

## NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20655

#### COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

#### DRESDEN NUCLEAR POWER STATION, UNIT 2

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118 License No. DPK-19

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated April 24, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

#### (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 118, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

 This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard J. Barrett, Director Project Directorate 111-2

Pivision of Reactor Projects - III/IV/V ice of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 11, 1992

# FACILITY OPERATING LICENSE NO. DPR-19 DOCKET NO. 50-237

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
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	6-27

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		<u> </u>	age	
4.9	Auxiliary Electrical Systems	3/4.	9 -	. 1
4.9.A	Station Batteries	3/4.	9	1
4.9.B	(N/A)			
4.9.C	Diesel Fuel	3/4.		
4.9.D	Diesel Generator Operability	3/4.		
4.10	Refueling	3/4.		
	Refueling Interlocks	3/4.		
	Core Monitoring	3/4.		
	fuel Storage Pool Water Level	3/4.		
	Control Rod Drive and Control Rod Drive Maintenance	3/4.		
	Extended Core Maintenance	3/4.		
	Spent Fuel Cask Handling	3/4.		
4.10.G 4.11	Fuel Storage Reactivity Limit	3/4.		
4.12	High Energy Piping Integrity	3/4.	11-	1
4.16	Fire Protection Systems - Sections 3.12.A through 4.12.H -			
	Deleted per Generic Letters 86-10 and			
	82-12 (Amendment 106)			
5.0	Design Features	5		7
5.1	Site	5		1
5.2	Reactor	5		1
5.3	Reactor Vessel	5		1
5.4	Containment	5		1
5.5	Fuel Storage	5		1
5.6	Seismic Design	5		2
6.0	Administrative Controls	6	100	1
6.1	Organization, Review, Investigation and Audit	6	-	1
6.2	Procedures and Programs	6	-	13
6.3	Action to be Taken in the Event of			
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6.4	Action to be taken in the Event a			
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6.5	Plant Operating Records	6		15
6.6	Reporting Requirements			17
6.7	Environmental Qualification	6		22
6.8	Offsite Dose Calculation Manual (ODCM)	6		24
6.9	Process Control Program (PCP)	6	-	25
6.10	Major Changes to Radioactive Waste Treatment			ne.
6 22	Systems (Liquid, Gaseous, Solid)	6		25
6.11	Radiation Protection Program High Radiation Area	6		26
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- d. An evaluation of the change which shows the predicted releases of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the license application and amendment:
- e. A comparison of the predicted releases of radioactive materials in liquid and gaseous effluents and in solid waste to the actual releases for the period in which the changes were made;
- f. An estimate of the exposure to plant operating personnel as a result of the change; and
- g. Documentation of the fact that the change was reviewed and found acceptable by the On-site Review Function.
- The change shall become effective upon review and acceptance by the On-site Review Function.

#### 6.11 RADIATION PROTECTION PROGRAM

6.11.1 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained, and adhered to for all operations involving personnel radiation exposure.

#### 6.12 HIGH RADIATION AREA

- 6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP)\*. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:
  - a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

<sup>\*</sup> Health Physics personnel or personnel escorted by health physics personnel shall be exempt from the RWP issuance requirements during the performance of their assigned radiation protection duties, provided they are otherwise following plant radiation protection procedures for entry into high radiation areas.

- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them.
- A health physics qualified individual (i.e., qualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by Health Physics in the RWP

6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose greater than 1000 mrem shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Radiation Protection Supervisor. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work area and the maximum allowable stay time for individuals that area.

For individual areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose in excess of 1000 mrem\*\* that are located within large areas, such as the containment, where no enclosure exists for purposes of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted, and a flashing light shall be activated as a warning device. In lieu of the stay time specification of the RWP, continuous surveillance, direct or remote (such as use of closed circuit TV cameras), may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area.

<sup>\*\*</sup> Measurement made at 30 cm (11.8 in) from the source of radioactivity.



## UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

#### COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

#### DRESDEN NUCLEAR POWER STATION, UNIT 3

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 114 License No. DPR-25

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated April 24, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations:
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Facility Operating License No. DPR-25 is hereby amended to read as follows:

#### B. <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 114, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

 liis license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard J. Barrett, Director Project Directorate I'I-2

Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regula

Attachment: Changes to the Technical Specifications

Date of Is wance: September 11, 1992

# FACILITY OPERATING LICENSE NO. DPR-25 DOCKET NO. 50-249

Revise the Appendix A Technical Inscifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

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4.9.B	(N/A)	
4.9.C	Diesel Fuel	3/4.9-5
4.9.D	Diesel Generator Operability	3/4.9-5
4.10.	Refueling	3/4.10-1
4.10.A	Refueling Interlocks	3/4.10-1
4.10.B	Core Monitoring	3/4.10-1
4.10.C 4.10.D	Fuel Storage Pool Water Level	3/4.10-2
4.10.E	Control Rod Drive and Control Rod Drive Maintenance Extended Core Maintenance	3/4.10-3
4.10.F		3/4.10-4
4.10.G	Fuel Storage Reactivity Limit	3/4.10-5
4.11	High Energy Piping Integrity	3/4.10-8 3/4.11-1
4.12	Fire Protection Systems -	3/4.11-1
***	Sections 4.12.A through 4.12.H -	
	Deleted per Generic Letters 86-10	
	and 88-12 (Amendment 101)	
5.0	Design Features	5-1
5.1	Site	5-1
5.2	Reactor	5-1
5.3	Reactor Vessel	5-1
5.4	Containment	5-1
5.5	Fuel Storage	5-1
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6.3	Action to be Taken in the Event of	
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- d. An evaluation of the change which shows the predicted releases of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the license application and amendments;
- e. A comparison of the predicted releases of radioactive materials in liquid and gaseous effluents and in solid waste to the actual releases for the period in which the changes were made;
- f. An estimate of the exposure to plant operating personnel as a result of the change; and
- g. Documentation of the fact that the change was reviewed and found acceptable by the On-site Review Function.
- The change shall become effective upon review and acceptance by the On-site Review Function.

#### 6.11 RADIATION PROTECTION PROGRAM

6.11.1 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained, and adhered to for all operations involving personnel radiation exposure.

#### 6.12 HIGH RADIATION AREA

- 6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP)\*. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:
  - a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

<sup>\*</sup> Health Physics personnel or personnel escorted by health physics personnel shall be exempt from the RWP issuance requirements during the performance of their assigned radiation protection duties, provided they are otherwise following plant radiation protection procedures for entry into high radiation areas.

- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them.
- c. A health physics qualified individual (i.e., qualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by Health Physics in the RWP.
- 6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose greater than 1000 mrem shall be provided with 1 sked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Radiation Protection Supervisor. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area.

For individual areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose in excess of 1000 mrem\*\* that are located within large areas, such as the containment, where no enclosure exists for purposes of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted, and a flashing light shall be activated as a warning device. In lieu of the stay time specification of the RWP, continuous surveillance, direct or remote (such as use of closed circuit TV cameras), may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area.

<sup>\*\*</sup> Measurement made at 30 cm (11.8 in) from the source of radioactivity.