DISTRIBUTION Central File RSB R/F JUN 0 3 1983 RSB S/F ABXRSXR&AGThomas GThomas R/F **BSheron** WHodges MEMORANDUM FOR: Domenic B. Vassallo, Chief, Operating Reactors Branch #2 Division of Licensing FROM: Brian W. Sheron, Chief, Reactor Systems Branch Division of Systems Integration SUBJECT: BWR SINGLE LOOP OPERATION AT 50% POWER Reference: Memo from George A. Schwenk to C. Berlinger, Chief

RSB agrees with the position that single loop operation (SLO) up to 50% power on a permanent basis can be approved under the conditions set forth in the SER and with the limits on decay ratio. However, we see no reason to exclude Browns Ferry from this generic approval. The RSB SER for SLO at Browns Ferry-1, 2 and 3 was issued on August 16, 1982.

CPB, DSI "CPB Position Regarding BWR Single Loop

Operation at 50% Power" dated May 24, 1983.

Original signed by: Brian W. Sheron

Brian W. Sheron, Chief Reactor Systems Branch Division of Systems Integration

- cc: R. Clark R. W. Houston
  - R. Mattson
  - L. Phillips
  - C. Berlinger
  - J. Hannon
  - G. A. Schwenk

| 1.2.1.2.1.1.1       |                     |                    | XP     | 4     |       |  |
|---------------------|---------------------|--------------------|--------|-------|-------|--|
| CONTACT:            | G. Thomas<br>X29445 | 1100               | 618    |       | a     |  |
| and a second second | - G                 | 306/4              | 51000  |       |       |  |
| DSI RSB             | DSTRSB              | USI RSB            |        |       | <br>  |  |
| GTHE As:gd          | WHodges 6/ 2 /83    | BSheron<br>6/3 /83 |        | ••••• | <br>  |  |
|                     |                     | OFFICIAL           | RECOPD | COPY  | <br>- |  |

RSB P/F: James A. Fitzpatrick

0

GThomas WHodges BSheron AD:RS Rdg. GThomas R/F

#### AUG 2 9 1983

Docket No.: 50-333

MEMORANDUM FOR: Gus C. Lainas, Assistant Director for Operating Reactors, DL

FROM: R. Wayne Houston, Assistant Director for Reactor Safety, DSI

SUBJECT: SAFETY EVALUATION OF JAMES A. EITZPATRICK NUCLEAR POWER PLANT, SINGLE LOOP OPERATION

| Plant Name:                           | James A. Fitzpatrick      | Nuclear | Power | Plant |
|---------------------------------------|---------------------------|---------|-------|-------|
| NSSS Supplier:<br>Responsible Branch: | General Electric<br>ORB-2 |         |       |       |
| Project Manager:<br>Review Status:    | J. Hegner<br>Complete     |         |       |       |

Enclosed is the Safety Evaluation Report to permit James A. Fitzpatrick Nuclear Power Plant to operate on a single loop with power limited to 50%. This completes one action under multiplant item E-04. (TAC #49371).

> Chicinal Picesd By G. Weyns Nouston

R. Wayne Houston, Assistant Director for Reactor Safety Division of Systems Integration

Enclosure: As stated cc: R. Mattson L. Phillips G. Schwenk Richard Clark D. B. Vassallo T. Marsh J. Rosenthal N. Lauben J. Hegner CONTACT: G. Thomas, RSB X29445 ASI : RSB DSI:AD:RS DSI:RSB DSI:RSB Af.ALU CBerlinger BWSheron RWHouston WHodges GThomas:jf 81, 4/83 8/ ~1/83 8/12/83 8/25/83 8/22/83 RECORD COPY OFFICIAL

#### ENCLOSURE

### SAFETY EVALUATION REPORT N-1 LOOP OPERATION JAMES A. FITZPATRICK NUCLEAR POWER PLANT (JAF)

#### 1.0 INTRODUCTION

٠.

The current JAF Technical Specifications do not allow plant operation beyond 24 hours if an idle recirculation loop can not be returned to service. The ability to operate at reduced power with a single loop is highly desirable from availability/outage planning standpoint in the event that maintenance or component unavailability rendered one loop inoperable.

By letter dated December 29, 1982 Power Authority of the State of New York (PASNY) (the licensee) requested changes to the Technical Specification for Single Loop Operation of JAF. The requested changes would permit JAF to operate at up to 50% of rated power with one recirculation loop out of service for unlimited time. While analyses indicate that it may be safe to operate BWRs on a single loop in the range higher than 50% of rated power, the experience (reference letter from L. M. Mills, TVA dated March 17, 1980 to H. Denton, NRC) at Browns Ferry Unit 1 has caused concern about flow and power oscillations. However, because single loop operation at 50% rated power at several plants, including Browns Ferry Plant Unit 1, has shown acceptable flow and power characteristics, we will permit PASNY to operate at power levels up to 50% of rated with one loop out of service.

-2--

If requested, we will reconsider operation at a higher power level for JAF with one recirculation loop out of service after staff concerns stemming from Browns Ferry - Unit 1 single loop operation are satisfied.

#### 2 EVALUATION

1

### 2.1 <u>Accidents (Other than Loss of Coolant Accident (LOCA)</u> and Transients Affected by One Recirculation Loop Out of Service

#### 2.1.1 One Pump Seizure Accident

The licensee states that the one-pump seizure accident is a relatively mild event during two recirculation pump operation. Similar analyses were performed to determine the impact this accident would have on one recirculation pump operation. These analyses were performed using NRC approved models for a large core BWR/4 plant. The analyses were conducted from steady-state operation at the following initial conditions, with the added condition of one inactive recirculation loop. Two sets of initial conditions were assumed:

a. Thermal Power = 75% and core flow = 58% of rated
b. Thermal Power = 82% and core flow = 56% of rated

These conditions were chosen because they represent reasonation upper limits of single-loop operation within existing Maximu-Average Planar Linear Heat Generation Rate (MAPLHGR) and Min-Critical Power Ratio (MCPR) limits at the same maximum pump speed. Pump seizure was simulated by setting the single operating Pump speed to zero instantaneously.

-3-

The anticipated sequence of events following a recirculation pump seizure which occurs during plant operation with the alternate recirculation loop out of service is as follows:

- a. The recirculation loop flow in the loop in which the pump seizure occurs drops instantaneously to zero.
- b. Core voiding increases which results in a negative reactivity insertion and sharp decrease in neutron flux.
- c. Heat flux drops more slowly because of the fuel time constant.
- d. Neuton flux, heat flux, reactor water level, steam flow, and feedwater flow all exhibit transient behavior. However, it is not anticipated that the increase in water level will cause a turbine trip and result in scram.

The transient will terminate at a condition of natural circus and reactor operation will continue. There will also be a small decrease in system pressure.

The licensee concludes that the MCPR for the pump seizure accident for the large core BWR/4 plant was determined to be greater than the fuel cladding integrity safety limit; therefore, no fuel failures were postulated to occur as a result of this analyzed event. These results are applicable to JAF.

#### 2.1.2 Abnormal Transients

#### 2.1.2.1 a. Idle Loop Startup

The idle loop startup transient was analyzed, in the JAF FSAR, with an initial power of 65%. The licensee is to operate at no greater than 50% power with one loop out of service. Additionally, the Technical Specifications are being modified to require that, during single loop operation, the suction valve in the idle loop be shut and electrically disconnected. These measures are being taken to preclude startup of an idle loop.

#### b. Flow Increase

For single-loop operation, the rated condition steady-state MCPR limit is increased by 0.01 to account for increased uncertainties in the core total flow and Traversing In-core Probe (TIP) There will also be a small decrease in system pressure.

The licensee concludes that the MCPR for the pump seizure accident for the large core BWR/4 plant was determined to be greater than the fuel cladding integrity safety limit; therefore, no fuel failures were postulated to occur as a result of this analyzed event. These results are applicable to JAF.

#### 2.1.2 Abnormal Transients

#### 2.1.2.1 a. Idle Loop Startup

The idle loop startup transient was analyzed, in the JAF FSAR, with an initial power of 65%. The licensee is to operate at no greater than 50% power with <u>one</u> loop out of service. Additionally, the Technical Specifications are being modified to require that, during single loop operation, the suction valve in the idle loop be shut and electrically disconnected. These measures are being taken to preclude startup of an idle loop.

#### b. Flow Increase

For single-loop operation, the rated condition steady-state MCPR limit is increased by 0.01 to account for increased uncertainties in the core total flow and Traversing In-core Probe (TIP) readings. The MCPR will vary depending on flow conditions.
 This leads to the possibility of a large inadvertent flow increase which could cause the MCPR to decrease below the Safety Limit for a low initial MCPR at reduced flow
 conditions. Therefore, the required MCPR must be increased at reduced core flow by a flow factor K<sub>f</sub>. The K<sub>f</sub> factors are derived assuming both recirculation loop pumps increase speed to the maximum permitted by the M-G set scoop tube position set screws. This condition maximizes the power increase and hence maximum A MLPR for transients initiated from less than rated conditions. When operating on one loop the flow and power increase will be less than associated with two pumps increasing speed, therefore, the K<sub>f</sub> factors derived from the two-pump assumption are conservative for single loop operation.

#### c. Rod Withdrawal Error

The rod withdrawal error at rated power is given in the FSAR for the initial core and in cycle dependent reload supplemental submittals. These analyses are performed to demonstrate that, even if the operator ignores all instrument indications and the alarm which could occur during the course of the transient, the rod block system will stop rod withdrawal at a minimum critical power ratio which is higher than the fuel cladding integrity safety limits. Correction of the rod block equation and lower initial power for single-loop operation assures that the MCPR safety limit is not violated.

-5-

- 3.

One-pump operation results in backflow through jet pumps while flow is being supplied to the lower plenum from the active jet pumps. Because of this backflow through the inactive jet pumps the present rod-block equation and APRM settings must be modified. The licensee has modified the two-pump rod block equation and APRM settings that exists in the Technical Specification for one-pump operation and the staff has found them acceptable.

-6-

----

The staff finds that one loop transients and accidents other than LOCA, which is discussed below, are bounded by the two loop operation analysis and are therefore acceptable.

#### 2.2 Loss of Coolant Accident (LOCA)

The licensee has contracted General Electric Co. (GE) to perform single loop operation analysis for JAF LOCA. The licensee states that evaluation of these calculations (that are performed according to the procedure outlined in NEDO-20556-2, Rev. 1) indicates that a multiplier of 0.84 (8x8R, P8x8R) and 0.85 (8x8 Fuel) (Ref: -NEDO 24281 August 1980, Errata and Addenda Sheet No. 1, December 1980) should be applied to the MAPLHGR limits for single loop operation of JAF. We find the use of these MAPLHGR multipliers to be acceptable.

#### 3. THERMAL HYDRAULICS

The licensee has confirmed that analysis uncertainties are independent of whether flow is provided by two loops or single loop. The only exceptions to this are core total flow and TIP reading. The effect of these uncertainties is an increase in the MCPR by .01, which is more than offset by the K<sub>f</sub> factor required at low flows. The steady state operating MCPR with single-loop operation will be conservatively established by multiplying the K<sub>f</sub> factor to the rated flow MCPR limit.

· · · · · · · · · · ·

#### 4. STABILITY ANALYSIS

The licensee will be required to operate in master manual to reduce the effects of instabilities due to controller feedback. The staff has accepted previous stability analyses results as evidence that the core can be operated safely while our generic evaluation of BWR stability characteristics and analysis methods continues. Our acceptance is based on the following:

> The licensee stated that oscillatory problems have <u>not</u> been experienced at the JAF Station during single loop operation.

> The licensee will monitor APRM flux noise and core plat delta P noise at about 40 percent of power for a period of 1/2 hour to 1 hour to establish a baseline noise level. This baseline noise level (peak to peak oscilla will be increased by 50 percent to establish a maximum

allowable level. The noise will be measured once per shift and the recirculation pump speed will be reduced if the flux noise exceeds the maximum allowable level. The maximum allowable power level during SLO will be 50 percent.

We conclude that with appropriate limitations to recognize and avoid operating instabilities, the reactor can be operated safely in the single loop mode. Our evaluation of the flow/power oscillations evidenced in Browns Ferry will continue and any pertinent conclusions resulting from this study will be applied to JAF.

## 5. SUMMARY ON SINGLE LOOP OPERATION

1. <u>Steady State Thermal Power Level will not</u> <u>exceed 50%</u>. Operating at 50% power with appropriate TS changes was approved on a cycle basis for Pilgrim 1, Cooper Nuclear Station, Monicello Nuclear Generating Station and Brunswick Units 1 & 2 (Safety Evaluation Reports (SER) dated December 15, 1981, December 10, 1981, September 10, 1982 and December 21, 1982 respectively). Authorization for single loop operation for extended periods was also given to Dresden Unit 2 and 3, Quad Cities Units 1 and 2, Peach Bottom Units 2 and 3 and Duane Arnold (SER July 9, 1981, SER November 19, 1981). It was concluded that for operation at 50%, power transient and accident bounds would not be exceeded for these plants.

### 2. <u>Minimum Critical Power Ratio (MCPR) Safety</u> Limit will be Increased by 0.01 to 1.08

÷. • \* ; •

The MCPR Safety Limit will be increased by 0.01 to account for increased uncertainties in core flow and Traversing Incore Probe (TIP) readings. The licensee has reported that this increase in the MCPR Safety Limit was addressed in GE reports specifically for JAF for one loop operation. On the basis of previous staff reviews for Pilgrim 1. Cooper. Duane Arnold. Monticello and Peach Bottom and our review of plant comparisons we find this analysis acceptable for JAF.

### 3. <u>Minimum Critical Power Ratio (MCPR) Limiting</u> <u>Condition for Operation (LCO) will be Increased</u> by 0.01

The staff requires that the operating limit MCPR be increased by 0.01 and multiplied by the appropriate two loop K<sub>f</sub> factors that are in the JAF TS. This will preclude an inadvertent flow increase from causing the MCPR to drop below the safety limit MCPR. This was also approved by the staff for Peach Bottom 2 and 3.

### 4. <u>The Maximum Average Planer Linear Heat Generation</u> <u>Rate (MAPLHGR) Limits will be Reduced by Appropriate</u> <u>Multipliers</u>

The licensee proposed reducing the TS MAPLHGR by 0.84 (8x8R, Px8x8R Fuel), 0.85 (8x8 Fuel) for Single Loop Operation. These reductions

were based on analyses by General Electric (GE) in reports NEDE 24011-P-A-1 and NEDO 24281. The Peach Bottom units were allowed to operate with their MAPLHGR values reduced by factors of 0.71, 0.83, and 0.81 for an unlimited period of time.

10

### 5. The APRM Scram and Rod Block Setpoints will be Reduced

The licensee proposed to modify the two loop APRM Scram, Rod Block and Rod Block Monitor (RBM) setpoints to account for back flow through half the jet pumps. The changes were based on plant specific analyses by GE. These setpoint equations will be changed in the JAF TS. The above changes are similar to the Peach Bottom TS changes and are acceptable to the staff.

### 6. <u>The Suction Valve in the Idle Loop is Closed and</u> Electrically Isolated

The licensee will close the recirculation pump suction valve and remove power from the valve. In the event of a loss of coolant accident this would preclude partial loss of LPCI flow through the recirculation loop degrading the intended LPCI performance.

The removal of power also helps to preclude a startup of an idle loop transient.

# 7. The Recirculation Control will be in Manual

-11-

#### Control

The staff requires that the licensee operate the recirculation system in the manual mode to eliminate the need for control system analyses and to reduce the effects of potential flow instabilities. This was also required of Peach Bottom.

### 8. Surveillance Requirements

The staff requires that the licensee perform daily surveillance on the jet pumps to ensure that the pressure drop for one jet pump in a loop does not vary from the mean of all jet pumps in that loop by more than 5%.

### 9. <u>Provisions to Allow Operation with One</u> Recirculation Loop Out of Service

- The steady-state thermal power level will not exceed 50% of rated
- 2. The Minimum Critical Power Ratio (MCPR) Safety Limit will be increased by .01 to 1.08 (T.S. 3.1 B)
- The MCPR Limiting Condition for Operation (LCO) will be increased by 0.01
- The Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) Limits will be reduced.

| Fuel Type | Reduction Factor |
|-----------|------------------|
| 8 . 8     | 0.85             |
| 8×8R      | 0.84             |
| P8x8R     | 0.84             |

~

5.

The APRM Scram and Rod Block Setpoints and the RBM Setpoints, shall be reduced to read as follows:

·. · \*.

- T.S. 2.1.C(1) S≤ (.66W + 54% -0.66 ΔW)
  T.S. 2.1.C(1)\* S≤ (.66W + 54% -0.66 ΔW) TPF(FRP)/MTPF(MF
  T.S 2.1.A.1.D S≤ (.66W + 42% -.66 ΔW)
  T.S. 2.1.A.1.D\*S≤ (.65W + 42% -.66 ΔW) TPF(FRP)/MTPF(MF
  APRM Upscale (.66W + 42% -0.66 ΔW)
  RBM Upscale (.66W + K -0.66 ΔW) (K= Intercept values of 39%, 40%, 41%, 42%, 43% and 44% can be used with Appropriate MCPR Limits From Section 3.1.B)
  - \* In the event MFLPD exceeds FRP
- 6. The suction value in the idle loop is closed and electrically isolated until the idle loop is being prepared for return to service.
- 7. The licensee will monitor APRM flux noise and core plate delta P noise at about 40 percent of power for a period of 1/2 hour to 1 hour to establish a baseline noise level. This baseline noise level (peak to peak oscillation) will be increased by 50 percent to establish a maximum allowable level. The noises will be measured once per shift and the recirculation pump speed will be reduced if the flux noise exceeds the maximum allowable level.

Based upon the above evaluation and a history of suscessful operation of other BWRs of the same type as JAF, we conclude... that single-loop operation of JAF up to a power level. of 50% and in accordance with the proposed TSs, will not exceed the accident and transient bounds previously found acceptable by the NRC staff and is therefore acceptable.

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered or create a possibility of an accident of a type different from any evaluated previously, and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

-13-

- ....