



GPU Nuclear Corporation  
Post Office Box 480  
Route 441 South  
Middletown, Pennsylvania 17057-0191  
717 944-7621  
TELEX 84-2386  
Writer's Direct Dial Number:

(717) 948-8005

September 14, 1992  
C311-92-2124

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)  
Operating License No. DPR-50  
Docket No. 50-289  
Monthly Operating Report for August 1992

Enclosed are two copies of the August 1992 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

T. G. Brughton  
Vice President and Director, TMI-1

WGH

Attachments

cc: Administrator, Region I  
TMI Senior Resident Inspector

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PDR ADOCK 05000289  
R PDR

OPERATIONS SUMMARY  
AUGUST 1992

The plant entered the month operating at 100% power producing 831 MWe. It continued at that power level until August 20, when reactor power was increased an additional 1.6% through the implementation of a feedwater flow correction factor. A new, more accurate full power level is achieved by compensating for feedwater flow nozzle fouling with the correction factor. The plant continued to operate at 100% power until August 31. A planned reduction to 50% power was completed at the close of this reporting period due to condenser fouling caused by Asiatic clam buildup on the condenser tube sheets.

On August 26, fire watches were established in plant areas where Thermo-Lag fire barrier envelopes are installed. The compensatory action was taken in response to information provided by the NRC in Bulletin 92-01 and Bulletin 92-01, Supplement 1.

MAJOR SAFETY RELATED MAINTENANCE

During August, the following major safety related maintenance activities were performed:

Decay Heat Removal Pump DH-P-1B

Decay Heat Removal Pump DH-P-1B was removed from service for a scheduled system outage. While the pump/motor coupling spool was removed for coupling lubrication and a coupling alignment check, internal rubbing was noted during hand rotation of the pump. The pump was subsequently disassembled and inspected. Inspection revealed slight scratches on the impeller and casing wearing ring surfaces and a small amount of metal filings in the vicinity of the wear ring in the bottom of the pump casing. There were no deficiencies found on either the bearings or shaft. After the impeller and casing wear ring clearances were measured and found to be within specified tolerances, they were hand "dressed". The pump rotating assembly was reinstalled in the casing and the pump shaft was rotated during the torquing of the casing bolts. The rubbing recurred prior to attaining the specified torque value.

Because there was no evidence of binding or change in pump vibration before the pump was taken out of service and the coupling spool piece removed, it was decided to reinstall the coupling spool piece and rotate the pump shaft. With the coupling spool reinstalled, the pump shaft rotated freely with no evidence of rubbing or binding. The pump/motor coupling alignment was found to be within specified limits and DH-P-1B reassembly work was completed. Post maintenance testing was completed with satisfactory vibration levels and required head and flow rate. The pump was returned to service. Plant Engineering is investigating the cause of the rubbing.

#### Miscellaneous Waste Transfer Pump WDL-P-7B

Miscellaneous Waste Transfer Pump WDL-P-7A was removed from service to repair mechanical seal leakage. While the pump was disassembled, components were inspected for damage. The bearings were found worn and were determined to be the cause of the seal leakage. The pump shaft, bearings, oil seals, and mechanical seal were replaced during the reassembly of WDL-P-7A. The pump was returned to service after post maintenance testing was completed.

#### Reactor Protection System (RPS)

During routine RPS testing, the "B" Loop T-Hot trip circuit did not trip as designed. The "blue ribbon" connectors were exercised and when the system was retested, the trip circuit tripped as designed. As required by Technical Specifications, the RPS "A", "C", and "D" Loop T-Hot trip functions were tested with no discrepancies. Subsequent analysis determined that the conservative course of action was to replace the "B" Loop T-Hot bistable module. The bistable was replaced and the system was returned to service.

OPERATING DATA REPORT

DOCKET NO. 50-289  
 DATE September 14, 1992  
 COMPLETED BY W G HEYSEX  
 TELEPHONE (717) 948-8191

OPERATING STATUS

- 1. UNIT NAME: THREE MILE ISLAND UNIT 1
- 2. REPORTING PERIOD: AUGUST 1992
- 3. LICENSED THERMAL POWER: 2568
- 4. NAMEPLATE RATING (GROSS MWe): 871
- 5. DESIGN ELECTRICAL RATING (NET MWe): 819
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: \_\_\_\_\_

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): \_\_\_\_\_

10. REASONS FOR RESTRICTIONS, IF ANY: \_\_\_\_\_

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
		-----	-----	-----
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	5855.0	157776.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	5855.0	82586.0
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2245.6
14. HOURS GENERATOR ON-LINE	(HRS)	744.0	5855.0	81511.2
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1906278	15015404	198884449
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	623941	4972179	66966438
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	589046	4694530	62851855
19. UNIT SERVICE FACTOR	(%)	100.0	100.0	51.7
20. UNIT AVAILABILITY FACTOR	(%)	100.0	100.0	51.7
21. UNIT CAPACITY FACTOR	(USING MDC NET)	100.7	102.0	50.7
22. UNIT CAPACITY FACTOR	(USING DER NET)	96.7	97.9	48.6
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	42.7
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60648.7
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289  
UNIT TMI-1  
DATE September 14, 1992  
COMPLETER ~ W G HEYSEK  
TELEPHONE (717) 948-8191

MONTH: AUGUST

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
1	791	17	791
2	791	18	787
3	787	19	791
4	788	20	800
5	794	21	806
6	794	22	803
7	793	23	801
8	790	24	797
9	782	25	791
10	784	26	788
11	782	27	786
12	789	28	783
13	791	29	801
14	792	30	799
15	794	31	783
16	794		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH August 1992

DOCKET NO. 50-289  
 UNIT NAME TMI-1  
 DATE September 14, 1992  
 COMPLETED BY W. G. Heysek  
 TELEPHONE (717) 948-8191

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report#	System Code ' & '	Component Code ' & '	Cause & Corrective Action to Prevent Recurrence
92-01	8/31/92	F	0	B	4	NONE	TBD	TBD	Condenser macrofouling (Asiatic clams) led to a 50% power reduction for nearly five days beginning 8/31/92 to accomplish tube and waterbox cleaning. The subsequent preventative actions include installation of smaller mesh screens in the cooling water system and increased frequency of "clam kill" chemistry treatment of the system.

1  
 F Forced  
 S Scheduled

2  
 Reason  
 A-Equipment Failure (Explain)  
 M-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & Licensing Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 R-Other (Explain)

3  
 Method  
 1-Manual  
 2-Manual Scram  
 3-Automatic Scram  
 4-Other (Explain)

4  
 Exhibit G - Instructions for preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)  
 5 Exhibit 1 same source  
 6 Actually used exhibits F & II NUREG 0161



REFUELING INFORMATION REQUEST

1. Name of facility: **Three Mile Island Nuclear Station, Unit 1**
2. Scheduled date for next refueling shutdown: **September 17, 1993 (10R)**
3. Scheduled date for restart following current refueling: **NA**
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **NA**

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

**None planned.**

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

GPU Nuclear has installed four Westinghouse Lead Test Assemblies during the reload of the TMI-1 core for cycle 9 operation. Westinghouse fuel technology will be utilized to the extent possible.

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 521

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1496. Reracking of spent fuel pool 'A' to attain the licensed capacity is in progress.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

The 9R (1991) refueling discharge was the last to allow full core off-load capacity (177 fuel assemblies). Upon completion of the reracking project, full core off-load is assured through the end of the current operating license and beyond.