



Public Service Electric and Gas Company, P.O. Box 236 Hancocks Bridge, New Jersey 08038
Hope Creek Generating Station

September 11, 1992

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATION STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for August are being forwarded to you along with the summary of changes, tests, and experiments for August 1992 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

J. J. Hagan
General Manager -
Hope Creek Operations

Ben RSC

RAR:ld
Attachments

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-354
UNIT Hope Creek
DATE 9/11/92
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

MONTH August 1992

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. 1038
2. 1012
3. 1042
4. 1034
5. 1039
6. 1041
7. 1041
8. 1035
9. 1019
10. 1026
11. 1025
12. 1037
13. 1042
14. 1042
15. 1042
16. 1029

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17. 1035
18. 1033
19. 1047
20. 1040
21. 1046
22. 1040
23. 1018
24. 1039
25. 1031
26. 1024
27. 1027
28. 1024
29. 1039
30. 1034
31. 1034

OPERATING DATA REPORT

DOCKET NO. 50-354
 UNIT Hope Creek
 DATE 9/11/92
 COMPLETED BY V. Zabielski *VZ*
 TELEPHONE (609) 339-3506

OPERATING STATUS

1. Reporting Period August 1992 Gross Hours in Report Period 744

2. Currently Authorized Power Level (MWt) 3293
 Max. Depend. Capacity (MWe-Net) 1031
 Design Electrical Rating (MWe-Net) 1067

3. Power Level to which restricted (if any) (MWe-Net) None

4. Reasons for restriction (if any)

	<u>This</u> <u>Month</u>	<u>Yr To</u> <u>Date</u>	<u>Cumulative</u>
5. No. of hours reactor was critical	<u>744.0</u>	<u>5537.5</u>	<u>42,698.8</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
7. Hours generator on line	<u>744.0</u>	<u>5475.4</u>	<u>42,050.0</u>
8. Unit reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. Gross thermal energy generated (MWH)	<u>2,443,707</u>	<u>17,650,961</u>	<u>133,648,104</u>
10. Gross electrical energy generated (MWH)	<u>804,080</u>	<u>5,864,090</u>	<u>44,216,584</u>
11. Net electrical energy generated	<u>768,896</u>	<u>5,605,288</u>	<u>42,256,837</u>
12. Reactor service factor	<u>100.0</u>	<u>94.6</u>	<u>85.5</u>
13. Reactor availability factor	<u>100.0</u>	<u>94.6</u>	<u>85.5</u>
14. Unit service factor	<u>100.0</u>	<u>93.5</u>	<u>84.2</u>
15. Unit availability factor	<u>100.0</u>	<u>93.5</u>	<u>84.2</u>
16. Unit capacity factor (using MDC)	<u>100.2</u>	<u>92.9</u>	<u>82.0</u>
17. Unit capacity factor (Using Design MWe)	<u>96.9</u>	<u>89.7</u>	<u>79.3</u>
18. Unit forced outage rate	<u>0.0</u>	<u>2.3</u>	<u>4.8</u>
19. Shutdowns scheduled over next 6 months (type, date, & duration): Refueling outage, 9/12/92, 60 days			
20. If shutdown at end of report period, estimated date of start-up: N/A			

OPERATING DATA REPORT
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-354
UNIT Hope Creek
DATE 9/11/92
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

MONTH August 1992

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REAC OR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENTS
						None

Summary

REFUELING INFORMATION

DOCKET NO. 50-354
 UNIT Hope Creek
 DATE 9/11/92
 COMPLETED BY S. Hollingsworth
 TELEPHONE (609) 339-1051

MONTH August 1992

1. Refueling information has changed from last month:

Yes No

2. Scheduled date for next refueling: 9/12/92

3. Scheduled date for restart following refueling: 11/11/92

4. A. Will Technical Specification changes or other license amendments be required?

Yes No

B. Has the reload fuel design been reviewed by the Station Operating Review Committee?

Yes No

If no, when is it scheduled?

5. Scheduled date(s) for submitting proposed licensing action: N/A

6. Important licensing considerations associated with refueling:

- Same fresh fuel as current cycle: no new considerations

7. Number of Fuel Assemblies:

A. Incore	<u>764</u>
B. In Spent Fuel Storage (prior to refueling)	<u>760</u>
C. In Spent Fuel Storage (after refueling)	<u>1008</u>

8. Present licensed spent fuel storage capacity: 4006

Future spent fuel storage capacity 4006

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: 11/4, 2010
 (EOC16)
 (does not allow for full-core off...)

HOPE CREEK GENERATING STATION

MONTHLY OPERATING SUMMARY

August 1992

Hope Creek entered the month of August at approximately 100% power. The unit operated for the entire month without experiencing any shutdowns or reportable power reductions. As of August 31, the plant had been on line for 78 consecutive days.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE HOPE CREEK GENERATING STATION

AUGUST 1992

The following items have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

DCP

Description of Safety Evaluation

4EC-3254/01

This DCP added two ventilation fans to each Filtration, Recirculation, and Ventilation System Ventilation System unit heater control panel. It also changed some internal wires, removed relays, and removed Temporary Modifications that bypassed the disconnect switches.

This DCP does not change the function of the Filtration, Recirculation, and Ventilation System Ventilation System. This DCP provides additional cooling for the components in the Ventilation System heater control panels. It does not alter the operation of the system in meeting its accident mitigation function. Therefore, this DCP does not involve any Unreviewed Safety Questions.

4EC-3329/01

This DCP allows the replacement of the Service Water Strainer elements with either the existing strainer element or an alternative strainer element design. The new design will provide increased structural stability with respect to the element over the convoluted perforated element design.

The safety functions, control, and operation of the Service water system remain unchanged. The replacement strainer element provides Service Water system protection similar to the original strainer element. The Service Water Pumps have instrumentation that sense shaft vibration and temperature. These instruments give an early indication of potential wear. Therefore, this DCP does not involve any Unreviewed Safety Questions.

4HX-0331/01

This DCP replaces mechanical snubbers with hydraulic snubbers which do not require seal maintenance and have a 40 year plant life. After the next outage, it is planned that these snubbers be removed and subjected to functional tests and evaluations. This test and experiment DCP will help to determine if 1130 mechanical snubbers will be replaced with hydraulic snubbers.

The hydraulic snubbers have the same design load, nuclear qualifications, and thermal and seismic piping application as the mechanical snubbers and will perform the same functions. Therefore, this DCP does not involve any Unreviewed Safety Questions.

TMR

Description of Safety Evaluation

92-020

This TMR installed Control Air tubing between a pressure control valve in the Gaseous Radwaste system and its associated instrumentation. This TMR will allow operators to restore system pressure within design parameters until equipment problems can be resolved.

The SAR bounds gross equipment failure due to hydrogen detonations or seismic events of greater magnitude than design. This TMR does not affect the seismic or explosion resistance of the system; therefore, it does not involve any Unreviewed Safety Questions.

Procedure
Revision

HC.SA-AP.ZZ-0052(Q)
Rev 7

Description of Safety Evaluation

This procedure revision provides guidance for the station departments involved in ensuring that water chemistry parameters are maintained in accordance with the appropriate vendor and industry guidelines.

Cracking problems in BWR austenitic stainless steel piping systems are a result of concurrent aggressive water environment, material susceptibility, and stress conditions. Establishing and maintaining appropriate water chemistry conditions improves plant availability and minimizes personnel radiation exposure. This procedure revision considers the most recent knowledge of how water chemistry parameters affect fuel performance, radiation field buildup, and pipe cracking. Therefore, this procedure does not involve an Unreviewed Safety Question.

HC.SA-AP.ZZ-0113(Q)
Rev 0

This new administrative procedure addresses a subset of proposed temporary modifications that may be implemented to bypass Control Room overhead annunciator input signals that have alarmed and cannot be immediately corrected by maintenance. The presence of the alarm represents a nuisance distraction to Control Room operators, may mask other contributions to the alarm window, and provides no useful to the operator.

The type of temporary modifications addressed in this procedure are limited to bypassing Control Room alarms and associated indications that have already actuated, been recognized by the Control Room operator, and thus have performed their intended design function. No changes are permitted to circuits or components with active control functions. Therefore, this procedure does not involve an Unreviewed Safety Question.

UFSAR Section

Description of Safety Evaluation

1.8.1.52.2
7.3.1.1.9.4

This UFSAR Change addresses the requirements of the Filtration, Recirculation, and Ventilation System Vent High Efficiency Particulate Air Filter Pressure Drop Alarm and Recorder instrumentation. The purpose of this UFSAR change is to correct discrepancies within the UFSAR, which does not involve any Unreviewed Safety Questions.