

strument uncertainty allotted for the trip unit drift. The numerical difference between the trip setpoints and allowable values for Grand Gulf include both trip unit drift and sensor drift. This extra drift allowance could permit continued operation with instrument channels that would not actuate safety systems to terminate transients or mitigate accidents within the bounds specified in the FSAR analyses. Therefore, the Grand Gulf-Unit 1 Technical Specification's trip setpoints and allowable values should be revised to include only the drift associated with that portion of the instrument channel, tested at 31 day intervals (i.e., trip unit). Additional changes to the Grand Gulf-Unit 1 Technical Specifications may be required following our review of the licensee's methodology position statement.

078

5. By letter dated October 11, 1983 from J. McGaughy (MP&L) to H. Denton (NRC) the licensee proposed changes to the Grand Gulf-Unit 1 Technical Specifications that address the RCIC actuation instrumentation. We have reviewed the proposed changes and found them acceptable. A discussion on the proposed changes is contained in the ICSB input to Supplement No. 5 of the SER. The Grand Gulf-Unit 1 Technical Specifications should be revised accordingly.

037

6. By letter dated October 14, 1983 from L. Dale (MP&L) to H. Denton the licensee committed to propose revisions to the Grand Gulf-Unit 1 Technical Specifications. The proposed changes will require more frequent calibrations of components in the containment isolation actuation instrumentation. We have reviewed the proposal and found it acceptable. A discussion on the proposal is contained in the ICSB input to Supplement No. 5 of the SER. The Grand Gulf-Unit 1 Technical Specifications should be revised accordingly.

DEFINITIONS FOR
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"
 FOR REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION
 INSTRUMENTATION TABLE 3.3.5-1

<u>Trip Unit</u>	<u>Parameter</u>	<u>Logic</u>	
B21-LS-N692B	RPV Level - Lvl 2	One out of two twice	RCIC Initiation
B21-LS-N692F	RPV Level - Lvl 2 *		
B21-LS-N692A	RPV Level - Lvl 2		
B21-LS-N692E	RPV Level - Lvl 2		
E51-HS-M625	Manual Initiation	Either One	RCIC Initiation
B21-LS-N693A	RPV Level High - Lvl 8		
B21-LS-N693B	RPV Level High - Lvl 8	Both	Close E51-F045
E51-LIS-N635A	Condensate Stg Tank Level Lo	Any One	Switch Pump Suction to Suppression Pool TRIP FUNCTION
E51-LIS-N635E	Condensate Stg Tank Level Lo		
E51-LIS-N636A	Suppression Pool Wtr Lvl Hi		
E51-LIS-N636E	Suppression Pool Wtr Lvl Hi		

TRIP SYSTEM

* One Channel (Typical of 11 shown on this page)

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 168

Priority: ZIA

Identified By _____ / _____
Date

Responsible Supervisor _____

Tech Spec Reference: 3.6.3.1

Problem Title: Suppression Pool Temp Require in OP condition 3

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):
The Spec is confusing because it doesn't contain any action statements for op condition 3, although the operator is instructed to go to Cold shutdown.

2. Safety Significance:
Operating instructions are confusing.

3. Anticipated Resolution:
Investigate to see if statement need rewording or if CCO should be applicable in op condition 3 as well as 1 and 2.
could use position statement when determine what spec should be.

4. NRC Response to Item (NRR/IE): _____

NRC Notified: _____ / _____
Individual Notified Date Time

5. Disposition: _____

Items Closed: (How) _____

_____ / _____
Date Time

cc: J. E. Cross
R. F. Rogers

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 185

Priority: 51A

Identified By _____ / _____
Date

Responsible Supervisor _____

Tech Spec Reference: T.S.3.11.2.7 & Surveillance 4.11.2.7.1

Problem Title: Change "Hot Standby" to "Cold Shutdown"

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

- 1) Spec directs operation to go to hot standby, but should say cold shutdown.
- 2) Sampling is required prior to entering op condition 2, when the system is not in service, which is an unnecessary monitoring requirement.

2. Safety Significance: None: Investigation Required.

3. Anticipated Resolution: 1) Change to cold shutdown. 2) Add to Surveillance 4.11.2.7.1 an exception to 4.0.4 to not sample till after entering op condition 2.
Could use Tech Spec position statement.

4. NRC Response to Item (NRR/IE): _____

NRC Notified: _____ / _____
Individual Notified Date Time

5. Disposition: _____

Items Closed: (How) _____

_____ / _____
Date Time

cc: J. E. Cross
R. F. Rogers

SUBJECT: Table 3.3.7.1-1 of Technical Specification 3.3.7.1, pages
3/4 3-56, 3/4 3-57 and 3/4 3-58

No. 198

DISCUSSION: This change proposes to revise the Minimum Operable Channels for items 7, 8 and 9 of Table 3.3.7.1-1 and to revise their associated Action Statements. The affected items are:

7. Containment and Drywell Ventilation Exhaust Radiation Monitor
8. Fuel Handling Area Ventilation Exhaust Radiation Monitor
9. Fuel Handling Area Pool Sweep Exhaust Radiation Monitor

Currently, 3 minimum channels operable are required for these items. This should be revised to 2/trip system for each item. In addition the b section of the Action statements for these items (Actions 74 and 75) should be revised such that 2 inoperable monitors per trip system, instead of 2 inoperable monitors, will cause entry into the b section and note h should be deleted from the table.

JUSTIFICATION: These revisions are required in order to reflect the design of the affected radiation monitors ^{system} (see attachment 1). The logic for each of these items consists of 4 monitors, with each monitor comprising one channel of a trip system. ^{The channels are grouped in pairs} ~~There are two~~ ^{separate trip systems} ~~trip systems~~ associated with ^{each} ~~each~~ ~~trip system~~ ~~consisting~~ ~~of~~ ~~2~~ ~~channels~~. Both channels ⁱⁿ a trip system are required to trip in order to initiate a trip function. The current requirement of 3 minimum operable channels does not apply to the instrumentation design ~~which only requires 2 monitors operable~~.

design and its associated two-out-of-two logic. By only requiring 3 monitors operable, it would be possible to have one trip system inoperable without entering the b section of the applicable Action statement. This would be inconsistent with the single-failure proof intent of the design of these monitoring systems. To make the specification consistent with the

~~the~~ design configuration a minimum of 2 operable channels per trip system is required. Revision of the applicable Action statements is required to prevent entry into the b section in the event that two channels in different trip systems are inoperable. This would be overly restrictive since two operable channels in different trip systems would still be available to provide monitoring and initiation of the trip function, if required. Note h should be deleted since it is no longer needed.

SIGNIFICANT HAZARDS CONSIDERATION:

This change makes the technical specification for the affected instrumentation systems consistent with the design configuration of the systems. ~~It~~^{It} ensures that adequate monitoring exists ^{and} that the plant is placed in a safe, conservative condition ~~in the event that~~ if such monitoring does not exist. This change does not a) reduce the margin of safety, b) increase the probability or consequences of a previously evaluated accident or c) create the possibility of a new or different kind of accident. Thus the proposed change does not involve a significant hazards consideration.

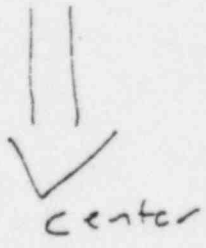
Attachment 1

<u>Trip unit/ monitor</u>	<u>Parameter</u>	<u>Logic Function</u>
D17-RITS-K609A	Containment + DW Exh. Hi Hi, Inop or Downscale	Both
D17-RITS-K609D	Containment + DW Exh. Hi Hi, Inop or Downscale	
----- TRIP SYSTEM -----		
D17-RITS-K609B	Containment + DW Exh. Hi Hi, Inop or Downscale	Both
D17-RITS-K609C	Containment + DW Exh. Hi Hi, Inop or Downscale	
----- TRIP SYSTEM -----		

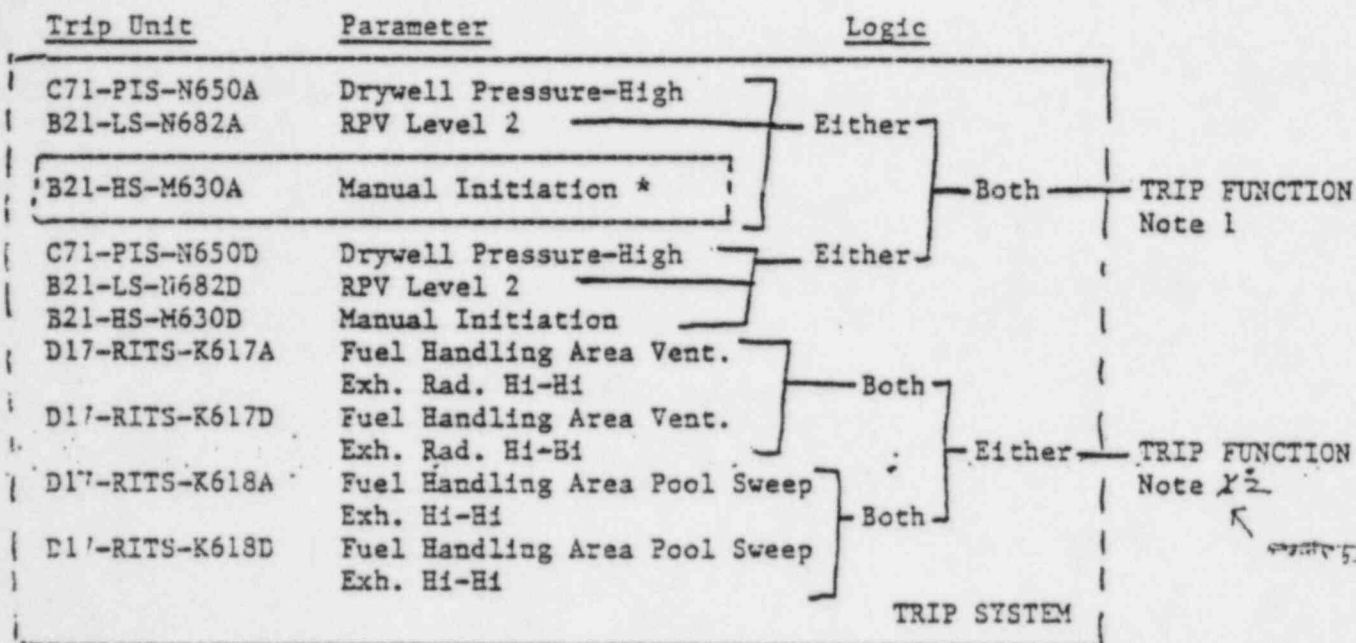
Closes outlo
 containment
 Gr 7 and
 Inboard
 Drywell Gr 7
 valve
 TRIP FUNCT

closes Inboard
 Containment
 Gr 7 and Outlo
 Drywell Gr 7 valve
 TRIP FUNCT

- ~~D17-RITS~~
- ~~D17-RITS~~
- ~~D17-RITS~~
- ~~D17-RITS~~
- D17
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- D17
- D17



Attachment 1 (continued)

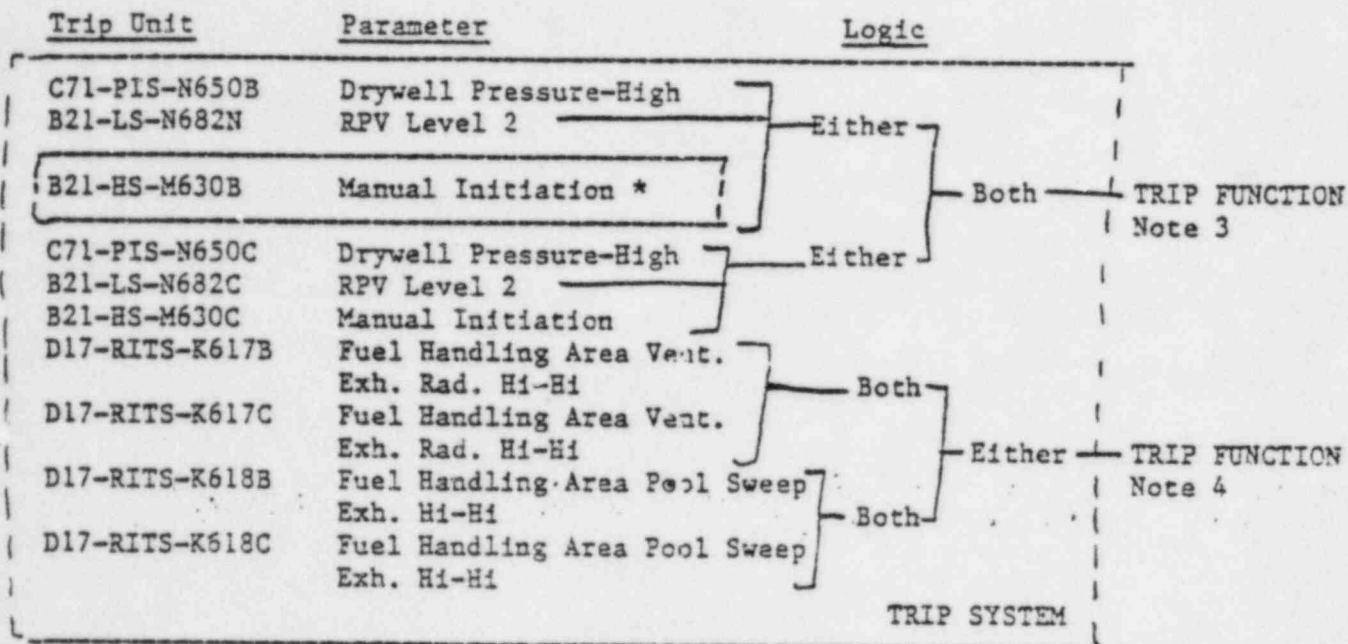


Notes: 1) Closes Containment Outboard Dampers, Drywell Ventilation Inboard Dampers, Starts Standby Gas Treatment "A", closes Auxiliary Building Fuel Handling Area Inboard Dampers, closes Auxiliary Building Ventilation System Inboard Dampers, closes Containment Ventilation System Inboard Dampers, closes Control Room Ventilation System Outboard Dampers, Starts Control Room Emergency Filtration "A".

2) Starts Standby Gas Treatment "A", close Auxiliary Building Fuel Handling Area Ventilation Inboard Dampers, closes Auxiliary Building Ventilation System Inboard Dampers, closes Containment Ventilation System (Auxiliary Building Inboard Dampers)

* One Channel (typical of 10 shown on this page)

Attachment 1 (continued)



Notes: 3) Closes Containment Inboard Dampers, Drywell Ventilation Outboard Dampers, Starts Standby Gas Treatment "B", closes Auxiliary Building Fuel Handling Area Outboard Dampers, closes Auxiliary Building Ventilation System Outboard Dampers, closes Containment Ventilation System Outboard Dampers, closes Control Room Ventilation System Inboard Dampers, Start Control Room Emergency Filtration "B".

4) Starts Standby Gas Treatment "B", Close Auxiliary Building Fuel Handling Area Ventilation System Outboard Dampers, closes Auxiliary Building Ventilation System Outboard Dampers, closes Containment Ventilation System (Auxiliary Building Outboard Dampers.)

* One Channel (Typical of 10 shown on this page)

TABLE 3.3.7.1-1 (Continued)
RADIATION MONITORING INSTRUMENTATION

INSTRUMENTATION	MINIMUM CHANNELS OPERABLE	APPLICABLE CONDITIONS	ALARM/TRIP SEIPOINT	MEASUREMENT RANGE	ACTION
10. Area Monitors					
a. Fuel Handling Area Monitors					
1) New Fuel Storage Vault	1	(e)	≤ 2.5 mR/hr/NA	10^{-2} to 10^3 mR/hr	72
2) Spent Fuel Storage Pool	1	(f)	≤ 2.5 mR/hr/NA	10^{-2} to 10^3 mR/hr	72
3) Dryer Storage Area		(g)	≤ 2.5 mR/hr/NA	10^{-2} to 10^3 mR/hr	72
b. Control Room Radiation Monitor	1	At all times	≤ 0.5 mR/hr/NA	10^{-2} to 10^3 mR/hr	72

* With RHR heat exchangers in operation.

** When irradiated fuel is being handled in the primary or secondary containment.

Initial setpoint. Final Setpoint to be determined during startup test program. Any required change to this setpoint shall be submitted to Commission within 90 days after test completion.

- (a) Trips system with 2 channels upscale-Hi Hi Hi, or one channel upscale Hi Hi Hi and one channel downscale or 2 channels downscale.
- (b) Isolates containment/drywell purge penetrations.
- (c) With irradiated fuel in spent fuel storage pool.
- (d) Also isolates the Auxiliary Building and Fuel Handling Area Ventilation Systems.
- (e) With fuel in the new fuel storage vault.
- (f) With fuel in the spent fuel storage pool.
- (g) With fuel in the dryer storage area.

TABLE 3.3.7.1-1
RADIATION MONITORING INSTRUMENTATION

INSTRUMENTATION	MINIMUM CHANNELS OPERABLE	APPLICABLE CONDITIONS	ALARM/TRIP SETPOINT	MEASUREMENT RANGE	ACTION
1. Component Cooling Water Radiation Monitor	1	At all times	$\leq 1 \times 10^5$ cpm/NA	10 to 10^6 cpm	70
2. Standby Service Water System Radiation Monitor	1/heat exchanger train	1, 2, 3, and*	$\leq 1 \times 10^5$ cpm/NA	10 to 10^6 cpm	70
3. Offgas Pre-treatment Radiation Monitor	1	1, 2	$\leq 5 \times 10^3$ mR/hr/NA	1 to 10^6 mR/hr	70
4. Offgas Post-treatment Radiation Monitor	2(d)	1, 2	$\leq 1 \times 10^5$ cpm (Hi), $\leq 1.0 \times 10^6$ cpm (Hi Hi Hi)	10 to 10^6 cpm	71
5. Carbon Bed Vault Radiation Monitor	1	1, 2	$\leq 2 \times$ full power background/NA	1 to 10^6 mR/hr	72
6. Control Room Ventilation Radiation Monitor	2/trip system	1,2,3,5 and**	≤ 4 mR/hr/ ≤ 5 mR/hr#	10^{-2} to 10^2 mR/hr	73
7. Containment and Drywell Ventilation Exhaust Radiation Monitor	2(h) 2/trip system	At all times	≤ 2.0 mR/hr/ ≤ 4 mR/hr(b)#	10^{-2} to 10^2 mR/hr	74
8. Fuel Handling Area Ventilation Exhaust Radiation Monitor	2(h) 2/trip system	1,2,3,5 and**	≤ 2 mR/hr/ ≤ 4 mR/hr(d)#	10^{-2} to 10^2 mR/hr	75
9. Fuel Handling Area Pool Sweep Exhaust Radiation Monitor	2(h) 2/trip system	(c)	≤ 18 mR/hr/ ≤ 35 mR/hr(d)#	10^{-2} to 10^2 mR/hr	75

INSTRUMENTATION

TABLE 3.3.7.1-1 (Continued)

RADIATION MONITORING INSTRUMENTATION

ACTION

- ACTION 70 - With the required monitor inoperable, obtain and analyze at least one grab sample of the monitored parameter at least once per 24 hours.
- ACTION 71 -
- a. With one of the required monitors inoperable, place the inoperable channel in the downscale tripped condition within one hour.
 - b. With both of the required monitors inoperable, be in at least HOT SHUTDOWN within 12 hours.
- ACTION 72- With the required monitor inoperable, perform area surveys of the monitored area with portable monitoring instrumentation at least once per 24 hours.
- ACTION 73 -
- a. With one of the required monitors in a trip system inoperable, place the inoperable channel in the downscale tripped condition within one hour; restore the inoperable channel to OPERABLE status within 7 days, or, within the next 5 hours; initiate and maintain operation of at least one control room emergency filtration system in the isolation mode of operation.
 - b. With both of the required monitors in a trip system inoperable, initiate and maintain operation of at least one control room emergency filtration system in the isolation mode of operation within one hour.
- ACTION 74 -
- a. With one of the required monitors inoperable, place the inoperable channel in the downscale tripped condition within one hour.
 - b. With two of the required monitors ^{in a trip system} inoperable, isolate the containment and drywell purge and vent penetrations within 12 hours.
- ACTION 75 -
- a. With one of the required monitors inoperable, place the inoperable channel in the downscale tripped condition within one hour.
 - b. With two of the required monitors ^{in a trip system} inoperable, initiate and maintain operation of at least one standby gas treatment subsystem within 12 hours.

Identified By _____ Date 1

Responsible Supervisor _____

Tech Spec Reference: Table 3.3.7.1-1 Item 6, 7, 8, 9

Problem Title: RADIATION MONITOR

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other): _____

Min channels required ^{operable} for items 6, 7, 8, 9 not the same even though logics are the same

2. Safety Significance: Investigation required if ^{2 or 3} minimum channels are required and since presently we only require 3 channels operable, we could be operating unconservatively in relation to FSAR analysis

3. Anticipated Resolution: investigate proposed Tech Spec changes if required

4. NRC Response to Item (NRR/IE): _____

NRC Notified: _____
Individual Notified _____ Date 1 Time _____

5. Disposition: _____

Items Closed: (How) _____
Date _____ Time _____

cc: J. E. Cross
R. F. Rogers

"TECH SPEC PRIORITY"

Punchlist Item # 198

Tech Spec 3/4.3.7

Priority 1A

TO: Manager of Nuclear Plant Engineering

FROM: Chairman, Prioritization and Disposition Chairman

SUBJECT: Technical Specifications Punchlist Item # 198

PDTS:84/ 0008

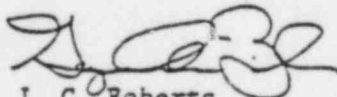
DATE: 3/10/84

The subject Tech Spec item has been determined by the Disposition Committee to require Engineering support.

DETAILS: The min. operable channels for items 6,7,8,9 of Table 3.3.7.1-1 are not the same even though the logics are similar. Evaluate making them consistent & make recommendations.

Please contact Joe Hendry at Extension 2678 for further information.

Please refer to the Tech Spec Punchlist item number in your response. Forward your response to G. Zinke


J. C. Roberts
Chairman

LLJ/JCR:swb

cc: Mr. C. L. Tyrone
Mr. J. E. Cross
Mr. D. Stonestreet
Mr. A. S. McCurdy
Mr. S. Hutchins
Mr. J. Hendry
File (Tech Spec Records)

A4/61swb1

"TECH SPEC PRIORITY"

Punchlist Item # See ATTACHED

Tech Spec See ATTACHED

Priority See ATTACHED

TO: Manager of Nuclear Plant Engineering
FROM: Chairman, Prioritization and Disposition Chairman
SUBJECT: Technical Specifications Punchlist Item # See ATTACHED
PDTS: 84/ 0014
DATE: 3/10/84

The subject Tech Spec item has been determined by the Disposition Committee to require Engineering support.

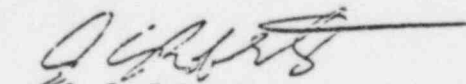
DETAILS: This letter identifies requested response dates for the following Tech Spec problems:

# 199 Letter No. PDTS 84/0001	# 015 Letter No. PDTS 84/0007
# 180 Letter No. PDTS 84/0002	# 198 Letter No. PDTS 84/0008
# 033 Letter No. PDTS 84/0003	# 202 Letter No. PDTS 84/0009
# 054 Letter No. PDTS 84/0004	# 213 Letter No. PDTS 84/0010
# 001 Letter No. PDTS 84/0005	# 219 Letter No. PDTS 84/0011
# 016 Letter No. PDTS 84/0006	# 118 Letter No. PDTS 84/0013

It is requested that the responses to the above items be completed by 3/13/84

Please contact Jenny Roberts at Extension 2695 for further information.

Please refer to the Tech Spec Punchlist item number in your response. Forward your response to George Zinke


J. C. Roberts
Chairman

LLJ/JCR:swb
cc: Mr. C. L. Tyrone
Mr. J. E. Cross
Mr. D. Stonestreet
Mr. A. S. McCurdy
Mr. S. Hutchins
Mr. J. Hendry
File (Tech Spec Records)

A4/61swb1

<u>Tech Spec Problem No.</u>	<u>Tech Spec</u>	<u>Priority</u>
199	Table 3.3.6-1.5	1B
180	4.8.4.3	1D
033	Table 3.3.8-2	1B
054	3/4.3.8	1B
001	3/4.5.1	1B
016	3/4.3.8	1B
015	3/4.3.2	1D
198	3/4.3.7	1B
202	3/4.3.7	1B
213	3/4.3.3	1B
219	Figure 3.4.6.1-1	1B
168	3.6.3.1	1B

"TECH SPEC PRIORITY"

MEMO TO: J. F. Pinto, Manager of Nuclear Plant Engineering

FROM: C. L. Tyrone, Project Manager

SUBJECT: Handling of Tech Spec Review Items

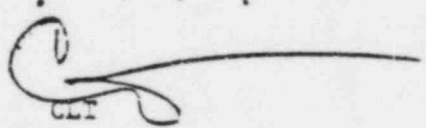
TSRO: 84/0001

DATE: March 11, 1984

This memorandum confirms our conversation of March 10, 1984. At that time, your assistance was requested in resolving discrepancies on eleven priority 1 items. Since then two items have been added. These items are all previously identified items which require early resolution with the NRC. A response is needed on these items by 12:30 PM on March 11, 1984. A list is attached.

Furthermore, all items of any priority identified (or previously known) which are being handled on this program require expeditious handling. This includes areas where requests are originated from other interfacing organizations such as the Plant or Nuclear Services. In any case where conflicts regarding highest priority is not clear, I am available to provide clarification.

It is suggested that you arrange 7 day a week support in this area as it is needed and arrange for all NPE personnel who will be involved in this effort to be available (or on call) in a manner that will support the Tech Spec Review program.


CLT

SHH:sad
Attachment

cc: J. B. Richard (w/a)
J. P. McGaughy (w/a)
J. F. Pinto (w/a)
J. E. Cross (w/a)
T. H. Cloninger (w/a)
H. J. Green (w/a)
R. C. Fron (w/a)
D. W. Stonestreet (w/a)

~~_____~~
T. E. Reaves, Jr. (w/a)
S. M. Feith (w/a)
J. G. Cesare (w/a)
G. W. Smith (w/a)
L. R. McKay (w/a)
L. C. Burgess (w/a)
File (Tech Spec Records) (w/a)

LIST OF CURRENT PRIORITY 1
ITEMS REQUIRING NPE SUPPORT

PDTS:84/	P/L #	Date Sent
001	199	3/10/84
002	180	3/10/84
003	033	3/10/84
004	054	3/10/84
005	001	3/10/84
006	016	3/10/84
007	015	3/10/84
008	198	3/10/84
009	202	3/10/84
010	213	3/10/84
011	219	3/10/84
012	083	3/10/84
013	168	3/10/84

DEFINITIONS FOR
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"
 FOR RADIATION MONITORING INSTRUMENTATION TABLE 3.3.7.1-1

<u>Trip Unit/ Monitor</u>	<u>Parameter</u>	<u>Logic/ Function</u>	
D17-LCRM-K607	Comp. Clg. Wtr.	Hi Alarm/Monitor	
D17-LCRM-K604	SSW Loop A	Hi Alarm/Monitor	
D17-LCRM-K605	SSW Loop B *	Hi Alarm/Monitor	
D17-RITS-K612	Offgas Pre-treatment	Hi Alarm/Monitor	
D17-LCRM-K601A	Offgas Post-treatment	Hi & Hi Hi Hi Alarm/Monitor	
D17-LCRM-K601B	Offgas Post-treatment	Hi & Hi Hi Hi Alarm/Monitor	
D17-LCRM-K601A	Offgas Post-treatment Hi Hi Hi, Inop or Downscale	Both	Closes Offgas Disch. N64-F060 TRIP FUNCTION
D17-LCRM-K601B	Offgas Post-treatment Hi Hi Hi, Inop or Downscale		
			TRIP SYSTEM
D17-RITS-K611	Carbon Bed Vault	Hi Alarm/Monitor	
D17-RITS-K621A	Control Room Vent.	Hi Alarm/Monitor	
D17-RITS-K621B	Control Room Vent.	Hi Alarm/Monitor	
D17-RITS-K621C	Control Room Vent.	Hi Alarm/Monitor	
D17-RITS-K621D	Control Room Vent.	Hi Alarm/Monitor	
D17-RITS-K621A	Control Room Vent. Hi Hi, Inop or Downscale	Both	Isolates CR and Starts CR Fresh Air Unit A TRIP FUNCTION
D17-RITS-K621D	Control Room Vent. Hi Hi, Inop or Downscale		
			TRIP SYSTEM
D17-RITS-K621B	Control Room Vent. Hi Hi, Inop or Downscale	Both	Isolates CR and Starts CR Fresh Air Unit B TRIP FUNCTION
D17-RITS-K621C	Control Room Vent. Hi Hi, Inop or Downscale		
			TRIP SYSTEM
D17-RITS-K609A	Containment & DW Exh	Hi Alarm/Monitor	
D17-RITS-K609B	Containment & DW Exh	Hi Alarm/Monitor	
D17-RITS-K609C	Containment & DW Exh	Hi Alarm/Monitor	
D17-RITS-K609D	Containment & DW Exh	Hi Alarm/Monitor	

*One channel (typical of 21 shown on this page)

DEFINITIONS FOR
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"
 FOR RADIATION MONITORING INSTRUMENTATION TABLE 3.3.7.1-1

<u>Trip Unit/ Monitor</u>	<u>Parameter</u>	<u>Logic Function</u>
D17-RITS-K609A	Containment & DW Exh. Hi Hi, Inop or Downscale	Both
D17-RITS-K609D	Containment & DW Exh. Hi Hi, Inop or Downscale	
TRIP SYSTEM		
D17-RITS-K609B	Containment & DW Exh. Hi Hi, Inop or Downscale	Both
D17-RITS-K609C	Containment & DW Exh. Hi Hi, Inop or Downscale	
TRIP SYSTEM		
D17-RITS-K617A	Fuel Handling Area Vent	(1) Hi Alarm/Monitor
D17-RITS-K617B	Fuel Handling Area Vent*	(1) Hi Alarm/Monitor
D17-RITS-K617C	Fuel Handling Area Vent	(1) Hi Alarm/Monitor
D17-RITS-K617D	Fuel Handling Area Vent	(1) Hi Alarm/Monitor
D17-RITS-K618A	Fuel Handling Area Pool Sweep Exh	(1) Hi Alarm/Monitor
D17-RITS-K618B	Fuel Handling Area Pool Sweep Exh	(1) Hi Alarm/Monitor
D17-RITS-K618C	Fuel Handling Area Pool Sweep Exh	(1) Hi Alarm/Monitor
D17-RITS-K618D	Fuel Handling Area Pool Sweep Exh	(1) Hi Alarm/Monitor
D21-RITS-K622	New Fuel Storage Vault	Hi Alarm/Monitor
D21-RITS-K623	New Fuel Storage Vault	Hi Alarm/Monitor
D21-RITS-K624	Spent Fuel Storage Pool	Hi Alarm/Monitor
D21-RITS-K625	Spent Fuel Storage Pool	Hi Alarm/Monitor
D21-RITS-K626	Dryer Storage Area	Hi Alarm/Monitor
D21-RITS-K600	Control Room	Hi Alarm/Monitor

*One channel (typical of 18 shown on this page)

(1) Logics, TRIP SYSTEMS, and TRIP FUNCTIONS are shown on page 46 of this attachment under Isolation Actuation Instrumentation Table 3.3.2-1.

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 202

Priority: 1A

Identified By _____ / _____
Date

Responsible Supervisor _____

Tech Spec Reference: Table 3.3.7.5-1.4

Problem Title: Post Accident Monitoring Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):
12 channels of suppression pool temp noted in FSAR Tech Spec only reference 6.
Should "6, 1/sector" be changed to "12, 1/sector" under "required number of
channels"?

2. Safety Significance: FSAR presently lists 12, Tech Specs could be non-
conservative in relation to FSAR.

3. Anticipated Resolution: Further investigation required; submit Tech
Spec Change if required.

could use Tech Spec position statement

4. NRC Response to Item (NRR/IE): _____

NRC Notified: _____ / _____
Individual Notified Date Time

5. Disposition: _____

Items Closed: (How) _____

_____ / _____
Date Time

cc: J. E. Cross
R. F. Rogers

Identified By _____

Date 1

Responsible Supervisor _____

Tech Spec Reference: Table 3.3.7.5-1.4

Problem Title: Post Accident Monitoring Instrumentation

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other): _____

12 channels of suppression pool temp noted in FSAR. Tech spec only references 6. Should "6, 1/chan sector" be changed to "12, 2/sector" under "required number of channels"

2. Safety Significance: ~~None~~ ^{FSAR} ~~FSAR~~

FSAR presently lists 12 Tech Specs could be more conservative in relation to FSAR

3. Anticipated Resolution: _____

further investigation required; submit Tech Spec change if required.

4. NRC Response to Item (NRR/IE): _____

NRC Notified: _____

Individual Notified _____

Date _____

Time _____

5. Disposition: _____

Items Closed: (How) _____

Date _____

Time _____

cc: J. E. Cross
R. F. Rogers

"TECH SPEC PRIORITY"

Punchlist Item # 202

Tech Spec 3/4.3.7

Priority 1A

TO: Manager of Nuclear Plant Engineering

FROM: Chairman, Prioritization and Disposition Chairman

SUBJECT: Technical Specifications Punchlist Item # 202

PDTS:84/ 0009


DATE: 3/10/84

The subject Tech Spec item has been determined by the Disposition Committee to require Engineering support.

DETAILS: Verify that the min. operable channels for Suppression Pool Temperature in table 3.3.7.5-1.4 should be "12, 2/sector" as recommended by Bechtel IAC review.

Please contact Joe Hendry at Extension 2670⁵ for further information.

Please refer to the Tech Spec Punchlist item number in your response. Forward your response to G. Zinke.


J. C. Roberts
Chairman

LLJ/JCR:swb

cc: Mr. C. L. Tyrone
Mr. J. E. Cross
Mr. D. Stonestreet
Mr. A. S. McCurdy
Mr. S. Hutchins
Mr. J. Hendry
File (Tech Spec Records)

A4/61swb1

"TECH SPEC PRIORITY"

Punchlist Item # See ATTACHED

Tech Spec See ATTACHED

Priority See ATTACHED

TO: Manager of Nuclear Plant Engineering
FROM: Chairman, Prioritization and Disposition Chairman
SUBJECT: Technical Specifications Punchlist Item # See ATTACHED

PDTS: 84/ 0014

DATE: 3/10/84

The subject Tech Spec item has been determined by the Disposition Committee to require Engineering support.

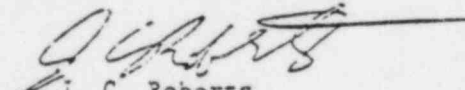
DETAILS: This letter identifies requested response dates for the following Tech Spec problems:

# 199 Letter No. PDTS 84/0001	# 205 Letter No. PDTS 84/0007
# 180 Letter No. PDTS 84/0002	# 198 Letter No. PDTS 84/0008
# 033 Letter No. PDTS 84/0003	# 202 Letter No. PDTS 84/0009
# 054 Letter No. PDTS 84/0004	# 213 Letter No. PDTS 84/0010
# 001 Letter No. PDTS 84/0005	# 219 Letter No. PDTS 84/0011
# 016 Letter No. PDTS 84/0006	# 118 Letter No. PDTS 84/0012

It is requested that the responses to the above items be completed by 3/13/84

Please contact Jerry Roberts at Extension 2695 for further information.

Please refer to the Tech Spec punchlist item number in your response. Forward your response to George ZINKE


J. C. Roberts
Chairman

LLJ/JCR:swb
cc: Mr. C. L. Tyrone
Mr. J. E. Cross
Mr. D. Stonestreet
Mr. A. S. McCurdy
Mr. S. Hutchins
Mr. J. Hendry
File (Tech Spec Records)

A4/61swb1

Tech Spec Problem No.Tech SpecsPriority

199	Table 3.3.6-1.5	1B
180	4.8.4.3	1D
033	Table 3.3.8-2	1B
054	3/4.3.8	1B
001	3/4.5.1	1B
016	3/4.3.8	1B
015	3/4.3.2	1D
198	3/4.3.7	1B
202	3/4.3.7	1B
213	3/4.3.3	1B
219	Figure 3.4.6.1-1	1B
168	3.6.3.1	1B

Item No. 11E

TECHNICAL SPECIFICATION PROBLEM SHEET

Priority _____

IE TEAM

12/24/84

Identified By

Date

Responsible Supervisor

Tech Spec Reference: 4.6.3.1.C (P 3¹/₄ 6-22)

Problem Title: SUPPRESSION POOL TEMPERATURE INSTRUMENTS (2/SECTION - 12 TOTAL)

SHOWN IN TABLE 4.6.3.1-1 ARE NOT SPECIFIED SEPARATE FR. P.A.

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other): TEMP COUPLERS.

SUPP. POOL HAS 12 INST. (TEMP) ASSOCIATED WITH THIS

SPEC (4.6.3.1.C) AND WHICH ARE REQUIRED TO

BE OPERABLE. THE INSTR. ²/₂ POST ACC. MONITORING T.S.

3.3.7.5. REQUIRES ON TABLE 3.3.7.5-1 THAT SIX

INST. - ONE/SECTION BE OPERABLE. IT IS CONFUSING

WHICH INST. ARE FOR WHICH SYSTEM. IT IS ALSO UNCLEAR

IF DOUBLE CREDIT CAN BE TAKEN FOR THEM (SHOULD

2. Safety Significance: BE ABLE TO).

↳ EDITORIAL / CLARITY

3. Anticipated Resolution: _____

4. NRC Response to Item (NRR/IE): _____

NRC Notified: _____

Individual Notified

Date

Time

5. Disposition: _____

Items Closed: (How) _____

Date

Time

cc: J. E. Cross
R. F. Rogers


"TECH SPEC PRIORITY"

MEMO TO: J. F. Pinto, Manager of Nuclear Plant Engineering
FROM: C. L. Tyrone, Project Manager
SUBJECT: Handling of Tech Spec Review Items
TSRO: 84/0001
DATE: March 11, 1984

This memorandum confirms our conversation of March 10, 1984. At that time, your assistance was requested in resolving discrepancies on eleven priority 1 items. Since then two items have been added. These items are all previously identified items which require early resolution with the NRC. A response is needed on these items by 12:30 PM on March 11, 1984. A list is attached.

Furthermore, all items of any priority identified (or previously known) which are being handled on this program require expeditious handling. This includes areas where requests are originated from other interfacing organizations such as the Plant or Nuclear Services. In any case where conflicts regarding highest priority is not clear, I am available to provide clarification.

It is suggested that you arrange 7 day a week support in this area as it is needed and arrange for all NPE personnel who will be involved in this effort to be available (or on call) in a manner that will support the Tech Spec Review program.


CLT

SHH:sad
Attachment

cc: J. B. Richard (w/a)
J. P. McGaughy (w/a)
J. F. Pinto (w/a)
J. E. Cross (w/a)
T. H. Cloninger (w/a)
H. J. Green (w/a)
R. C. Fron (w/a)
D. W. Stonestreet (w/a)

~~XXXXXXXXXXXXXXXXXXXX~~
T. E. Reaves, Jr. (w/a)
S. M. Feith (w/a)
J. G. Cesare (w/a)
G. W. Smith (w/a)
L. R. McKay (w/a)
L. C. Burgess (w/a)
File (Tech Spec Records) (w/a)

LIST OF CURRENT PRIORITY 1
ITEMS REQUIRING NPE SUPPORT

PDTS:84/	P/L #	Date Sent
001	199	3/10/84
002	180	3/10/84
003	033	3/10/84
004	054	3/10/84
005	001	3/10/84
006	016	3/10/84
007	015	3/10/84
008	198	3/10/84
009	202	3/10/84
010	213	3/10/84
011	219	3/10/84
012	083	3/10/84
013	168	3/10/84

DEFINITIONS FOR
 "CHANNELS", "TRIP SYSTEMS", AND "TRIP FUNCTIONS"
 FOR ACCIDENT MONITORING INSTRUMENTATION - TABLE 3.3.7.5-1

<u>Transmitter/Sensor</u>	<u>Parameter</u>	<u>Recorder</u>	<u>Function</u>
B21-PT-N062A B21-LT-N091A	Reactor Vessel Pressure Reactor Vessel Level	B21-UR-R623A	Monitor
B21-PT-N062B B21-LT-N091B	Reactor Vessel Pressure Reactor Vessel Level	B21-UR-R623B	Monitor
E30-LT-N003C E30-LT-N003D	Supp. Pool Water Level Supp. Pool Water Level	E30-LR-R600A E30-LR-R600B	Monitor Monitor
(a) M71-TE-N012A(B) (a) M71-TE-N022A(B) (a) M71-TE-N023A(B)	Supp. Pool Temp. 40° Supp. Pool Temp. 82° Supp. Pool Temp. 142°	M71-TR-R605A(B)	Monitor
(a) M71-TE-N024A(B) (a) M71-TE-N025A(B) (a) M71-TE-N026A(B)	Supp. Pool Temp. 180° Supp. Pool Temp. 262° Supp. Pool Temp. 318°	M71-TR-R605C(D)	Monitor
E61-PDT-N014A	Ctmt/DW Diff. Pressure *	E61-PDR-R601A	Monitor
E61-PDT-N014B	Ctmt/DW Diff. Pressure	E61-PDR-R601B	Monitor
M71-PDT-N001A M71-PDT-N001B	Drywell Pressure Drywell Pressure	M71-PDR-R601A M71-PDR-R601B	Monitor Monitor
** M71-TE-N013A(B) ** M71-TE-N008A(B)	CRD Cavity Temp. Drywell Temp. - 166' el.	M71-TR-R602A(B)	Monitor
** M71-TE-N013C(D) ** M71-TE-N008C(D)	CRD Cavity Temp. Drywell Temp. - 166' el.	M71-TR-R603A(B)	Monitor
E61-AITS-K001A(B) E61-AITS-K002A(B)	Ctmt H ₂ Conc. Analyzer DW H ₂ Conc. Analyzer	E61-AR-R602A(B)	Monitor
M71-PDT-N002A M71-PDT-N002B M71-PDT-N027A M71-PDT-N027B	Ctmt Narrow Range Pressure Ctmt Narrow Range Pressure Ctmt Wide Range Pressure Ctmt Wide Range Pressure	M71-PDR-R601A M71-PDR-R601B M71-PDR-R601A M71-PDR-R601B	Monitor Monitor Monitor Monitor
** M71-TE-N007A(B) ** M71-TE-N007C(D)	Ctmt Temp. - 139' el. Ctmt Temp. - 139' el.	M71-TR-R602A(B) M71-TR-R603A(B)	Monitor Monitor

* One Channel (typical of 28 shown on this page)

TECHNICAL SPECIFICATION PROBLEM SHEET

Item Number: 219

Priority: BIA

Identified By _____ / _____
Date

Responsible Supervisor _____

Tech Spec Reference: Figure 3.4.6.1-1

Problem Title: Pressure/Temperature Limit Curves

1. Problem Description (Tech Spec, FSAR, SER, GE Design, Other):

The pressure-temperature limit curves do not comply with the closure flame pressure-temperature safety margins in Program IV.A.2 of App. G to 10CFR50. Proof and review comment from MEB, December 15, 1983.

2. Safety Significance:

3. Anticipated Resolution:

Evaluate to determine if the figure will require revision.

4. NRC Response to Item (NRR/IE): no action required till

NRC Notified: _____

Individual Notified _____ Date _____ Time _____

5. Disposition: _____

Items Closed: (How) _____

_____ / _____
Date Time

J. E. _____

F. P. _____