



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

K. Seyfrit
File
SALP

AUG 04 1983

MEMORANDUM FOR: Richard L. Spessard, Director
Division of Projects and Residence Programs
NRC Region III

FROM: Karl V. Seyfrit, Chief
Reactor Operations Analysis Branch
Office for Analysis and Evaluation
of Operational Data

SUBJECT: EVALUATION OF MONTICELLO LERs FOR THE PERIOD
JULY 1, 1982 TO JUNE 30, 1983

The Office for Analysis and Evaluation of Operational Data has assessed the Licensee Event Reports (LERs) submitted under Docket No. 50-263 during the subject period. This has been done in support of the ongoing SALP review of the Northern States Power Company, with regard to their performance as licensee of the Monticello Nuclear Generating Plant. Our perspective would be indicative of that of a BWR system safety engineer who, although knowledgeable, is not intimately familiar with the detailed site - specific equipment arrangements and operations. Our review focused on the technical accuracy, completeness, and intelligibility of the LERs.

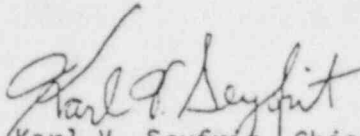
The majority of the LERs submitted were adequate in all important respects with few exceptions. The LERs typically provided clear descriptions of the cause and nature of the events as well as adequate explanations of the effects on both system function and public safety. In some LERs supplemental information was provided in attachments to the LER forms. This enabled the LER reviewer to better understand the nature of the events encountered, thereby facilitating evaluation of the safety significance of the event. In most cases the described corrective actions taken or planned by the licensee were considered to be commensurate with the nature, seriousness and frequency of the problems found. The enclosure provides additional observations from our review of the LERs.

In summary, our review of the licensee's LERs indicates that in most cases the licensee provided an adequate description of the events. However, the licensee did not provide adequate information in some of the LERs as mentioned above.

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If you have any questions please contact either myself or Sal Salah (492-4432) of my staff.



Karl V. Seyfrit, Chief
Reactor Operations Analysis Branch
Office for Analysis and Evaluation
of Operational Data

Attachments:

As stated

cc: H. Nicholaras, NRR
C. Brown, SRI
C. Heltemes, AEOD

SALP REVIEW FOR MONTICELLO

The licensee submitted eighteen LERs in the assessment period from July 1, 1982 to June 30, 1983. Our review included the following LER numbers:

82-007 through 82-013
83-001 through 83-010

The LER review covered the following subjects and the general instructions of NUREG-0161. The SALP review is presented with the topic reviewed followed by comments on that topic.

1. Review of LERs for Completeness

- a) Is the information sufficient to provide a good understanding of the event?

We found the information in the narrative sections and the included attachments to be exceptionally informative except for one LER. One LER (82-12) mentioned local leak rate testing for A0-2379, A0-2380, A0-2378, A0-2396 and A0-2387 but did not say where these components are located or what these components are.

- b) Review of Coded Information

We have checked the codes the licensee selected against the narrative description of the event for accuracy for every coded field. We agreed with the licensee in every coded block.

- c) Do the reports contain supplementary information when needed?

Some reports contained supplementary information as a separate attachment to the LER forms.

- d) Followup Reports

The licensee promised a final report of the event without a completion date or where report will be reported for LER 82-13-1.

e) Were similar occurrences properly referenced?

Yes, in the review of the Monticello LERs the following similar events were reported:

- (i) There were five LERs (82-11, 82-13-10, 82-13-12, 82-17 and 83-10) with one previous similar event.
- (ii) There was one LER (83-01) with two previous similar events.
- (iii) There was one LER (82-12) with three previous similar events.

2. Is component failure or other appropriate information being reported to NPRDS?

The licensee reported 15 out of the 18 LERs to NPRDS.

3. Multiple Event Reporting in a Single LER

The licensee did not report multiple events.

4. Relationship Between PNs and LERs.

5 PNs and two updates to these PNs were issued in the assessed period.

5 PNs dealt with:

- a. Reactor placed in hot shutdown to investigate and repair steam leak.
- b. Reactor placed in cold shutdown after turbine developed high vibration.
- c. Maintenance worker suffered second degree burn from contaminated steam.
- d. Ultrasonic testing of recirculation cooling system piping identified cracks.

e. Reactor taken to cold shutdown using turbine bypass valves.

None of these PNs were reported in the LERs in the same period; however, item (d) should probably have been reported as an LER.



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AUG 3 1983

MEMORANDUM FOR: James E. Gagliardo, Acting Director
Division of Resident Reactor Projects
and Engineering Programs
Region IV

FROM: Karl V. Seyfrit, Chief
Reactor Operations Analysis Branch
Office for Analysis and Evaluation
of Operational Data

SUBJECT: EVALUATION OF LERs FOR ARKANSAS UNIT 1 FOR THE PERIOD
FROM JULY 1, 1982 TO JUNE 30, 1983 - AEOD INPUT TO SALP
REVIEW

In support of the ongoing SALP reviews, AEOD has reviewed the LERs for Arkansas-1. AEOD's review focused on the accuracy and completeness of the individual licensee event reports. The review included LERs for Arkansas-1 which were written between July 1, 1982 and June 30, 1983. The licensee submitted about 36 LERs during the report period. In general, the LERs were consistent and reasonably detailed to permit understanding the events.

The enclosure provides observations from our review of the LERs.

If you have any questions regarding this matter, please contact myself or Medhat El-Zeftawy. (301) 492-4434

Karl V. Seyfrit
Karl V. Seyfrit, Chief
Reactor Operations Analysis Branch
Office for Analysis and Evaluation
of Operational Data

Enclosure:
As stated

cc: w/enclosure
G. Vissing, NRR

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SALP REVIEW FOR ARKANSAS-1

The review and comment on LERs covered the following subject matter:

1. Review of LERs for completeness:

- a) The information in the narrative sections was generally sufficient to provide the reader with a good understanding of the event.
- b) There are no significant problems with the coded information provided by the licensee.
- c) The licensee submitted five 10-day reports in the assessment period. Of these LER 82-032 did not provide the supplemental information required to explain the tube defects in the "A" once through steam generator (OTSG). Thirty-one 30-day reports were submitted with several of them containing additional supplemental information to provide a clearer understanding of the event. Two LERs 83-002/03L and 83-007/03L were revised for the same events they were originally written for and resubmitted using Nos. 83-002/03X and 83-007/03X, respectively.
- d) In many instances when the licensee promised a follow-up report he did. However, for LER #82-032/01T-0 the follow-up report was not received during the assessment period.
- e) In many cases the licensee referenced LERs pertaining to previous events of a similar nature. For example, in LER Nos. 83-006, other LERs 74-008, 76-003, 83-003, 83-004 and 83-007 were referenced. For many LERs, the licensee stated that there had been no similar previous occurrences.

2. Is component failure or other appropriate information being reported to NPRDS?

For most LERs reviewed the licensee has reported the failures to NPRDS.

3. Multiple event reporting in a single LER.

Multiple events of component failures, occurring within the reporting period of the first such event were reported correctly in accordance with NUREG-0161 (general instruction #7) e.g. LERs 83-002 and 83-003.

4. Relationship between PNs and LERs.

Thirteen PNs were issued in the SALP assessment period. Only PNs (PNO-IV-82-029 and 83-020) were further documented by LERs (82-020/03L, 82-021/03L, and 83-014/03L).

The following PNs reported with significant events that were not further documented by LERs:

PNO-IV-83-012

Undervoltage trip mechanism of ac reactor trip breaker A failed.

PNO-IV-82-050

While in refueling outage, special inspection found cracked thermal shield bolts. During replacement of bolts, bolt heads found separated from bolts.

March 24, 1983

PRELIMINARY NOTIFICATION OF EVENT OR UNUSUAL OCCURRENCE -- PNO-IV-83-12

This preliminary notification constitutes EARLY notice of events of POSSIBLE safety or public interest significance. The information is as initially received without verification or evaluation, and is basically all that is known by NRC staff on this date.

FACILITY: Arkansas Power & Light Co.
Arkansas Nuclear One, Unit 1
Docket No: 50-313
Russellville, Arkansas

Licensee Emergency Classification:
 Notification of Unusual Event
 Alert
 Site Area Emergency
 General Emergency
 Not Applicable

SUBJECT: REACTOR TRIP BREAKER FAILURE

On March 23, 1983, following completion of the control rod drive mechanism patch verification test, the licensee deenergized the CRDM's by performing a manual reactor trip from the control room. The 'A' AC reactor trip breaker failed to trip, indicating a failure of the undervoltage trip mechanism. The breaker was subsequently tripped mechanically. The breaker is a General Electric type AK-2A. It had been successfully tested on 3/14/83 in response to IEB 83-04.

The ANO-1 reactor protection system and manual trip design open the reactor trip breakers using an undervoltage trip mechanism only. The shunt trip mechanism is not safety-related and may be activated only at the breaker location.

Vendor representatives will go to the site to assist in the investigation. The unit is in cold shutdown, making preparation for heatup following a refueling outage.

No media interest has occurred concerning the reactor trip breakers at ANO. The licensee does not plan a news release, but will respond to inquiries. The NRC does not plan a news release.

This information is current as of 10:00 a.m. on March 24, 1983.

CONTACT: W. D. Johnson, 728-8155; L. J. Callan, (501)968-3290

DISTRIBUTION:

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Licensee:
(Reactor Licensees)

IE 34

December 21, 1982

PRELIMINARY NOTIFICATION OF EVENT OR UNUSUAL OCCURRENCE -- PNO-IV-82-50

This preliminary notification constitutes EARLY notice of events of POSSIBLE safety or public interest significance. The information is as initially received without verification or evaluation, and is basically all that is known by NRC staff on this date.

FACILITY: Arkansas Nuclear One, Unit 1
Russellville, Arkansas
DN 50-313

Licensee Emergency Classification:
___ Notification of Unusual Event
___ Alert
___ Site Area Emergency
___ General Emergency
X Not Applicable

SUBJECT: THERMAL SHIELD BOLT FAILURE

Arkansas Nuclear One, Unit 1 is presently in a refueling outage. A special inspection of the thermal shield bolts was conducted as a result of the Ocone 1 experience of July 1981. Visual inspection of the 96 thermal shield bolts revealed no problem. UT inspection, however, identified 51 bolts with indications of cracking. During replacement 48 bolt heads were found to be completely separated from the bolts. There were no missing pieces.

The licensee plans to replace all bolts with acceptable fasteners as identified by B&W.

Neither Region IV nor the licensee plan a press release.

Region IV was initially notified of the condition by the SRI on December 20, 1982. This information is current as of 11:00 a.m. (CST), December 21, 1982.

CONTACT: W. D. Johnson, 728-8155; L. J. Callan, (501)968-3290

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