

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3160-0104
EXPIRES - 8/31/85

FACILITY NAME (1) Peach Bottom Atomic Power Station - Unit 2 DOCKET NUMBER (2) 05101010277 PAGE (3) 1 OF 013

TITLE (4) Exceeding Local Leak Rate Test Allowable Limit

EVENT DATE (6)			LER NUMBER (8)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (9)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER (5)	
01	08	85	85	001	00	01	31	08		0510101011	
										0510101011	

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)

OPERATING MODE (9) N	20.402(b)	20.406(a)	60.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 000	20.406(a)(1)(ii)	60.38(a)(1)	60.73(a)(2)(v)	73.71(a)
	20.406(a)(1)(v)	60.38(a)(2)	60.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 206-U)
	20.406(a)(1)(iii)	60.73(a)(2)(ii) X	60.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	60.73(a)(2)(iv)	60.73(a)(2)(viii)(B)	
	20.406(a)(1)(vi)	60.73(a)(2)(iii)	60.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
J. C. Nagle, Engineer - Special Projects	2115 8411-151814

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if you complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15) MONTH 06 DAY 01 YEAR 85

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 2-85-01

On January 8, 1985, the containment leakage test program identified that the total combined leakage of the Type B and C tests exceeded the allowable leak rate limit. Several valves tested had excessive leakage rates which contributed to the combined leakage total. These valves will be repaired and retested prior to returning Unit 2 to power operation.

8502060523 850130
PDR ADOCK 05000277
S PDR

IE 22
111

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Peach Bottom Atomic Power Station - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 7 7					LER NUMBER (6)			PAGE (3)		
						YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
						8 5	0 0 1	0 0	0 2	OF	0 3

TEXT (if more space is required, use additional NRC Form 366A's) (17)

Description of the Event:

On January 8, 1985, it was determined that the combined leakage rate of all penetrations and valves subject to Type B and C local leak rate tests had exceeded the allowable limit of 0.6 La. La is defined as 0.5%/day leakage of the contained volume of gas in containment at peak accident pressure. For Peach Bottom Unit 2, 0.6 La is 71,186 scc/min at 49.1 psig.

The following valves were identified as having excessive leakage rates: Chilled Water Valves MO-2200A, MO-2200B, MO-2201A; Outboard Drywell Floor Drain Isolation Valve AO-2-20-83; Scram Discharge Volume Outboard Vent Valve AO-2-3-35A; 'A' ADS Backup Instrument Nitrogen Check Valve; and Feedwater Check Valve 2-6-28B.

Consequences of the Event:

Each valve listed, with the exception of the drywell chilled water valves, has a redundant series isolation valve. The local leak rate test results of the redundant isolation valves were acceptable, therefore, containment integrity would have been maintained.

The actual leakage of the drywell chilled water valves would be substantially less than the local leak rate test values since these valves isolate a water-filled system. Although the valves are normally water-bound, they are conservatively tested with air during the local leak rate tests.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Peach Bottom Atomic Power Station - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 7 7	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		85	-0 0 1	-0 0	0 3	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Cause of the Event:

The cause of the excessive valve leakage will be determined when the valves are disassembled and inspected. The results of these inspections will be submitted in a follow-up LER report.

Corrective Actions:

Unit 2 is presently in a refueling outage. All valves with unacceptable leak rates will be repaired and retested prior to returning Unit 2 to operation. These corrective actions will ensure that the total combined leakage rate for all Type B and C tests will be below 0.6 La.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET
P.O. BOX 8699
PHILADELPHIA, PA. 19101
(215) 841-4000

January 30, 1985

Docket No. 50-277

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report
Peach Bottom Atomic Power Station - Unit 2

This LER deals with exceeding the total leakage rate limit for Type B and C tests.

Reference: Docket No. 50-277
Report Number: 2-85-01
Revision Number: 00
Event Date: January 8, 1985
Report Date: January 30, 1985
Facility: Peach Bottom Atomic Power Station
RD #1, Box 208, Delta, PA 17314

This letter is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator
Region I, USNRC

Mr. T. P. Johnson, Resident Inspector

LE22
1/1