



September 14, 1992

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U. S. Nuclear Regulatory Commission
Document Control Desk
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Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Small Break LOCA Analysis

Gentlemen:

During a conference call with the NRC Staff on August 20, 1992, Entergy Operations was advised to upgrade the Arkansas Nuclear One, Unit 2 (ANO-2) small break loss of coolant accident (SBLOCA) analysis to reflect a quantitative evaluation of the spectrum of break sizes if future modifications are made to the facility which would have a substantial impact on the existing ANO SBLOCA model. The NRC request was based on a recent review conducted by the NRC Staff which concluded that the current SBLOCA analyses are adequate and do not currently pose a safety question for continued ANO-2 operation, but that future plant modifications which could affect SBLOCA analyses may require additional quantitative analysis for smaller break sizes.

Entergy Operations mutually recognizes the NRC's desire to apply more quantitative SBLOCA analyses for ANO-2 and has been participating in the ABB-CE Owners Group to develop the Realistic Evaluation Model (REM) methodology since mid-1991. Once approved by the NRC Staff, this improved methodology will address the NRC's concern for a more quantitative spectrum of break sizes. Entergy Operations requests that continued NRC coordination with ABB-CE and expeditious review of the new REM methodology be pursued to reach a mutual resolution.

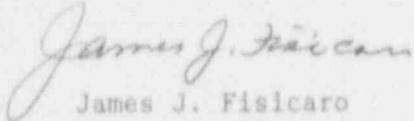
However, it is understood that for future SBLOCA evaluations necessary to support facility modifications, additional quantitative evaluations addressing the SBLOCA spectrum may be necessary if the REM methodology has not been accepted by the NRC Staff. Such quantitative analysis would specifically involve the "limiting small break" (i.e., break size solely controlled by HPSI flow). Entergy Operations wishes to express that, given the need to reanalyze any condition or modification that may affect safety, it is our practice to conduct evaluations and analyses sufficiently to assure ourselves that plant safety is maintained. Those evaluations include a review of analysis assumptions and applicability to the condition or proposed modification. This practice will be continued independent of the NRC Staff's request.

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Please notify me if you require any additional information.

Very truly yours,



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