



**Commonwealth Edison**  
Byron Nuclear Station  
4450 North German Church Road  
Byron, Illinois 61010

September 14, 1992

Ltr: BYRON 92-0609

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.73(a)(2)(i)(B).

This report is number 92-005; Docket No. 50-455.

Sincerely,

R. Pleniewicz  
Station Manager  
Byron Nuclear Power Station

RP/CW/mw

Enclosure: Licensee Event Report No. 92-005

cc: A. Bert Da..., NRC Region III Administrator  
W. Kropp, NRC Senior Resident Inspector  
INPO Record Center  
CECo Distribution List

(0960R/VS)

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LICENSEE EVENT REPORT (LER)

Form Rev 2.0

Facility Name (1) Byron, Unit 2 Docket Number (2) 0 | 5 | 0 | 0 | 0 | 4 | 5 | 5 Page (3) 1 | of | 0 | 3

Title (4) LCOAR Inadvertently Not Entered

Event Date (5)			LER Number (6)			Report Date (7)			Other Facilities Involved (8)	
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
0   8	1   4	9   2	9   2	0   0   5	0   0	0   8	1   4	9   2	Byron, Unit 1	0   5   0   0   0   4   5   4

OPERATING MODE (9) 1

POWER LEVEL (10) 0 | 9 | 5

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input checked="" type="checkbox"/> Other (Specify in Abstract below and in Text)
<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

Name J. Penick, Technical Staff Engineer Ext. 2656 TELEPHONE NUMBER 3E | 5 | 2 | 3 | 4 | - | 5 | 4 | 4 | 1

R. Hopkins, Operating Engineer Ext. 2614

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
				N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

Expected Submission Date (15) 0 | 1 | 1 | 5 | 9 | 3

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

On 08/13/92, at approximately 0742, Mechanical Maintenance initiated repairs to Extraction Steam non-return check valve 2ES002, per Nuclear Work Request (NWR) B93935. Packing was adjusted to stop a steam leak. Work to the valve was completed on 08/13/92. On 08/14/92, while reviewing NWR B93935, the Unit 2 Shift Foreman noted that valve 2ES002 was a Technical Specification valve. The Shift Foreman immediately initiated 2BOS 3.4.2.c-1, "Extraction Steam Non-Return Check Valve Monthly Surveillance". The partial surveillance was successfully completed at 1714 on 08/14/92 and 2ES002 was declared operable.

The cause of this event was cognitive personnel error by the Operating Engineer who approved the NWR and failed to recognize the 2ES002 valve as Technical Specification related.

Immediate corrective action was successful partial performance of surveillance 2BOS 3.4.2.c-1 which showed valve operability. Any further corrective actions determined from further investigation will be delineated in a Supplemental Report.

This event is reportable per 10CFR50.73(a)(2)(i)(B), any operation or condition prohibited by the plants Technical Specifications.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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Byron, Unit 2	0   1   0   0   0   4   5   5	9   2	-   0   0   5	-   0   0	0   2	OF	0   3

TEXT Energy Industry Identification System (EIS) codes are identified in the text as [XX]

A. PLANT CONDITIONS PRIOR TO EVENT:Event Date/Time 08/14/92 / 1638Unit 1 MODE 1 - Power Operation Rx Power 97% RCS [AB] Temperature/Pressure NOT / NOPUnit 2 MODE 1 - Power Operation Rx Power 95% RCS [AB] Temperature/Pressure NOT / NOPB. DESCRIPTION OF EVENT:

On August 13, 1992, Mechanical Maintenance Department personnel began work on Extraction Steam Non-Return Check Valve 2ES002. Packing was adjusted to stop a steam leak. Work on the valve was completed that same day. The appropriate Limiting Condition for Operation Action Requirement (LCOAR), 2B05 3.4-1a, "LCOAR, Instrumentation, Turbine Overspeed Protection", was never entered. Action Requirement "b" of Technical Specification 3.3.4, Turbine Overspeed Protection, requires that the turbine must be isolated from the affected steam supply within 6 hours. Per Technical Specification Interpretation, this action requirement can be met by closing the associated Motor Operated Valve (MOV), 2ES001 or Manual Isolation Valves, 2ES003A and 2ES003B. No isolation was performed resulting in this action requirement not being met.

On August 14, 1992 at 1638 it was discovered by Operating Department personnel that LCOAR 2B05 3.4-1a had not been entered as required. A partial surveillance of 2B05 3.4.2.c-1, "Extraction Steam Non-Return Check Valve Monthly Surveillance", was immediately performed to verify operability of the valve. The surveillance was successfully completed.

No plant systems or components were previously inoperable that contributed to this event. This event is reportable per 10CFR50.73(a)(2)(i)(B), any operation or condition prohibited by the plant's Technical Specifications, in that LCOAR 2B05 3.4-1a was not entered and the action requirement was not satisfied.

C. CAUSE OF EVENT:

The cause of this event was cognitive personnel error by the acting Operating Engineer who approved the NWR. The NWR did not identify the valve as a technical Specification related component. Because the NWR was not marked as Technical Specification related, no further Senior Reactor Operator (SRO, licensed) review was required prior to beginning work on the valve. The mechanic who performed the work was only required to sign in with the Shift Engineer Assistant (non-licensed). He was not required to sign in with the Shift Engineer or his designated SRO licensed assistant because the NWR was not stamped as Technical Specification related.

D. SAFETY ANALYSIS:

Following the packing adjustment, the valve remained capable of performing its intended function as demonstrated by the partial surveillance of 2B05 3.4.2.c-1. The safety of the plant and the public was not affected by this event.

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TEXT Energy Industry Identification System (EIS) codes are identified in the text as [XX]

E. CORRECTIVE ACTIONS:

Immediate corrective action was successful partial performance of surveillance 2BOS 3.4.2.c-1 which showed valve operability.

Any further corrective actions determined from further investigation will be delineated in a Supplemental Report. NTS #4551809200500.

F. RECURRING EVENTS SEARCH AND ANALYSIS:

a) EVENT SEARCH (DIR, LER)

LER 50-454-92-002, LCOAR not entered due to lack of knowledge of equipment status.

b) INDUSTRY SEARCH (OPEX's NPRDS)

None found.

c) NHR

B93935

d) ANALYSIS

No trends currently identified. Although the current LER (455-92-005) and the previous LER (454-92-002) resulted in LCOARs not being entered, the causes were different.

G. COMPONENT FAILURE DATA:

MANUFACTURER	NOMENCLATURE	MODEL NUMBER	MFG PART NUMBER
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None.