System, Chemical and Volume Control System, RHR System, and the Nuclear Sampling System (PASS only). The program shall include the following:

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- 1) Preventive maintenance and periodic visual inspection requirements, and
- 2) Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.

c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- 1) Identification of a sampling schedule for the critical variables and control points for these variables,
- 2) Identification of the procedures used to measure the values of the critical variables,
- 3) Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in-leakage,
- 4) Procedures for the recording and management of data,
- 5) Procedures defining corrective action for all off-control point chemistry conditions, and
- 6) A procedure identifying: (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

d. Post-accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- 1) Training of personnel,
- 2) Procedures for sampling and analysis, and
- 3) Provisions for maintenance of sampling and analysis equipment.