



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
101 MARIETTA STREET, N.W.  
ATLANTA, GEORGIA 30323

ENCLOSURE 1

EXAMINATION REPORT - 50-327/92-301

Facility Licensee: Tennessee Valley Authority

Facility Name: Sequoyah Nuclear Plant

Facility Docket Nos.: 50-327 and 50-328

Facility License Nos.: DPR-77 and DPR-79

The NRC administered examinations at the Sequoyah Nuclear Plant near Soddy-Daisy, Tennessee.

Chief Examiner:

Michael E. Ernstes  
Michael E. Ernstes

7/7/92  
Date Signed

Approved By:

Lawrence L. Lawyer  
Lawrence L. Lawyer, Chief  
Operator Licensing Section 1  
Division of Reactor Safety

7/9/92  
Date Signed

SUMMARY

The examination team conducted requalification retake examinations on July 16, 1992 for three individuals who failed the requalification examination given in December of 1991. There was no program evaluation made due to the small sample size.

The team administered examinations to three Senior Reactor Operators (SROs) in the areas that the operators had previously failed. All individuals passed the examinations.

The NRC reviewed the examination proposed by the facility and found it to be acceptable. The NRC examiners replaced several job performance measure (JPM) system questions with other facility generated questions that were deemed more acceptable. The facility and the NRC generated one new JPM as a replacement.

## REPORT DETAILS

### 1. Facility Employees Attending Exit

D. Ashley, Operations Training Manager  
R. Beecken, Plant Manager  
R. Thompson, Licensing Compliance Manager  
T. Lundy, Simulator Engineer  
M. Cooper, Site Licensing Manager  
R. King, Manager, Operations Training  
V. Keyser, Instructor  
S. Michael, Manager, Simulator Support  
M. Sheperd, Manager, Nuclear Training  
J. Wilson, Site Vice President

### 2. NRC Personnel Attending Exit

\*M. Ernstes, License Examiner, DRS  
S. Cahill, License Examiner, DRS  
W. Holland, Senior Resident Inspector  
S. Sparks, Resident Inspector

\*Chief Examiner

### 3. Discussion

#### a. Examination Development

The facility proposed a walk-through examination consisting of ten JPMs with questions which the NRC reviewed and found acceptable. The examination covered a broad range of topics and the examiners found the JPM format to be very easy to follow and administer. However, several of the system questions did not meet the NRC guidelines for open-reference questions due to being direct lookups or overly simplistic. These were replaced by extra questions that the facility submitted for each JPM.

The facility and the NRC jointly developed a single JPM to administer to one operator for an Emergency Actuation Level (EAL) classification task. The JPM originally proposed for this task was one of only four of this type in the facility bank. This raised concerns about the candidate's familiarity with the test material. The support of the facility representatives to the NRC in correcting this inadequacy was excellent.

The facility proposed a written examination consisting of a Part A static simulator portion and Part B administrative limits and controls portion which was reviewed and found acceptable for use by the NRC. No changes were necessary.

b. Examination Administration

The facility administration of the examination was smooth with no significant problems. Facility evaluator performance on the JPM walk-through portion was excellent, particularly in the area of ensuring that the candidate determine the completion point for each task.

The NRC identified a problem with Procedure O-SO-202-4, Section 8.1.1, which directed the transfer of a 6.9 kV electrical bus. The procedure directed the operator to select the 1A-A voltmeter to the "BUS VOLTAGE" position although this position does not exist. The facility evaluator initiated appropriate corrective actions.

The NRC examiners noted another problem with magnetic information plates used with the hydrogen recombiner panels to record setpoint data. These plates also had labelling on them which served an identification function to differentiate between trains A and B. These plates had been accidentally switched between trains on the simulator which caused confusion due to the disagreement with the train identification on the permanently mounted label plates. The NRC found similar magnetic plates being used in the Control Room on various components. The facility took corrective action to permanently mount these plates. The examiners relayed their concern over the use of these type of labels on other equipment to the NRC Resident Inspector.

4. Exit Meeting

At the conclusion of the site visit, the examiners met with representatives of the plant staff to discuss the results of the examinations.

The licensee did not identify as proprietary any material provided to or reviewed by the examiners.

July 1992

SRO Part A

#2

**QUESTION**

Which ONE of the following signals/conditions has caused the closure of the Main Steam Isolation Valves (MSIVs)?

- A. Low Steam Header Pressure.
- B. Hi-Hi Containment Pressure
- C. High Negative Rate on Steamline Pressure.
- D. Loss of Non-Essential Control Air

**ANSWER** A. Increasing/Decreasing Distractors N

- A. Low Steam Header Pressure.

**REFERENCE**

37WELL-1-1 (R 2)  
Precautions, Limitations & Setpoints (Rev 9)

Total Points: 1.00 Estimated Completion Time: 3 SRO Only: N

**K/A Reference:** Task: 30100106  
039000 K4.00 (3.7 - 3.7)

**Lesson Plan/Objective:** OPL2739041 / B.5 (90 Regual WK 2)

**QUESTION**

If RCV-62-85 were closed by operator action, which ONE of the following describes the systems response?

- A. CCP discharge pressure would increase but flow to the RCS would remain constant.
- B. CCP discharge pressure would increase and flow to the RCS would decrease.
- C. CCP discharge pressure would remain constant but flow to the RCS would decrease.
- D. CCP discharge pressure would remain constant and flow to the RCS would remain constant.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. CCP discharge pressure would remain constant and flow to the RCS would remain constant.

**REFERENCE**

Simulator Configuration  
47W809-1  
47W811-1

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

**K/A Reference:**

004020 A1.07 (3.0 - 3.0)  
006000 K1.08 (3.6 - 3.9)

Task: 00401501

Lesson Plan/Objective: OPL2739063 / B.2 (90 Req Wk 4)

**QUESTION**

Assuming greater than one minute has elapsed since the SI was initiated.

Which ONE of the following describes the anticipated response?

- A. The SI may be reset but NOT terminated.
- B. The SI may be reset and terminated.
- C. The SI may NOT be reset or terminated.
- D. The SI may NOT be reset but may be terminated.

**ANSWER** A. Increasing/Decreasing Distractors N

- A. The SI may be reset but NOT terminated.

**REFERENCE**

E-1 (R10), Step 9

Total Points: 1.00      Estimated Completion Time: 3      SRO Only: N

**K/A Reference:**

**Task:** 30100106

- 006000 A1.06 (3.6 - 3.9)
- 006000 K6.1b (3.5 - 3.9)
- 006050 A1.02 (4.0 - 4.4)

**Lesson Plan/Objective:** OPL2739014 / B.2.y (90 Req Wk 1 Sim)

**QUESTION**

If you needed to isolate the Cold Leg Accumulators, which ONE of the following actions would be required to close the accumulator isolation valves?

- A. Reset the Safety Injection.
- B. Reset the Safety Injection and restore power to the valves.
- C. Block P-11 Low Pressure SI and restore power to the valves.
- D. Block P-11 Low Pressure SI.

**ANSWER** B. Increasing/Decreasing Distractors N

- B. Reset the Safety Injection and restore power to the valves.

**REFERENCE**

47W611-63-6

Total Points: 1.00      Estimated Completion Time: 3      SRO Only: N

**K/A Reference:**

**Task:** 00600501

006000 K4.11 (3.0 - 3.1)  
006000 K4.13 (4.1 - 4.3)

**Lesson Plan/Objective:** OPL2739060 / B.2.a (90 Requal Wk 4)



**QUESTION**

If 6.9kV Unit Bd 1C were to trip and lockout on a differential relay actuation, which ONE of the following would describe the response of the 6.9 Kv Shutdown Bd 1B-B?

- A. Auto transfer to CSST C.
- B. Auto transfer to CSST A.
- C. Trip and remain de-energized.
- D. Transfer to D/G 1-B.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. Transfer to D/G 1-B.

**REFERENCE**

45N765-1 & 2

Total Points: 1.00      Estimated Completion Time: 3      SRO Only: N

**K/A Reference:**

064000 K1.01 (4.1 - 4.4)

**Task:** 06400401

**Lesson Plan/Objective:** OPL2739071 / B.11 (90 Requal Wk 4)

**QUESTION**

WHICH ONE of the following describes the plant response to this failure if PT-68-340 failed high?

- A. PZR PORV PCV-68-340 would open causing depressurization of the RCS.
- B. PZR Spray Valves would open causing depressurization of the RCS.
- C. PZR PORV PCV-68-334 would open causing depressurization of the RCS.
- D. PZR Spray valves would open with no effect on RCS pressure.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. PZR Spray valves would open with no effect on RCS pressure.

**REFERENCE**

47W611-68- 3 (R0)

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

**K/A Reference:**

000027 EA2.15 (3.7 - 4.0)

**Task:** 01000701

**Lesson Plan/Objective:** OPL2738902/ B.2.c (89 Req Wk 1)  
OPL2739143/ B.7 (91 Req Wk 4)

**QUESTION**

The motor-driven auxiliary feedwater pumps received an automatic start signal. WHICH ONE of the following actions must the operator perform to take manual control of the flow from the motor driven AFW pumps to steam generators #1 and #2?

- A. Stop motor driven pumps 1A-A and 1B-B, place 1-HS-3-164 and 1-HS-3-156 in manual, then restart the pumps.
- B. Reset the SI signal and place 1-HS-3-164 and 1-HS-3-156 in manual bypass.
- C. Reset the SI signal, place 1-HS-3-164 and 1-HS-3-156 to accident reset and then to manual.
- D. Place 1-HS-3-164 and 1-HS-3-156 to accident reset and then place each in manual bypass.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. Place 1-HS-3-164 and 1-HS-3-156 to accident reset and then place each in manual bypass.

**REFERENCE**

SQN Drawing 47W611-3-4 (R4)

Total Points: 1.00 Estimated Completion Time: 3 SRO Only: N

**K/A Reference:**

Task: 06100901

061000 G7 (3.6 - 3.7)  
061000 G9 (3.8 - 3.9)

**Lesson Plan/Objective:** OPL2738965 / B.1 (89 Requal Wk 5)  
\* 1991 Simulator LP on Yokogawa Controllers.

**QUESTION**

WHICH ONE of the following operator actions would be required to shutdown the ABGTS fans?

- A. Reset Phase A, Hi Rad & ABI and place the ABGTS fans to stop.
- B. Reset Phase A and place the ABGTS fans to stop.
- C. Reset SI, Hi Rad and ABI and place the ABGTS fans to stop.
- D. Reset SI, reset Hi Rad and place the ABGTS fans to stop.

**ANSWER** A. Increasing/Decreasing Distractors N

- A. Reset Phase A, Hi Rad & ABI and place the ABGTS fans to stop.

**REFERENCE**

47W611-30-6 (R3)  
0-SO-30-18 (R1)

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

**K/A Reference:** Task: 08809901  
013000 K4.10 (3.3 - 3.7)

**Lesson Plan/Objective:** OPL2738943 / B.4 (89 Requal Wk 3)

**QUESTION**

How much additional time must elapse until the RHR/Containment Sump swapover will automatically occur? (Assume adequate containment sump level will be available.)

- A. 3.3 hours
- B. 3.9 hours
- C. 4.2 hours
- D. 4.8 hours

**ANSWER** A. Increasing/Decreasing Distractors Y

A. 3.3 hours

SIS flow = 470 + 460 gpm & CCPIT flow = 320 gpm for total of  $\approx$  1250 gpm.

Current RWST level 95% with level at swapover 27%.

|     |             |
|-----|-------------|
| 95% | 360,000 gal |
| 27% | 112,500 gal |
|     | -----       |
|     | 247,500 gal |

247,500 gals / 1250 gpm = 198 mins = 3.3 hrs

95% - 27% = 68%

380,000 gal/100% = 3800 gal/%

|                  |   |                    |
|------------------|---|--------------------|
| 68% x 3800 gal/% | = | 206 min. = 3.4 hrs |
| -----            |   |                    |
| 1250 gpm         |   |                    |

**REFERENCE**

TI-28, (R78) Figure C.11

ES-1.3 (R4) Step 4

Total Points: 1.00      Estimated Completion Time: \*\*\*      SRO Only: N

**K/A Reference:**

**Task:** 00601501

006000 K4.09 (3.8 - 4.1)

006020 A1.08 (3.5 - 3.8)

006030 A1.02 (4.2 - 4.3)

**Lesson Plan/Objective:** OPL2739060/ (90 Req Wk 4)

**QUESTION**

[NOT RELATED TO SCENARIO]

WHICH ONE of the following describes the response of the pressurizer heaters if a Safety Injection Signal were initiated and then RESET after the 60 second time delay had elapsed?

- A. Pressurizer Backup Heater Groups 1A-A, 1B-B and 1C would automatically energize, Control Group 1D would have to be manually energized.
- B. Pressurizer Backup Heater Groups 1A-A and 1B-B would automatically energize, Backup Heater Group 1C and Control Group 1D would have to be manually energized.
- C. Pressurizer Backup Group 1C would automatically energize, Backup Heater Groups 1A-A, 1B-B and Control Group 1D would have to be manually energized.
- D. All Pressurizer Heater Groups would have to be manually energized.

**ANSWER** C. Increasing/Decreasing Distractors N

- C. Pressurizer Backup Group 1C would automatically energize, Backup Heater Groups 1A-A, 1B-B and Control Group 1D would have to be manually energized.

**REFERENCE**

1,2-45N765-11 (R0)

Total Points: 1.00      Estimated Completion Time: 4      SRO Only: N

**K/A Reference:**

010000 K6.03 (3.2/3.6)

**Task:** 01000101**Lesson Plan/Objective:** OPL

**QUESTION**

[NOT RELATED TO SCENARIO]

WHICH ONE of the following will help prevent Pressurizer surge line thermal stratification while heating up the RCS?

- A. Maintain pressurizer saturation temperature as LOW as possible and Loop 2 Hot Leg temperature as LOW as possible.
- B. Maintain pressurizer saturation temperature as LOW as possible and Loop 2 Hot Leg temperature as HIGH as possible.
- C. Maintain pressurizer saturation temperature as HIGH as possible and Loop 2 Hot Leg temperature as LOW as possible.
- D. Maintain pressurizer saturation temperature as HIGH as possible and Loop 2 Hot Leg temperature as HIGH as possible.

**ANSWER** B. Increasing/Decreasing Distractors N

- B. Maintain pressurizer saturation temperature as LOW as possible and Loop 2 Hot Leg temperature as HIGH as possible.

**REFERENCE**

GOI-1, Heatup from Cold to Hot Shutdown, V.A.17.a (Rev 91)  
47W813-1 (R 12)

Total Points: 1.00 Estimated Completion Time: 0 SRO Only: N

**K/A Reference:**

010000 GK5 (3.2/3.8)

**Task:**

**Lesson Plan/Objective:** OPL2739147/ B.3 (91 Req Wk 4)

**QUESTION**

[NOT RELATED TO SCENARIO]

Pressurizer level transmitter LT-68-320 has been INOPERABLE for 2 days and was removed from service per 1-SI-IXX-099-483.0. You are notified that pressurizer level transmitter LT-68-335 just failed low. The unit is operating at 100% power.

1. What MODE is the plant required to be placed in?  
[0.5 pts]
2. What is the maximum amount or time you are allowed to place the unit in the required MODE?  
[0.5 pts]

**ANSWER**

- a. MODE 3 (0.5 pts)
- b. 7 hours [1 hr. action to restore, 6 hrs. to reach HSB]  
(0.5 pts)

**REFERENCE**

Tech Spec LCO 3.3.1.1, action 6  
Tech Spec LCO 3.0.3, action

Total Points: 1.00      Estimated Completion Time: 0      SRO Only: N

**K/A Reference:**

194001 A1.02 (4.1/3.9)

**Task:** 11901503

**Lesson Plan/Objective:** OPL2739044/ (90 Req Wk 3)  
Simulator Review of 3.0.3 & 3.0.4



**QUESTION**

[NOT RELATED TO SCENARIO]

WHICH ONE of the following statements is correct concerning the ERCW pumps if the 6.9KV Shutdown Boards were de-energized, after an SI was actuated, causing the D/Gs to start and energize the shutdown boards?

- A. The pumps that were running prior to the SI would be running.
- B. Only 4 ERCW pumps would be running.
- C. The pumps that were running prior to the blackout will be running.
- D. All of the pumps would be running.

**ANSWER** B. Increasing/Decreasing Di:

Only 4 ERCW pumps would be running.

**REFERENCE**

45N765-15 (R11)

Total Points: 1.00      Estimated Completion Time: 10      SRO Only: N

**K/A Reference:**

Task: 06400401

000050 BK1.01 (3.5 - 3.9)

000050 BK1.07 (3.2 - 3.2)

**Lesson Plan/Objective:** OPI.2739071 / B.13 (90 Requal Wk 4)

July 1992  
SRO Part A  
#6

**QUESTION**

Annunciator XA-55-5C Window 3 is in, which ONE of the following actions is required?

- A. Restore the coolers to OPERABLE status within 7 days or be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- B. Restore the coolers to OPERABLE status within 72 hours or be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- C. Restore the coolers to OPERABLE status within 1 hour or be in HOT STANDBY in the next 6 hours, HOT SHUTDOWN in the following 6 hours and COLD SHUTDOWN in the following 24 hours.
- D. No action required, all coolers are in service.

**ANSWER** C. Increasing/Decreasing Distractors N

- C. Restore the coolers to OPERABLE status within 1 hour or be in HOT STANDBY in the next 6 hours, HOT SHUTDOWN in the following 6 hours and COLD SHUTDOWN in the following 24 hours.

**REFERENCE**

Tech Spec 3/4.6.2 (R160) P. 3/1 6-16b.  
1-AR-M5-C (LC)

Total Points: 1.00 Estimated Completion Time: 5 SRO Only: N

**K/A Reference:** Task: 02200401  
022000 G5 (J.C - 3.7)

**Lesson Plan/Objective:** OPL273C202 / V.A (88 Regual)

**QUESTION**

During review of SI-128.1 for 1A-A RHR pump, it was determined that the pump discharge pressure had been incorrectly logged. It was originally logged as 170 psig, but was actually 160 psig. Which ONE of the following describes the most limiting required Tech Spec actions?

- A. No action required, pump discharge pressure is within limits.
- B. Restore the pump to OPERABLE status in 72 hours or be in HOT STANDBY in 6 hours, HOT SHUTDOWN in the next 6 hours and COLD SHUTDOWN in the subsequent 24 hours
- C. Restore the pump to OPERABLE status in 2 hours or be in HOT STANDBY in 6 hours, HOT SHUTDOWN in the following 6 hours and COLD SHUTDOWN in the subsequent 24 hours.
- D. Restore the pump to OPERABLE status in 1 hour or be in HOT STANDBY in 6 hours, HOT SHUTDOWN in the following 6 hours and COLD SHUTDOWN in the subsequent 24 hours.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. Restore the pump to OPERABLE status in 1 hour or be in HOT STANDBY in 6 hours, HOT SHUTDOWN in the following 6 hours and COLD SHUTDOWN in the subsequent 24 hours.

**REFERENCE**

TS 3.0.3 (R160)  
TS 3.5.2 (R160)  
TS 3 0.5 (R160)

Total Points: 1.00      Estimated Completion Time: 4      SRO Only: Y

**K/A Reference:**

006000 G5 (3.5 - 4.2)  
006000 G11 (3.6 - 4.2)

Task: 19401503

**Lesson Plan/Objective:** OPL2739060 / B.3.b (90 Requir 1 Wk 4)

**QUESTION**

If TE-68-60 failed low with 1-HS-68-340AD and 1-HS-68-334D (COPS ARM/BLOCK PSS) in the "ARM" position, which ONE of the following describes the plant response?

- A. PORV-68-340 would open, PORV-68-334 would remain closed.
- B. PORV-68-340 would remain closed, PORV-68-334 would open.
- C. PORV-68-340 would open, PORV-68-334 would open.
- D. PORV-68-340 would remain closed, PORV-68-334 would remain closed.

**ANSWER** B. Increasing/Decreasing Distractors N

- B. PORV-68-340 would remain closed, PORV-68-334 would open.

**REFERENCE**

47W611-68-3 (R0)

Total Points: 1.00      Estimated Completion Time: 6      SRO Only: N

**K/A Reference:**      **Task:** 01001301  
000022 EK3.03 (3.7 - 4. )

**Lesson Plan/Objective:** OPL27389143/ B.4 (91 Req Wk 4)

**QUESTION**

If a fire were to occur on the Unit 1 Main Turbine Oil Tank, which one of the following describes how the deluge valve, FCV-26-85, would be opened?

- A. FSV-26-85 would energize by fire protection zone 388 sensing a high temperature condition.
- B. FSV-26-85 would energize by placing HS-26-85A to the "ON" position.
- C. 1-26-587 would be manually aligned to the "drain" position.
- D. 1-26-586 would be manually aligned to the "closed" position.

**ANSWER** C. Increasing/Decreasing Distractors N

- C. 1-26-587 would be manually aligned to the "drain" position.

**REFERENCE**

47W850-1 (R10)  
45N626-1 (R8)

Total Points: 1.00 Estimated Completion Time: 0 SRO Only: N

**K/A Reference:** Task: 08600501  
086000 A4.05 (3.0/3.5)

**Lesson Plan/Objective:**

**QUESTION**

If the 480V Shutdown Board 1A1-A were to become deenergized at this time, which ONE of the following is the most limiting action that is applicable?

- A. Restore the affected equipment to OPERABLE status within 2 hours or be in HOT STANDBY within the following 6 hours.
- B. Restore the affected equipment to OPERABLE status immediately or establish CONTAINMENT INTEGRITY within 8 hours.
- C. Restore the affected equipment to OPERABLE status within 8 hours or be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- D. Restore the affected equipment to OPERABLE status within 1 hour or be in HOT STANDBY within the following 6 hours and in HOT SHUTDOWN within the subsequent 6 hours.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. Restore the affected equipment to OPERABLE status within 1 hour or be in HOT STANDBY within the following 6 hours and in HOT SHUTDOWN within the subsequent 6 hours.

Answer based on power unavailable to FCV-63-73 and FCV-63-72, 3.5.2 requires auto swapover capability to sump. Also > 1 battery bd. inop under LCO 3.8.2.3.

**REFERENCE**

TS LCO 3.8.2.3. & 3.0.3. (R160)  
TS LCO 3.8.2.1 action b. (R160)  
45N700-1 (R4)  
TS LCO 3.5.2 (R160)

Total Points: 1.00 Estimated Completion Time: 5 SPO Only: N

**K/A Reference:**

Task: 19401503

006000 G5 (3.5 - 4.2)  
006000 G11 (3.6 - 4.2)

Lesson Plan/Objective: OPL2739060 / B.3.b (90 Requal Wk 4)

**QUESTION**

What would happen to control bank D if the rod control mode selector were placed in the "automatic" position?

- A. Move in at a speed higher than currently indicated.
- B. Move in at a speed lower than currently indicated.
- C. Move out at a speed higher than currently indicated.
- D. Move out at a speed lower than currently indicated.

**ANSWER** C. Increasing/Decreasing Distractors Y

- C. Move out at a speed higher than currently indicated.

Indicated T-avg/T-ref mismatch is  $\approx$  5-6°F.

**REFERENCE**

Precautions, Limitations & Setpoints, page 1-26 (R9)

Total Points: 1.00      Estimated Completion Time: 4      SRO Only: N

**X/A Reference:**

**Task:** 00103001

001010 K5.07 (3.2 - 3.6)

001050 K5.01 (3.3 - 3.6)

**Lesson Plan/Objective:**



**QUESTION**

Which ONE of the following describes the plant response if the "51", overcurrent relay, for the normal power supply to 6.9 kV Unit Bd 1A picked up?

- A. 6.9kV Shutdown Bd 1A-A would automatically transfer to its alternate power supply.
- B. 6.9kV Shutdown Bd. 1A-A would be de-energized and remain de-energized.
- C. Reactor trip on low RCS flow.
- D. Reactor trip on SSPS Train "A" General Warning.

**ANSWER** C. Increasing/Decreasing Distractors N

C. Reactor trip on low RCS flow.

**REFERENCE**

47W611-99-6 (R0)  
45N763-1 (R0)  
1-45N721-1 (R2)

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

**K/A Reference:** Task: 01200501  
012000 K4.02 (3.9 - 4.3)

**Lesson Plan/Objective:** OPL2739041 / B.5 (90 Requal Wk 2)

**QUESTION**

[NOT RELATED TO SCENARIO]

With the unit operating at 100% power, the system engineering group determined that one of the main steam line safety valves had been incorrectly set (its present setting is 50 psig above its allowable setting). Which ONE of the following actions is required concerning Reactor Power?

- A. May be maintained at 100%.
- B. Must be reduced to < 87%.
- C. Must be reduced to < 65%.
- D. Must be reduced to < 43%.

**ANSWER** B. Increasing/Decreasing Distractors Y

- B. Must be reduced to < 87%.

**REFERENCE**

Unit 1 Tech Specs LCO 3.7.1.1 (R160)

Total Points: 1.00 Estimated Completion Time: 3 SRO Only: N

**K/A Reference:**

Task: 19401503

039000 G5 (2.7 - 3.1)  
039000 G11 (2.8 - 3.1)**Lesson Plan/Objective:**

**QUESTION**

[NOT RELATED TO SCENARIO]

DETERMINE which ONE of the following conditions must be met to OPEN FCV-74-1 when initially establishing normal RHR Shutdown Cooling with RCS at 345°F & 365 psig.

- A. FCV-63-72, RHR hot leg injection, & FCV-63-1, RHR RWST suction, CLOSED.
- B. FCV-63-72 & FCV-63-73, Cntmt sump suction, CLOSED.
- C. FCV-63-1, RHR RWST suction, & FCV-63-72, Cntmt sump suction, CLOSED.
- D. FCV-63-73, Cntmt sump suction, & FCV-74-21, RHR suction, CLOSED.

**ANSWER** C. Increasing/Decreasing Distractors N

- C. FCV-63-1, RHR RWST suction, & FCV-63-72, Cntmt sump suction, CLOSED.

**REFERENCE**

47W611-74-1 (R1)

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

**K/A Reference:**

005-GENERIC #7 (3.3/3.4)

**Task:** 00500102

Lesson Plan/Objective: OPL2739006/ (90 Req Wk 1)

**QUESTION**

[NOT RELATED TO SCENARIO]

What 3 signals will cause FCV-62-136 to automatically open? (Include setpoints where applicable)

**ANSWER**

- a. Safety Injection Signal (.25 pts)
- b. Volume Control Tank Level Lo Lo (.25 pts),  
<7% (.125 pts)
- c. FCV-62-132 closed (.25 pts), >50% closed (.125 pts)

**REFERENCE**

1-47W611-62-4 (R5)  
Precautions, Limitations, & Setpoints (R9)

Total Points: 1.00      Estimated Completion Time: 0      SRO Only: N

K/A Reference:      Task: 00401501  
004010 K4.04 (3.1/3.4)

Lesson Plan/Objective: OPL2739063/ B.1 (90 Req Wk 4)

**QUESTION**

[NOT RELATED TO SCENARIO]

Which ONE of the following describes the response of the plant if 1-FCV-1-17 were to inadvertently close and remain closed? Assume the TDAFW is operating normally after a Rx trip.

- A. FCV-1-51 & FCV-1-15 would auto close immediately, then FCV-1-16 would open.
- B. FCV-1-51 & FCV-1-15 would auto close  $\approx$  60 seconds later, then FCV-1-16 would open.
- C. FCV-1-51 would auto close and FCV-1-16 would open immediately, then FCV-1-15 would close.
- D. FCV-1-51 and FCV-1-15 would auto close immediately,  $\approx$ 60 seconds later FCV-1-51 would open.

**ANSWER** B. Increasing/Decreasing Distractors N

- B. FCV-1-51 & FCV-1-15 would auto close  $\approx$  60 seconds later, then FCV-1-16 would open.

**REFERENCE**

45N657-26 (R4)  
45N779-28 (R2)  
47W611-1-1 & 4 (R2)

Total Points: 1.00 Estimated Completion Time: 5 SRO Only: N

K/A Reference: Task: 06100601  
001000 K4.14 (3.5 - 3.7)

Lesson Plan/Objective: OPL2738965/ B (89 Req Wk 5)

**QUESTION**

(NOT RELATED TO SCENARIO)

If PT-1-73 were to fail high (120%) and 30 seconds later a Reactor Trip was initiated by operator action, which one of the following describes the response of the steam dump system with steam dump controls in normal configuration?

- A. Steam dump system would attempt to maintain RCS Tav<sub>g</sub> at 552° via the Reactor Trip controller.
- B. Steam dump system would attempt to maintain PCS Tav<sub>g</sub> at 578° via the Load Rejection controller.
- C. Steam dump system would attempt to maintain RCS Tav<sub>g</sub> at 547° via the Reactor Trip controller.
- D. Steam dump system would not respond due to the absence of a valid arming signal.

**ANSWER**

Increasing/Decreasing Distractors N

- A. Steam dump system would attempt to maintain RCS Tav<sub>g</sub> at 552° via the Reactor Trip controller.

(DO NOT USE WITH 6.14)

**REFERENCE**

47W611-1-2 (R2)

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

K/A Reference:  
041020 K4.17 (3.7/3.9)

Task: 04100501

Lesson Plan/Objective: OPL2739146/ B.4 (9.1 Req Wk 4)

**QUESTION**

[NOT RELATED TO SCENARIO]

Which ONE of the following conditions would cause PCV-62-81 to automatically close?

- A. The pressure in the letdown header upstream of PCV-62-81 being lower than the controller's setpoint.
- B. The pressure in the letdown header upstream of PCV-62-81 being higher than the controller's setpoint.
- C. The pressure in the letdown header downstream of PCV-62-81 being lower than the controller's setpoint.
- D. The pressure in the letdown header downstream of PCV-62-81 being higher than the controller's setpoint.

ANSWER A. Increasing/Decreasing Distractors N

- A. The pressure in the letdown header upstream of PCV-62-81 being lower than the controller's setpoint.

**REFERENCE**

47W611-62-3 (R2) 47W809-1 (R15)

Total Points: 1.00 Estimated Completion Time: 3 SRO Only: N

**K/A Reference:**

Task: 00401501

191003 K1.03 (3.1 - 3.1)  
191003 K1.04 (2.8 - 3.0)  
004000 A3.02 (3.6 - 3.6)**Lesson Plan/Objective:**

OPL2739063 / B.1 &amp; B.3 (90 Requal Wk 4)

July 1992  
SRD Part B



**QUESTION**

Unit 1 is at 3% LTP with a startup in progress. The operating crew is notified that the Noble gas activity monitor for the auxiliary building ventilation radiation monitor (0-RM-90-101B) has failed to meet the acceptance criteria of a scheduled surveillance. Regarding ODCM compliance for the present plant condition, which ONE of the following is correct.

- A. The noble gas monitor is not required by plant ODCM; therefore, declare the noble gas monitor inoperable & continue with startup.
- B. Declare the auxiliary building vent radiation monitor inoperable and take necessary samples per ODCM; however startup may continue.
- C. Declare the noble gas monitor inoperable and take necessary samples per ODCM; however, startup may continue provided reactor power does not exceed 5%.
- D. Declare the noble gas monitor inoperable, take necessary samples per ODCM, and place the unit in Hot Standby until the monitor is restored to operability.

**ANSWER** B. Increasing/Decreasing Distractors N

- B. Declare the auxiliary building vent radiation monitor inoperable and take necessary samples per ODCM; however startup may continue.

**REFERENCE**

Tech Spec 3.3.3.10.b., Table 3.3 - 13, Item 5.a., Action 42  
OCM Table 1.5 Action 42 Page 91 (Feb 25)

Total Points: 1.00 Estimated Completion Time: 3 SRO Only: Y

**K/A Reference:**

073000 G5 (3.1 - 3.6)  
073000 G11 (2.8 - 3.4)

Task: 11901503

**Lesson Plan/Objective:** OPL273C202 / Objective B.1. (88 Requal WK 2)  
OPL2739135 / B.2 & B.3 (91 Requal WK 3)

**QUESTION**

Unit 1 is operating at 65% RTP. A RO-trainee informs you that one of the shutdown rod individual rod position indicators (RPIs) is indicating 220 steps. Reactor Engineering subsequently verifies the rod to be actually at 228 steps via a flux map. Which ONE of the following describes the proper response to this situation?

- A. Fully withdraw the affected rod within one (1) hour.
- B. Reduce power to less than 50% RTP within two (2) hours.
- C. No action required, as long as group demand indicates 228 steps and RPI is within  $\pm 12$  steps.
- D. No action required, the Technical Specification governing this event is only applicable in MODE 1 when greater than 75% RTP.

**ANSWER** C. Increasing/Decreasing Distractors N

- C. No action required, as long as group demand indicates 228 steps and RPI is within  $\pm 12$  steps.

**REFERENCE**

TS 3.1.3.6. & 3.1.3.5. & 3.1.3.2.  
AOI-2 (Rev 14)  
Obtained from Salem Part B Exam Bank

Total Points: 1.00 Estimated Completion Time: 3 SRO Only: N

**K/A Reference:****Task:** 31100106

014000 K4.06 (3.4 - 3.7)  
000005 EK.06 (3.9 - 4.2)  
001000 Gc (2.9 - 3.8)

**Lesson Plan/Objective:** OPL2739132 / B.1 (91 Regual Wk 3)

**QUESTION**

Unit 1 is in MODE 2 with reactor startup in progress. The following RCS leakages were determined per 1-SI-OPS-068-137.0, SI-137.3 & SI-137.5. (Note: Assume leakages other than those given as 0 gpm)

|                              |   |          |
|------------------------------|---|----------|
| Total RCS leakage            | = | 4.9 gpm  |
| PRT leakage                  | = | 0.81 gpm |
| CLA #1 leakage               | = | 0.19 gpm |
| RCDT leakage                 | = | 2.7 gpm  |
| HUT, CLA #2, #3, #4          | = | 0.0 gpm  |
| Controlled leakage           | = | 38 gpm   |
| S/G #1 leakage               | = | 0.40 gpm |
| S/G #2, #3, #4 leakage       | = | 0.0 gpm  |
| No Pressure Boundary Leakage |   |          |

Which ONE of the following describes the reason LCO 3.4.6.2 action b must be entered based upon the above leakages?

- A. UNIDENTIFIED LEAKAGE limit has been exceeded.
- B. IDENTIFIED LEAKAGE limit has been exceeded.
- C. PRIMARY-TO-SECONDARY LEAKAGE limit has been exceeded.
- D. CONTROLLED LEAKAGE limit has been exceeded.

ANSWER C. Increasing/Decreasing Distractors N

- C. PRIMARY-TO-SECONDARY LEAKAGE limit has been exceeded.

Identified Leakage = PRT + CLA + RCDT + S/G Leakages =  
0.81 + 0.19 + 2.7 + 0.4 = 4.1 gpm

Unidentified Leakage = RCS leakage - Identified leakage =  
4.9 - 4.1 = 0.8 gpm

Primary-to Secondary Leakage = 0.4 gpm = 576 gpd.

**REFERENCE**

Tech Spec 3.4.6.2 & Tech Spec "Definitions"

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

**K/A Reference:**

Task: 11901503

002000 G5 (3.6 - 4.1)  
002000 G11 (3.3 - 4.0)

Lesson Plan/Objective: OPL2739025 / B.4 (90 Requal Wk 2)

**QUESTION**

With both units at 100% power, CO2 is required to be isolated to the Unit 1 Auxiliary Instrument Room for maintenance. This event was logged in the following logs at the times specified.

- Control Room AUO Time 1616
- Unit 1 RO Log Time 1606
- SOS Log Time 1611

Which ONE of the following describes the minimum action required?

- A. Establish a continuous fire watch with backup suppression equipment by 1711.
- B. Establish hourly fire watch patrol by 1716.
- C. Establish hourly fire watch patrol by 1706.
- D. Establish a continuous fire watch with backup suppression equipment by 1706.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. Establish a continuous fire watch with backup suppression equipment by 1706.

**REFERENCE**

TS 3.7.11.3  
AI-6 (Rev 22)

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

**K/A Reference:**

194001 A1.03 (2.5 - 3.4)  
096000 G15 (3.3 - 3.6)

Task: 119015 ?

Lesson Plan/Objective: OPL2739107 / B.2 (91 Requal Wk 2)

**QUESTION**

On March 2, Unit 1 was operating at 70% power. Diesel Generator 1A-A was removed from service at 0800 hours to correct fuel injection problems. On March 4 at 1400 hours with the 1A-A D/G still inoperable 1B-B SI pump was declared INOPERABLE and in need of maintenance. The repairs to 1B-B SI pump are estimated to require 48 hours to complete. Which ONE of the following describes the most restrictive action which must be taken to comply with Technical Specifications?

- A. D/G 1A-A must be restored to OPERABLE by March 7 at 1400 hours or be in HOT SHUTDOWN by March 8 at 0200 hours.
- B. 1B-B SI pump must be restored to OPERABLE status by March 7 at 1400 hours or be in HOT STANDBY by March 8 at 0200 hours.
- C. Action must be initiated by March 4 at 1500 hours to place the Unit 1 in HOT STANDBY by March 4 at 2100 hours and HOT SHUTDOWN by March 5 at 0300 hours and COLD SHUTDOWN by March 6 at 0300.
- D. Action must be initiated by March 4 at 1600 hours to place the Unit 1 in HOT STANDBY by March 4 at 2200 hours and HOT SHUTDOWN by March 5 at 0400 hours.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. Action must be initiated by March 4 at 1600 hours to place the Unit 1 in HOT STANDBY by March 4 at 2200 hours and HOT SHUTDOWN by March 5 at 0400 hours.

**REFERENCE**

TS 3.0.3. & 3.5.2. & 3.5.3.  
Obtained from Salem Part B Exam Bank

Total Points: 1.00      Estimated Completion Time: 4      SRO Only: Y

**K/A Reference:**

Task: 31100106

006000 G6 (2.9 - 4.0)  
006000 G11 (3.6 - 4.2)

**Lesson Plan/Objective:** OPL273906C / B.3 (90 Requal Wk 4)  
OPL2739071 / B.5 (90 Requal Wk 4)

**QUESTION**

The unit is operating at 30% reactor power when RCP #1 trips due to a valid ground relay operation.

Select the ONE response below which describes the effect this will have on the unit.

- A. The loss of one RCP will NOT initiate a Rx trip but the operating crew should perform a controlled shutdown to Hot Standby within one hour.
- B. The loss of the RCP will initiate a low RCS flow reactor trip via the SSPS.
- C. The loss of a RCP will NOT initiate a Rx trip: power operation may continue provided the pump is restored to operation prior to exceeding 35% rated thermal power.
- D. The loss of one RCP will NOT initiate a Rx trip, but the operating crew should immediately trip the reactor.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. The loss of one RCP will NOT initiate a Rx trip, but the operating crew should immediately trip the reactor.

**REFERENCE**

Tech Specs 3.4.1.1 p. 3/4 4-1.  
Tech Specs 3.3.1.1 "Rx Trip Inst"  
AOI-5 Rev 13  
Event at the Plant.

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

**K/A Reference:** Task: 00008205  
003000 G5 (3.4 - 3.8)

**Lesson Plan/Objective:** OPL2739022 / B.2.z (90 Requal Wk 2 Sim)  
OPL271C072 / B.3 (Cert)  
OPL271C079 / B.3 (Cert)  
OPL271C083 / A (Cert)

**QUESTION**

The unit is operating at 85% RTP when a rod in control bank 'C' falls into the core. Assuming the reactor did not trip, select the ONE response below which states the proper action of the operating crew.

- A. Match T-avg with T-ref and reduce turbine power to  $\leq$  75% prior to retrieving dropped rod.
- B. Reduce turbine load to  $\leq$  75% and verify adequate shutdown margin prior to retrieving dropped rod.
- C. Insert control bank 'D' to match T-avg with T-ref prior to retrieving the dropped rod.
- D. Verify the automatic primary side runback to 75% turbine load/reactor power prior retrieving the dropped rod.

**ANSWER** A. Increasing/Decreasing Distractors N

- A. Match T-avg with T-ref and reduce turbine power to  $\leq$  75% prior to retrieving dropped rod.

**REFERENCE**

AOI-2 (Rev. 14)

Total Points: 1.00 Estimated Completion Time: 3 SRO Only: N

**K/A Reference:**

Task: 00000105

000003 EK1.01 (3.2 - 3.7)

000003 EK3.04 (3.8 - 4.1)

**Lesson Plan/Objective:** OPL273S212 / B.1, B.2, B.3 (Requal 88, Wk #2);  
OPL2739009 / A (90 Requal Wk 1)

**QUESTION**

You are the Unit 1 SRO in charge of refueling. You just noticed a significant decrease in level in the reactor cavity area. The control room subsequently informs you of a lot of water leaking into lower containment. You presently have spent fuel assemblies (SFA) located in the following places:

- One spent fuel assembly (SFA) up in the manipulator crane.
- One spent fuel assembly (SFA) in RCCA change fixture.
- One spent fuel assembly (SFA) up on spent fuel bridge crane.

Which ONE of the following describes the proper course of actions?

- A. Close the wafer valve, place both SFAs from the manipulator crane & RCCA change fixture into core and place SFA on spent fuel bridge into Spent Fuel Storage pit (store in any location).
- B. Transfer all three spent fuel assemblies into Spent Fuel Storage pit (store in any location) via fuel transfer cart/spent fuel bridge and close wafer valve.
- C. Place SFA from manipulator crane into core via manipulator crane, place SFA on spent fuel bridge into Spent Fuel Storage pit (store in any location), transfer SFA in RCCA change fixture into Rx side upender and transport to spent fuel pit side upender leaving in horizontal position and close wafer valve.
- D. Place SFA from manipulator crane into Rx side upender and transport to spent fuel pit side upender leaving in horizontal position, place SFA on spent fuel bridge into Spent Fuel Storage pit (store in any location), transfer SFA in RCCA change fixture into core via manipulator crane and close wafer valve.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. Place SFA from manipulator crane into Rx side upender and transport to spent fuel pit side upender leaving in horizontal position, place SFA on spent fuel bridge into Spent Fuel Storage pit (store in any location), transfer SFA in RCCA change fixture into core via manipulator crane and close wafer valve.

**REFERENCE**

AOI-29 (Rev 8) Section D

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: Y

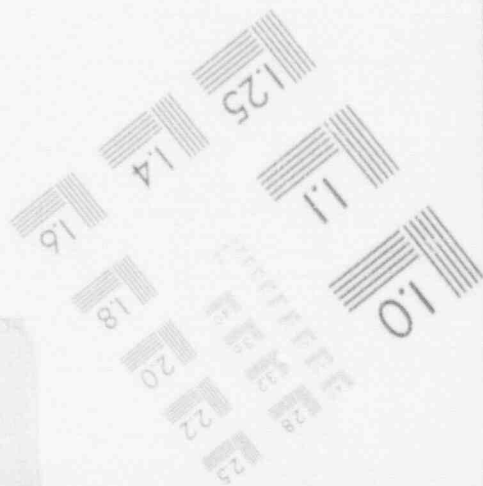
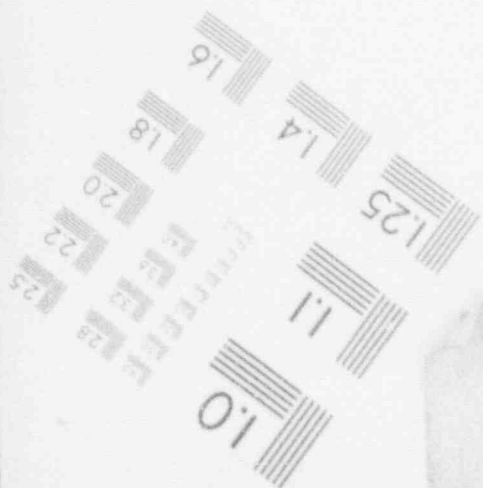
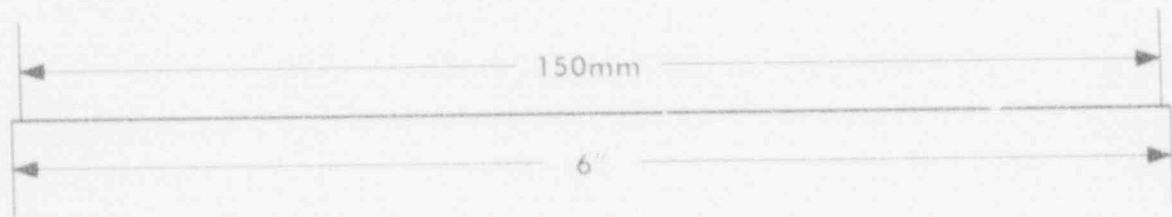
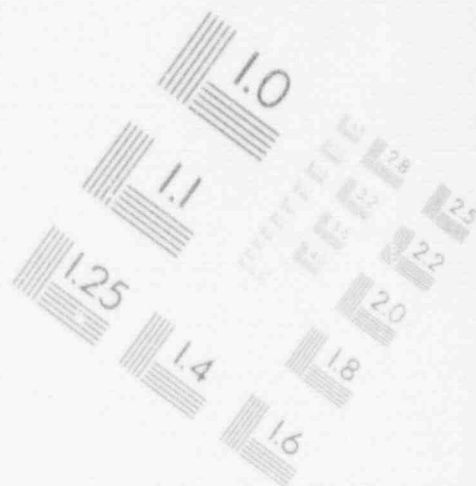
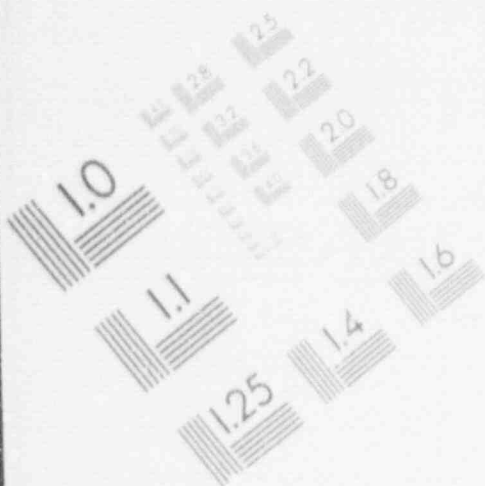
K/A Reference:

Task: 03400501



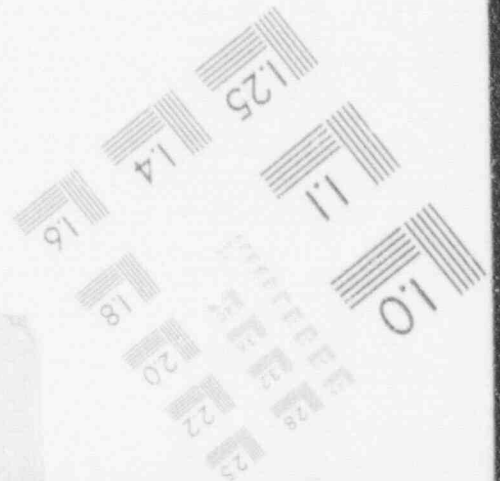
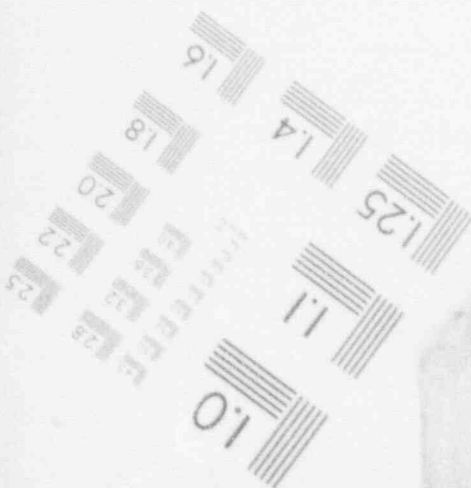
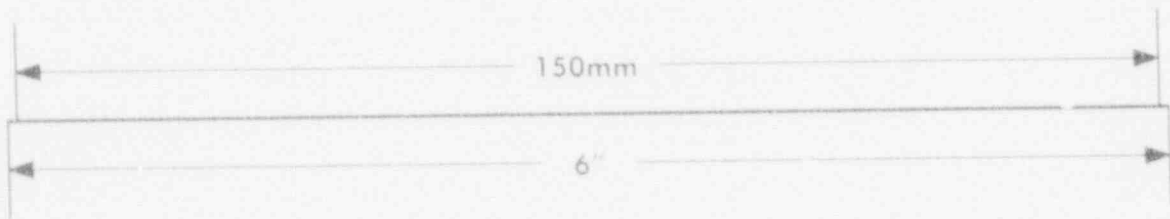
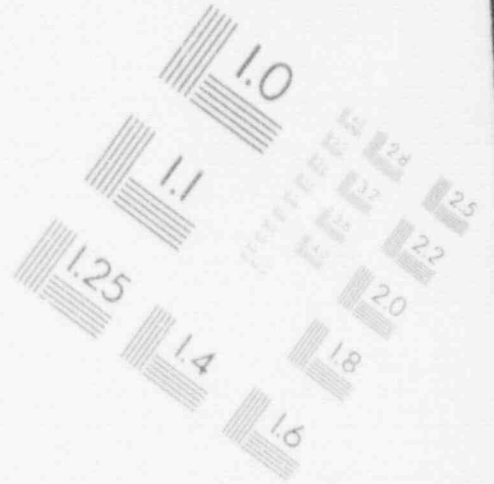
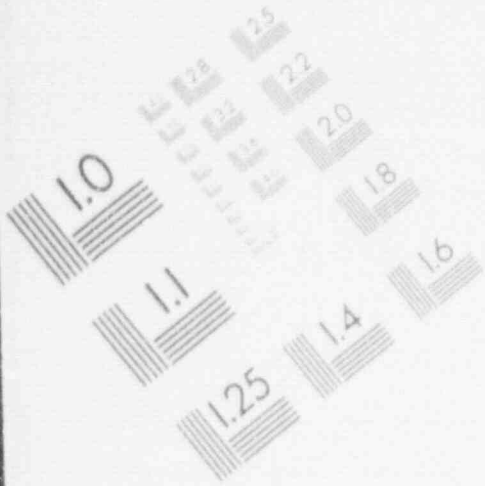
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## IMAGE EVALUATION TEST TARGET (MT-3)



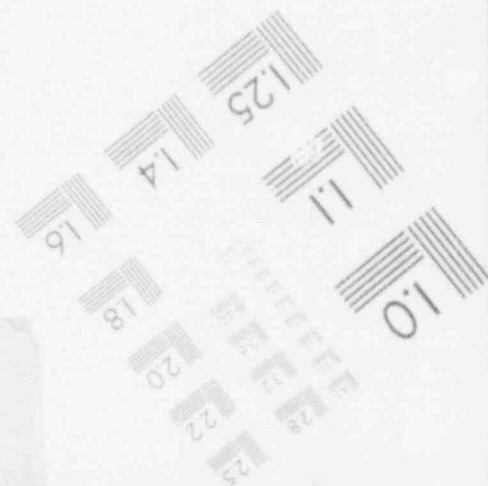
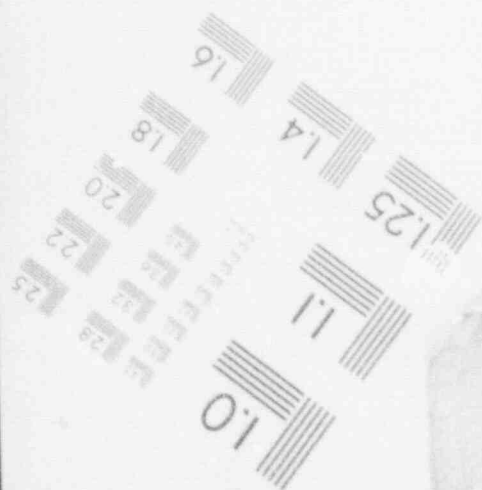
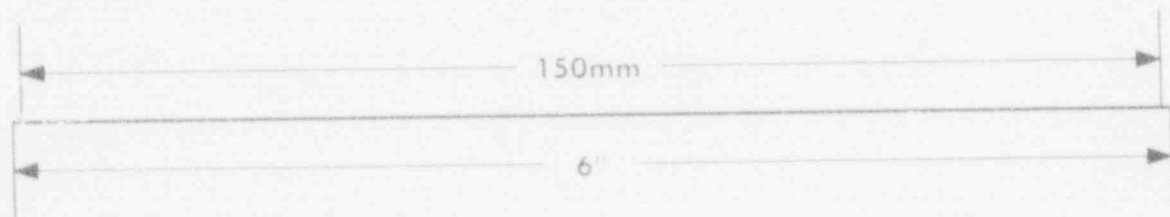
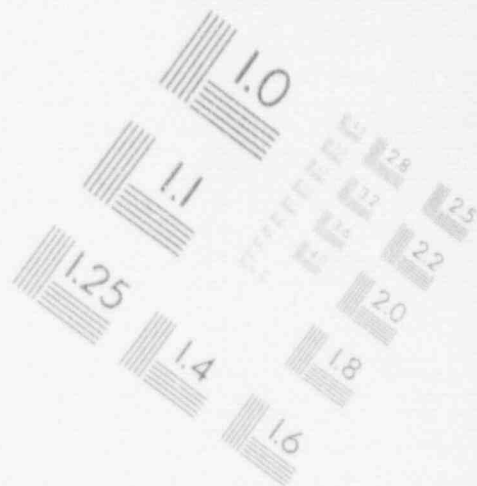
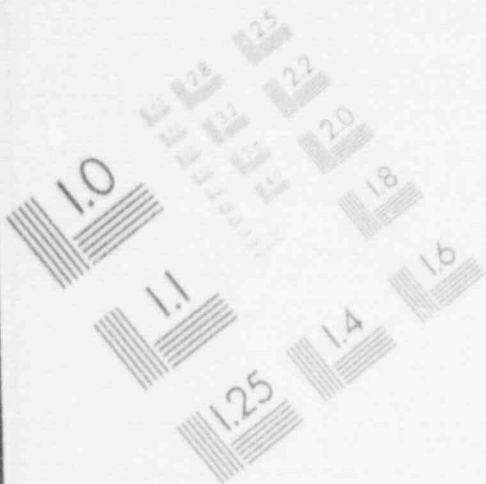
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## IMAGE EVALUATION TEST TARGET (MT-3)



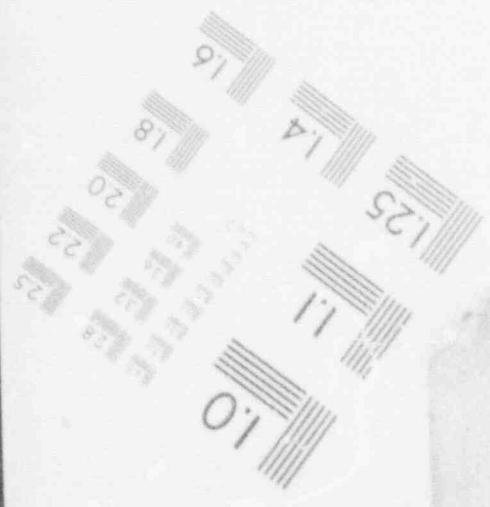
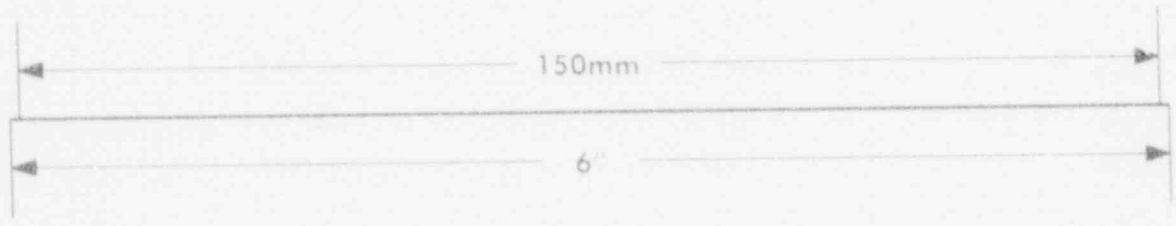
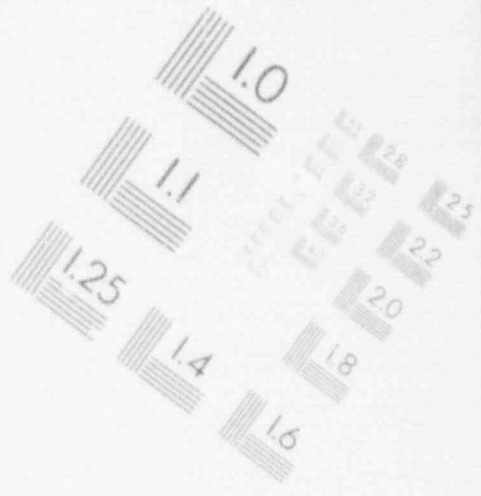
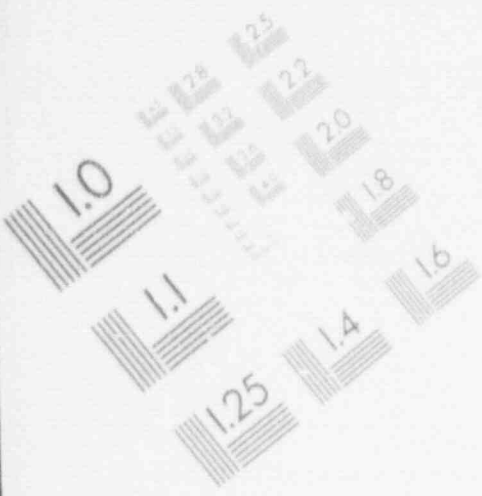
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## IMAGE EVALUATION TEST TARGET (MT-3)



# 1

## IMAGE EVALUATION TEST TARGET (MT-3)



000036 EK3.03 (3.7 - 4.1)  
000036 G11 (3.5 - 3.9)  
000036 G12 (3.2 - 3.4)

**Lesson Plan/Objective:** OPL2739129 / B.2 & B.4 (91 Requal Wk 3)

**QUESTION**

Unit 1 is at 100% RTP when a small RCS leak develops. The operating crew takes the necessary actions to stabilize PZR level by starting a second charging pump. Subsequent to these actions the crew attempts to determine the leakage source. After ISOLATING LETDOWN the crew observes the following parameters/indications:

Let-down flow 0 gpm  
Cnmt pressure  $\approx$  0.2 and increasing  
RCS pressure  $\approx$  2210  
Pzr level  $\approx$  58% and stable  
Charging flow 125 gpm  
Cntmt radiation monitors increasing

Which ONE of the following actions should the operating crew perform next?

- A. Continue Power Operations, no further actions required.
- B. Immediately trip the Reactor.
- C. Place excess let-down inservice and adjust charging.
- D. Continue attempts to determine source of leak and proceed to COLD SHUTDOWN.

ANSWER D. Increasing/Decreasing Distractors N

- D. Continue attempts to determine source of leak and proceed to COLD SHUTDOWN.

**REFERENCE**

AOI-6 (Rev 27)

Total Points: 1.00 Estimated Completion Time: 3 SRO Only: N

**K/A Reference:**

000009 EK3.21 (4.2 - 4.5)  
000009 EA2.06 (3.8 - 4.3)

Task: 32100504

Lesson Plan/Objective: OPL2739025 / B.3 (90 Requal Wk 2)

**QUESTION**

The operators are in the process of transitioning out of E-0 when the STA observes the following information.

Plant has experienced a LOCA and ESP actuation

- AFW flows
  - S/G #1 105 gpm
  - S/G #2 100 gpm
  - S/G #3 100 gpm
  - S/G #4 100 gpm
- S/G narrow range levels
  - S/G #1 20%
  - S/G #2 21%
  - S/G #3 22%
  - S/G #4 21%
- Cntmt pressure 5.4 psi
- No RCPS operating
- RCS subcooling  $\approx$  5°F
- RVLIS lower range 32%
- IRM SUR + 0.2 dpm
- PRMs  $\approx$  1% power
- Core exit T/Cs avg 713°F
- Core exit max T/Cs 755°F
- All ECCS components in service

Which ONE of the following procedures must be implemented first based on the above conditions?

- A. FR-S.1
- B. FR-C.1
- C. FR-H.1
- D. FR-Z.1

ANSWER B. Increasing/Decreasing Distractors N

- B. FR-C.1

**REFERENCE**

FR-0 "Status Trees" (Rev. 9)

Total Points: 1.00 Estimated Completion Time: 5 SRO Only: N

**K/A Reference:**

Task: 31100306

- 002000 A2.04 (4.3 - 4.6)
- 000054 SG11 (3.4 - 3.3)
- 000074 SG11 (4.5 - 4.6)
- 000029 SG11 (4.4 - 4.6)

Lesson Plan/Objective: OPL2739112 / B.8 & B.12 (91 Requal Wk 2)

Sequoyah Nuclear Plant  
Question # B-0005A

06/28/92

Revision 2

OPL2739005 / B.5 (90 REQUAL)  
OPL2739126 / B.4 & B.8 (91 Requal Wk 3)



**QUESTION**

The control room operators are performing the immediate action steps following a reactor trip with SI. Containment pressure has exceeded the containment spray automatic setpoint, so the operators verify that containment spray has initiated and that the containment isolation phase "B" valves has closed. At this point, the applicable procedure instructs them to trip the RCPs.

Which ONE of the following would describe the likely consequences of NOT tripping the RCPs at this point?

- A. Forced circulation will make the containment high-pressure transient more severe by sustaining an excessive energy release rate to containment.
- B. The RCP motor bearings will overheat because component cooling flow to them has been isolated.
- C. Radioactive materials will be released from containment because a potential release path remains unisolated.
- D. The RCPs will trip on low RCS pressure because it is assumed that if containment spray actuates, an RCS depressurization will be in progress.

**ANSWER** B. Increasing/Decreasing Distractors N

- B. The RCP motor bearings will overheat because component cooling flow to them has been isolated.

**REFERENCE**

E-0 (Rev 12)

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

K/A Reference: Task: 30100106  
000007 EK3.01 (4.0 - 4.6)

**Lesson Plan/Objective:** OPL2739113 / B.3 (91 Regual Wk 2)  
OPL2749202 / B.c(92 Special Regual Wk 2)

**QUESTION**

During a Loss of All A/C Power, when power can not be quickly restored, a rapid cooldown/depressurization is initiated per ECA-0.0.

Which ONE of the following is the reason for these actions?

- A. Drop RCS pressure to reduce ECCS leakage thereby maintaining pressurizer level onscale.
- B. Cooldown to COLD SHUTDOWN which is a stable condition so RHR can be placed in service.
- C. Maintain natural circulation thereby preventing hydrogen gas forming in the vessel head area.
- D. Cooldown and depressurize to reduce leakage through the RCP seals.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. Cooldown and depressurize to reduce leakage through the RCP seals.

**REFERENCE**

ECA-0.0 "Loss of All AC Power" (Rev. 6)

Total Points: 1.00 Estimated Completion Time: 3 SRO Only: N

**K/A Reference:**

**Task:** 00008605

000055 EK3.02 (4.3 - 4.6)

**Lesson Plan/Objective:** OPL2738903 / B.1 & B.2 (89 Requal Wk 1)

Sequoyah Nuclear Plant  
Question # B-0303C

Revision 0

**QUESTION**

The plant is initially operating at 100% FTP with all controls in automatic. The following conditions appear almost simultaneously:

SI on low pressurizer pressure  
S/Gs #2, #3 & #4 pressure at  $\approx$  1005 psig  
S/G #1 pressure at  $\approx$  650 psig & decreasing  
High radiation alarms on Condenser Vacuum pump exhaust, S/G  
blowdown & Loop #1 MS Line.  
Containment pressure  $\approx$  0.1 psid.

Which ONE of the following is the proper emergency operating procedure flowpath for the crew?

- A. E-0, E-2, ECA-3.1.
- B. E-0, E-2, E-1, E-3.
- C. E-0, E-2, E-1, ECA-3.1.
- D. E-0, E-2, E-3, ECA-3.1.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. E-0, E-2, E-3, ECA-3.1.

**REFERENCE**

E-0 Rev 12

E-3 Rev 8

E-2 Rev 7

WOG Training Part B Question Item # 000-038-011

Total Points: 1.00

Estimated Completion Time: 4

SRO Only: N

**K/A Reference:****Task:** 00003805

000038 EA2.02 (4.5 - 4.8)

000038 GA11 (4.2 - 4.3)

000038 EA1.12 (4.3 - 4.3)

000038 GK1 (3.3 - 4.1)

000038 EA1.10 (3.7 - 3.7)

**Lesson Plan/Objective:** OPL2739114 / B.1 (91 Requal Wk 2)  
OPL2739115 / B.1 (91 Requal Wk 2)  
OPL2749201 / B.a (92 Requal Special Wk 2)  
OPL2739210 / B.a (92 Requal Wk 2)

**QUESTION**

A natural circulation cooldown is in progress after a reactor trip from 100% power because offsite power has been lost. Two CRDM cooling fans are inoperable.

Which ONE of the following describes how the inoperability of the CRDM fans will affect the cooldown and depressurization?

- A. It has no effect because the amount of RCS heat removal by running the fans is insignificant compared to that removed by steaming the secondary plant.
- B. Less subcooling can be maintained and this will result in a longer upper head cool-off time period.
- C. Greater minimum subcooling must be maintained and the total upper head cooldown rate will be less.
- D. The upper head will void since there is not enough cooling available with only two CRDM fans.

**ANSWER** C. Increasing/Decreasing Distractors N

- C. Greater minimum subcooling must be maintained and the total upper head cooldown rate will be less.

**REFERENCE**

ES-0.2 (Rev 7)

WOG Written Training Item # 000-056-012

Total Points: 1.00

Estimated Completion Time: 2

SRO Only: N

**K/A Reference:**

000056 EK3.02 (4.4 - 4.7)

**Task:** 00201001

**Lesson Plan/Objective:** OPL2739112 / B.3 & B.2 (91 Requal Wk 2)  
OPL273C101 / V.G (Requal 88 Wk 1)  
OPL2739153 / B.9 (91 Requal Wk 5)

**QUESTION**

Unit 1 is operating at 100% RTP when a LOCA occurs. The operating crew has transitioned to E-1 when the following conditions are observed:

No CCPs are available  
 RWST level 92%  
 Cntmt sump level ≈1%  
 All hot leg temperatures (WR) > 700°F  
 Core exit T/Cs > 1250°F & increasing  
 Cntmt press 3.2 psid and MSIVs closed  
 RVLIS lower range 37% and decreasing  
 RCPS off  
 RCS pressure ≈1600 psig

Select the ONE response below which describes the proper operating crew action for the given conditions?

- A. Cooldown the RCS within the 100°F/hr cooldown limit using the S/G atmospheric relief vales. This should result in a subsequent decrease in RCS pressure allowing increase ECCS flow.
- B. Cooldown the RCS within the 100°F/hr cooldown limit using the steam dump valves. This should result in a subsequent decrease in RCS pressure allowing increase ECCS flow.
- C. Cooldown the RCS as quickly as possible using the steam dump valves. This should result in a subsequent decrease in RCS pressure allowing increase ECCS flow.
- D. Cooldown the RCS as quickly as possible using the S/G atmospheric relief valves. This should result in a subsequent decrease in RCS pressure allowing increase ECCS flow.

**ANSWER** D. Increasing/Decreasing Distractors N

- D. Cooldown the RCS as quickly as possible using the S/G atmospheric relief valves. This should result in a subsequent decrease in RCS pressure allowing increase ECCS flow.

**REFERENCE**

FR-C.1 "Response to Inadequate Core Cooling" (Rev 6)

Total Points: 1.00 Estimated Completion Time: 5 SRO Only: N

K/A Reference:

Task: 31100306

000074 EK3.11 (4.0 - 4.4)

Lesson Plan/Objective: EGT006.4 / J (87 Requal)  
OPL2739004 / B.1 (90 Requal Wk 1)  
OPL2739126 / B.2 (91 Requal Wk 3)

**QUESTION**

A main steam line break occurred on Unit 1. The operating crew completes all necessary actions. During the event RCS cold leg temperatures dropped from 547°F to 220°F in less than 30 minutes.

Based on the above information, which ONE of the following explains when the operating crew can begin a cooldown to cold shutdown?

- A. Immediately, provided during controlled cooldown the T-cold does NOT decrease more than 100°F in any one (1) hour.
- B. Immediately, provided the cooldown rate is slow enough such that when target T-cold is reached the average cooldown rate is  $\leq 100^\circ\text{F}$  per hour.
- C. Cooldown can NOT resume until RCS temperatures have been stable for  $\geq 1$  hour.
- D. Cooldown can NOT resume until an engineering evaluation is performed to determine the effects of the cooldown on the reactor pressure vessel.

**ANSWER** C. Increasing/Decreasing Distractors N

- C. Cooldown can NOT resume until RCS temperatures have been stable for  $\geq 1$  hour.

**REFERENCE**

FR-P.1 rev 7

WOG Training Part B Question Item # 000-040-007

Total Points: 1.00      Estimated Completion Time: 3      SRO Only: N

**K/A Reference:**

000040 CA12 (3.8 - 4.1)  
000040 EA1.12 (4.2 - 4.2)

**Task:** 31101406

**Lesson Plan/Objective:** EGT222.024 (89 Requal Wk 5)  
OPL2739126 / B.3 (91 Requal Wk 3)  
OPL2739027 / B.1, .4, .5 (90 Requal Wk 2)  
OPL2749202 / B.b(92 Special Requal Wk 2)

**QUESTION**

The plant has experienced a reactor trip from 100% power. During response to the trip, both AFW LCVs to S/G #1 failed to open. Maintenance was dispatched to repair the LCVs. The following conditions exist when maintenance has repaired the LCVs.

- Core exit T/Cs  $\approx$  500°F and stable.
- Loop 1 T-hot  $\approx$  490°F and stable.
- S/G #1 WIDE RANGE level  $\approx$  5%
- S/Gs #2, #3, & #4 narrow range  $\approx$  33%.

Under these conditions which ONE of the following AFW flow rates is advisable to S/C #1?

- A. Maximum rate ( $\approx$  440 gpm) to prevent imminent core uncover.
- B. Minimum rate ( $\approx$  120 gpm) required to satisfy the heat sink status tree.
- C. Very slow rate ( between 25 & 100 gpm) to minimize thermal shock to S/G #1.
- D. AFW flow should not be established to S/G #1.

**ANSWER** C. Increasing/Decreasing Distractors Y

- C. Very slow rate ( between 25 & 100 gpm) to minimize thermal shock to S/G #1.

**REFERENCE**

WOG ERG Background Document Rev 1  
FR-H.5 Rev 3  
WOG Training Item # 000-054-010

Total Points: 1.00      Estimated Completion Time: 2      SRO Only: N

**K/A Reference:**

**Task:** 31102206

000054 G12 (3.2 - 3.2)  
000054 EK3.04 (4.4 - 4.6)

**Lesson Plan/Objective:** OPL2739026 / B.9 & B.4 (90 Requal Wk 2)  
OPL2739126 / B.2 (91 Requal Wk 3)



**QUESTION**

Four (4) hours after a LOCA has occurred on Unit 1 the following conditions exist:

- All ECCS pumps and Contmt Spray pumps running aligned to RHR Containment Sump.
- Faults in the 161/500KV switchyards result in a loss of offsite power.

Which ONE of the following operator actions is initially required in response to this condition?

- A. Reinitiate Safety Injection (SI) and return to E-0, 'Reactor Trip or Safety Injection'.
- B. Ensure both trains of shutdown power are available and continue procedure & step.
- C. Ensure Diesel Generators start and ALL loads sequence on properly.
- D. Pull-to-Lock both Centrifugal Charging Pumps (CCPs).

**ANSWER** D. Increasing/Decreasing Distractors N

- D. Pull-to-Lock both Centrifugal Charging Pumps (CCPs).

**REFERENCE**

ES-1.3 (Rev 4)  
Obtained from WBNP Part B Bank

Total Points: 1.00      Estimated Completion Time: 5      SPO Only: N

**K/A Reference:**      **Task:** 00001105  
000011 G12 (4.0 - 4.1)

**Lesson Plan/Objective:** OPL2739114 / B.1 (91 Requal Wk 2)  
OPL2749202 / B.b (92 Special Requal Wk 2)

**QUESTION**

A LOCA has occurred inside containment, and the control room operators are responding to a loss of RHR sump recirculation. They are unable to restore emergency coolant recirculation, so they add makeup to the RWST, verify that containment spray operating requirements are met and establish only one train of ECCS flow.

The basis for establishing only one train of ECCS flow is because of which ONE of the following reasons?

- A. Delay the time to depletion of the Refueling Water Storage Tank.
- B. Reduce the break flow from the Loss of Coolant Accident.
- C. Begin depressurizing the RCS to minimize subcooling.
- D. Provide the proper RCS pressure conditions for re-establishing emergency coolant recirculation.

**ANSWER** A. Increasing/Decreasing Distractors N

- A. Delay the time to depletion of the Refueling Water Storage Tank.

**REFERENCE**

EC-1.1 (Rev 6)

WOG Written Training Item # 000-011-009

Total Points: 1.00      Estimated Completion Time: 3      SRO Only: N

**K/A Reference:**

000011 EK3 12 (4.4 - 4.6)

**Task:** 00007805

**Lesson Plan/Objective:** OPL2738925 / B.1 (89 Requal Wk 2)  
OPL2739127 / B.2 (91 Requal Wk 3)

**QUESTION**

The reactor has been tripped and the operating crew has been directed to E-3 "SGTR". The ruptured S/G PORV controller is left in automatic and the setpoint is increased to 100%.

Which ONE of the following responses explains the purpose of leaving the controller in automatic & increasing the setpoint upon identifying it is the ruptured S/G?

- A. Minimize atmospheric releases and prevent lifting of the code safety valves.
- B. Ensure subsequent cooldown and depressurization of the ruptured S/G will be possible.
- C. Increase ruptured S/G pressure to the point at which primary-to-secondary leakage should be terminated.
- D. Stabilize ruptured S/G pressure and level to prevent an uncontrolled cooldown of the RCS.

**ANSWER** A. Increasing/Decreasing Distractors N

- A. Minimize atmospheric releases and prevent lifting of the code safety valves.

**REFERENCE**

E-3 rev 8

WOG Training Part B Question Item # 000-038-006

Total Points: 1.00

Estimated Completion Time: 2

SRO Only: N

**K/A Reference:****Task:** 00003805

000038 EK3.02 (4.4 - 4.5)  
000038 GA6 (3.8 - 4.0)  
000038 EA1.16 (3.9 - 3.9)  
000038 EK3.06 (4.2 - 4.5)  
000038 GA12 (3.8 - 4.0)  
000038 EA2.02 (4.5 - 4.8)

**Lesson Plan/Objective:** OPL2739115 / B.2 (91 Requal Wk 2)  
OPL2749201 / B.c (92 Special Requal Wk 2)

**QUESTION**

The control room operators are responding to a loss of secondary heat sink and have initiated RCS feed-and-bleed. The SOS directs the STA to determine if feed-and-bleed is adequate and to make recommendations. The STA observes that ECCS train A is in service and train B is not. All pressurizer PORVs are open in manual. The level in S/G #1 is 8% on the narrow range and the level in the other S/Gs is below the narrow range. Core exit T/Cs and RCS T-hot are decreasing.

Based on these indications, which ONE of the following should the STA report?

|    | Feed Path  | Bleed Path | Recommendations  |
|----|------------|------------|--|
| A. | Adequate   | Adequate   | ECCS train B should be placed into service if possible to maximize RCS     |
| B. | Adequate   | Adequate   | Feed-and-Bleed can be terminated because adequate secondary heat           |
| C. | Inadequate | Adequate   | ECCS train B must be placed into service to provide adequate feed          |
| D. | Adequate   | Inadequate | The pwr PORVs should be in auto, cycling open at their pressure setpoints. |

**ANSWER** A. Increasing/Decreasing Distractors N

|    |          |          |  |
|----|----------|----------|--|
| A. | Adequate | Adequate | ECCS train B should be placed into service if possible to maximize RCS |
|----|----------|----------|--|

**REFERENCE**

FR-H.1 Rev 7

WOG Training Part B Question Item # 000-054-005

Total Points: 1.00      Estimated Completion Time: 7      SRO Only: N

**K/A Reference:****Task:** 31100606

000054 EK1.04 (4.4 - 4.6)

000054 SG12 (3.2 - 3.2)

**Lesson Plan/Objective:** OPL2739126 / B.3 (91 Requal Wk 3)  
OPL2739026 / B.1, .4, .5 (90 Requal Wk 2)

**QUESTION**

Unit 1 is in Mode 5 with RCS at Mid-Loop with 1A-A RHR pump in service. Which ONE of the following action(s) will be required IF the 1A-A RHR pump motor amps become very erratic and continues to be erratic even after flow is reduced?

- A. Make up to the RCS immediately, by opening FCV-63-1.
- B. Make up to the RCS immediately via FCV-62-135 & 136, suction of CCPs from the RWST.
- C. Stop 1A-A RHR pump, increase the level in the RCS and attempt to restore RHR.
- D. Stop 1A-A RHR pump and start 1B-B RHR pump.

**ANSWER** C. Increasing/Decreasing Distractors N

- C. Stop 1A-A RHR pump, increase the level in the RCS and attempt to restore RHR.

**REFERENCE**

AOI-14 Rev 14

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

K/A Reference: Task: 00008305

000025 EK1.01 (3.0 - 4.3)

Lesson Plan/Objective: OPL2739006 / B.8 (90 Requal Wk 1)

**QUESTION**

The unit is shutdown in mode 5. The RCS is being filled from mid-loop and has just reached elevation 699'.

How many gallons of water must be added to the RCS to bring the RCS level elevation 705'9".

- A. 12,840 gallons
- B. 11,240 gallons
- C. 10,390 gallons
- D. 9,810 gallons

**ANSWER** A. Increasing/Decreasing Distractors Y

- A. 12,840 gallons

**REFERENCE**

TI-28 Rev 78  
 O-PI-OPS-068-673.S (Rev 1)  
 O-SO-68.1 (Rev 5)  
 O-GO-4.0 (Rev 4)

Total Points: 1.00 Estimated Completion Time: 11 SRO Only: N

**K/A Reference:** Task: 00200501  
 000025 EK1.01 (3.9 - 4.3)

**Lesson Plan/Objective:** OPL2739006 / B.7 (90 Requal Wk 1)

**QUESTION**

Which ONE of the following explains the requirements for declaring equipment operable after maintenance or testing?

- A. All required testing is complete, deficiencies recorded, and data sheets reviewed by appropriate system supervisor.
- B. All testing complete, all acceptance criteria required by technical specifications satisfied, review and approval of test(s) by the SRO has been completed.
- C. All testing/maintenance has been completed, responsible foreman notifies RO/SRO on affected unit that no deficiencies exist, and then the SRO can declare equipment operable.
- D. Equipment can be declared operable after inspection by operations, maintenance personnel ensures components/equipment is ready for return to service and the SOS authorizes the return to service.

**ANSWER** B. Increasing/Decreasing Distractors N

- B. All testing complete, all acceptance criteria required by technical specifications satisfied, review and approval of test(s) by the SRO has been completed.

**REFERENCE**

AI-30 Rev 36  
Obtained from Wk2 1989 Requal

Total Points: 1.00 Estimated Completion Time: 2 SRO Only: N

**K/A Reference:**

194001 A1.02 (4.1 - 3.9)  
194001 A1.08 (2.6 - 3.1)

Task: 11901503

**Lesson Plan/Objective:** OPL2738920 / B.2 (Wk 2 Requal 1989)  
OPL2739141 / B.2 (91 Requal Wk 4)

**QUESTION**

A control room operator (RO) is returning from a special assignment which removed him from his license duties for a period of 7 months. Which ONE of the following best describes the minimum action(s) necessary for the operator to assume a license position?

- A. Stand 8 hours under the direction of a Reactor Operator in the position to which they will be assigned. Perform a complete plant tour and be briefed by operations management on changes which have occurred during the interim.
- B. Stand 7 8-hour shifts under the direction of a Reactor Operator in the position to which they will be assigned. Satisfactorily pass a written and oral requalification examination.
- C. Stand 40 hours under the direction of a Reactor Operator in the position to which they will be assigned. Within the 40 hours, perform a complete tour of the plant and all required shift turnover procedures. Recieve letter from OPS Manager.
- D. Stand 56 hours under the direction of a Reactor Operator in the position to which they will be assigned. Within the 56 hours perform all required shift turnover procedures. Receive a letter from the Operations Manager reinstating the operator's license to an "active" status.

**ANSWER** C. Increasing/Decreasing Distractors N

- C. Stand 40 hours under the direction of a Reactor Operator in the position to which they will be assigned. Within the 40 hours, perform a complete tour of the plant and all required shift turnover procedures. Recieve letter from OPS Manager.

**REFERENCE**

AI-30 (Rev 33) Section 30  
Obtained from Salem Part B Exam Bank

Total Points: 1.00 Estimated Completion Time: 4 SRO Only: N

K/A Reference: Task: 11901503  
194001 AI.0 (2.5 - 3.4)

Lesson Plan/Objective: OPL2739141 / B.10 (91 Requal Wk 4)



**QUESTION**

An accident has occurred on Unit 1. The following plant conditions currently exist:

- Total loss of main feedwater, condensate & AFW to the S/Gs for greater than 30 minutes.
- Core Cooling status tree indicates an "ORANGE" path.
- RCS pressure  $\approx$  1900 psig.
- All S/G wide range levels decreasing at  $\approx$  35%.
- Containment upper & lower high range radiation monitors indicate  $<$  1 REM/hr.
- Containment parameters are beginning to increase above normal.
- No plant releases are detected.
- Both Containment Spray pumps & FCV-72-40/41 are all unavailable.

Which ONE of the following is the appropriate protective action recommendation?

- A. Shelter 2 mile radius. Shelter actual and projected downwind to 5 miles. (Recommendation 2)
- B. Evacuate to 2 mile radius. Evacuate actual and projected downwind to 5 miles. Shelter other sectors to 5 miles. (Recommendation 3)
- C. Shelter to 5 mile radius. Prepare to evacuate to 2 mile radius and actual and projected downwind sectors to 5 miles. (Recommendation 4)
- D. Evacuate actual and projected downwind sectors to 10 miles. Evacuate other sectors to 5 miles. Shelter others to 10 miles. Prepare to evacuate all sectors to 10 miles. (Recommendation 5)

**ANSWER** C. Increasing/Decreasing Distractors Y

- C. Shelter to 5 mile radius. Prepare to evacuate to 2 mile radius and actual and projected downwind sectors to 5 miles. (Recommendation 4)

**REFERENCE**

EPIP-5 (Rev. 4)

EPIP-1 (Rev. 6)

Total Points: 1.00

Estimated Completion Time: 5

SRO Only: Y

**K/A Reference:**

194001 A1.16 (3.1 - 4.4)

**Task:** 34401703

Lesson Plan/Objective: OPL2738836 / 4 (88 Requal)  
OPL2738976 / B.2 & B.9 (89 Wk 6 Requal)  
OPL2739075 / B.2 (90 Requal Wk 4)

**QUESTION**

The plant experiences a loss of all AC power. Efforts to restore AC power prove unsuccessful. RCS inventory depletion from RCP seal leakage continues, eventually draining the upper head of the reactor vessel and causing steam voids to form in the S/G U-tubes.

If the operators are unable to restore AC power, which ONE of the following explains the status of natural circulation?

- A. Stop and all means of decay heat removal will be lost. Extensive core damage will occur soon after the interruption of natural circulation.
- B. Stop and reflux boiling will remove decay heat until enough inventory is lost to prevent decay heat removal. Then inadequate core cooling may occur.
- C. Stop but reflux boiling will provide adequate decay heat removal for as long as necessary.
- D. Decrease but continue to provide adequate decay heat removal for as long as necessary.

**ANSWER** B. Increasing/Decreasing Distractors N

- B. Stop and reflux boiling will remove decay heat until enough inventory is lost to prevent decay heat removal. Then inadequate core cooling may occur.

**REFERENCE**

ECA-0.0 (Rev 6)  
WOG Training Part B Item # 000-055-011

Total Points: 1.00 Estimated Completion Time: 2 SRO Only: N

**K/A Reference:** Task: 00008605  
000055 EK1.02 (4.1 - 4.4)  
000055 EA2.02 (4.4 - 4.6)

**Lesson Plan/Objective:** OPL2739127 / B.2 (91 Requal Wk 3)  
OPL2739153 / B.9 (91 Requal Wk 5)  
EGT006.4 (Wk 5/6 87) / V.D.E.G.H  
EGT222.024 / B.7 & B.8 (89 REQUAL WK 5)

## QUESTION

-3

Unit 1 is being maintained at 10 % power on the Intermediate Range Monitors to take critical data. Boron concentration is 1280 ppm, Beginning of Life, Control Bank "D" is at 147 steps. Assume all appropriate systems are in automatic and function as designed.

Positive reactivity ( $\approx 100$  pcm) is added by rod withdrawal to establish an 0.25 dpm SUR. Which ONE of the following would occur assuming no further operator action?

- A. Reactor power would continue to increase at a constant rate until a reactor trip occurred at  $\approx 10\%$  power.
- B. Reactor power would continue to increase at a constant rate until a reactor trip occurred at  $\approx 25\%$  power.
- C. Reactor power would increase above the point of adding heat and stabilize.
- D. Reactor power would stabilize at the point of adding heat.

ANSWER C. Increasing/Decreasing Distractors N

- C. Reactor power would increase above the point of adding heat and stabilize.

## REFERENCE

Nupop Unit 1 Cycle 6

TI-44 "Boron Tables"; Table 7 - RCS @ 547F and 2235 psig. (Rev 7)

Total Points: 1.00 Estimated Completion Time: 2 SRO Only: N

## K/A Reference:

Task: 00400701

004000 K5.06 (3.0 - 3.3)  
 004000 K5.20 (3.6 - 3.7)  
 004010 A2.03 (3.9 - 4.2)  
 004000 A4.04 (3.2 - 3.6)  
 004000 A4.02 (3.2 - 3.9)

Lesson Plan/Objective: EGT222.023 / A (88 Requal)  
 OPL2739069 / B.3 (90 Requal Wk 4)  
 EGT222.023 / B.12 (89 REQUAL WK 5)

**QUESTION**

Given the following RCS parameters:

- PZR pressure 2210 psig.
- RCS wide range pressure = 2185 psig.
- Core exit T/Cs = 610°F.
- RCS hot leg temperature = 605°F.

Select the ONE response below which states the subcooling margin.

- A. 22°F
- B. 28°F
- C. 35°F
- D. 40°F

**ANSWER** D. Increasing/Decreasing Distractors Y

- D. 40°F

**REFERENCE**

Moved from B-0247  
NRC Generic Fundamentals Exam 6/28/89

Total Points: 1.00      Estimated Completion Time: 4      SRO Only: N

**K/A Reference:**      **Task:** 30100106  
006050 A1.02 (4.0 - 4.4)  
000009 ER1.02 (3.5 - 4.2)

**Lesson Plan/Objective:** OPL2739154 / B.2 (91 Requal Wk 5)

**QUESTION**

Which ONE of the following explains when a clearance may be released, by the SOS, with persons still holding that clearance?

- A. Plant emergency conditions exist when immediate action is needed to protect the health and safety of the public and plant personnel.
- B. The Radiological Emergency Plan is implemented and the Plant Manager orders the release.
- C. A National Emergency exists as defined in 10 CFR 50.34.
- D. The clearance is on a 161KV or 500KV breaker and the CLD can't be contacted.

**ANSWER** A. Increasing/Decreasing Distractors N

- A. Plant emergency conditions exist when immediate action is needed to protect the health and safety of the public and plant personnel.

**REFERENCE**

AI-3 Rev 52 pg 3/41  
Obtained from 1989 Week 4 Requal Bank

Total Points: 1.00      Estimated Completion Time: 2      SRO Only: Y

**K/A Reference:**      **Task:** 34400023  
194001 K1.02 (3.7 - 4.1)

**Lesson Plan/Objective:** OPL2738952 / B.1(89 Requal Wk 4)  
OPL2739140 / B.A (91 Requal Wk 5)