



**Commonwealth Edison**

Quad Cities Nuclear Power Station  
22710 206 Avenue North  
Cordova, Illinois 61242  
Telephone 309/654-2241

GCT-92-37

September 2, 1992

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2  
Monthly Performance Report  
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of August 1992.

Respectfully,

COMMONWEALTH EDISON COMPANY  
QUAD-CITIES NUCLEAR POWER STATION

Gerald C. Tietz  
Technical Superintendent

GCT/MB/dak

Enclosure

cc: A. B. Davis, Regional Administrator  
T. Taylor, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

AUGUST 1992

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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## I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Matt Benson and Debra Kelley, telephone number 309-654-2241, extensions 2995 and 2240.

## II. SUMMARY OF OPERATING EXPERIENCE

### A. Unit One

Quad Cities Unit One was in coast down for the month of August in preparation for the upcoming refuel outage.

Only one significant load reduction occurred in August. On August 15, Unit One took a manual SCRAM when the electro-hydraulic control (EHC) system loss level due to a fluid leak.

Numerous normal load reductions of less than 20% were performed per our Load Dispatcher but not reported.

### B. Unit Two

Quad Cities Unit Two reduced power to 200 MWe on August 2, 9, and 13 so that work could be performed in the Main Steam Isolation Valve (MSIV) room. Power was also reduced on August 22, to 650 MWe for a weekly turbine surveillance.

Chicago Load Dispatch requested the following load reductions for Unit Two for the month of August;

<u>Date</u>	<u>Load</u>
8-1-92	470 MWe
7-6-92	550 MWe
7-7-92	455 MWe
7-8-92	555 MWe
7-11-92	625 MWe
7-12-92	538 MWe
7-14-92	600 MWe
7-15-92	545 MWe
7-16-92	450 MWe
7-18-92	600 MWe
7-20-92	550 MWe
7-21-92	550 MWe
7-23-92	600 MWe
7-25-92	600 MWe
7-26-92	550 MWe
7-27-92	475 MWe
7-28-92	470 MWe
7-31-92	500 MWe

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS,  
AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.



UNIT 1 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q02353	5200	Repair leak on Diesel Generator fuel oil return line.	Removed old union, cleaned threads, sealed mating surfaces, reinstalled and tightened union.
Q02391	9400	Check refrigerant charge on refrigeration condenser unit control room HVAC "B" train.	Installed new filters and adjusted the thermal expansion valve.
Q02659	6601	Investigate and adjust U1/2 Diesel Generator and adjust governor for drift.	Adjusted the 1/2 Diesel Generator governor compensation screw and compensation pointer.
Q02462	6601	Investigate Unit 1 Diesel Generator.	Adjusted governor bleed screw and compensation. After adjustment there was no governor drift and the engine speed was stable.
Q02471	5727	Investigate Unit 1 Diesel Generator vent fan.	Cut back burnt wires and crimped two new lugs on one phase inside peckerhead. Taped all leads. Fan ran acceptably.
Q02494	6601	Repair fuel oil leak at suction to engine driven pump.	As Found: Leak found to be cracked thread on nipple. As Left: Repaired and replace nipple. Replaced pipe reducer.
Q02514	6601	Repair fuel oil leak at discharge of shaft driven oil pump.	Repaired and tested. Test failed. New work request written (Q02521).
Q02521	6601	Repair fuel oil leak at discharge of shaft driven oil pump.	As Found: Found cracks inside the nipple of the pipe. As Left: Replaced with new fuel oil pump. Welded new nipple. Replaced fuel oil line. Retightened one fitting.

UNIT 2 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q01126	640	Reprogram LED digital scaling on reactor level/pressure/steam flow recorder.	Reprogrammed recorder PEU #3 to read out in psig and changed scale for PEU #3.
Q01293	2301	Troubleshoot and repair air-operated steam trap bypass valve operator. AO 2-2301-31 HPCI steam trap bypass.	The pressure and exhaust ports were reversed.
Q02047	1002	Tighten or repair 2B RHR pump motor bottom motor bearing oil plug.	As Found: Found plug leaking and slightly loose. As Left: Tightened plug.
Q02365	261	Repair main steam temperature switch.	As Found: 2-261-150 switch reads open after being sprayed with water for a long period of time. As Left: Replaced switch with like for like and stopped water leak.
Q02412	5745	Investigate D RHR service water pump room cooler motor.	Replace both contactor coil leads.
Q74199	1705	Replace back plate of 2B fuel pool radiation monitor.	As Found: Back plate was broken. As Left: Installed new trip unit and set trip points. Installed sensor convertor.
Q98851	300	Repair sealtight on U2 CRD module 50-51.	Replace damaged sealtite and documented lifted leads on 700-14.



#### IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

##### UNIT 1

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
92-017	08-04-92	Control Room HVAC A/C Comp Inop.
92-018	08-13-92	Chlorine Analyzer Failed to Trip.
92-019	08-15-92	U-1 Rx Scram Due to EHC Fluid Leak at the #3 Turbine Control Valve
92-021	08-25-92	1/2 DG Inop from Governor instability.

##### UNIT 2

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
92-020	06-12-92	"R" Gate Found Open. Cancelled - investigated under a Radiation Occurrence Report.
92-020	08-11-92	RCIC Failure to Reach Speed Or Pressure.

## V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

APPENDIX C  
OPERATING DATA REPORT

DOCKET NO 50-254  
UNIT One  
DATE September 4, 1992  
COMPLETED BY Matt Benson  
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 080192 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 2511 MAX. DEPEND. CAPACITY: 769  
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL .....	<u>666.30</u>	<u>5381.90</u>	<u>141893.00</u>
6. REACTOR RESERVE SHUTDOWN HOURS .....	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. HOURS GENERATOR ON LINE .....	<u>654.30</u>	<u>5329.50</u>	<u>137560.60</u>
8. UNIT RESERVE SHUTDOWN HOURS .....	<u>0.0</u>	<u>0.0</u>	<u>909.2</u>
9. GROSS THERMAL ENERGY GENERATED (MWH).....	<u>1163726.40</u>	<u>12082257.6</u>	<u>296149280.60</u>
10. GROSS ELECTRICAL ENERGY GENERATED (MWH).....	<u>361522.0</u>	<u>3890923.0</u>	<u>95965259.0</u>
11. NET ELECTRICAL ENERGY GENERATED (MWH).....	<u>341423.0</u>	<u>3740411.0</u>	<u>90449779.0</u>
12. REACTOR SERVICE FACTOR.....	<u>89.56</u>	<u>91.92</u>	<u>79.39</u>
13. REACTOR AVAILABILITY FACTOR.....	<u>89.56</u>	<u>91.92</u>	<u>81.31</u>
14. UNIT SERVICE FACTOR .....	<u>87.94</u>	<u>91.02</u>	<u>76.97</u>
15. UNIT AVAILABILITY FACTOR .....	<u>87.94</u>	<u>91.02</u>	<u>77.48</u>
16. UNIT CAPACITY FACTOR (Using MDC) .....	<u>59.68</u>	<u>83.07</u>	<u>65.81</u>
17. UNIT CAPACITY FACTOR (Using Design MWe) .....	<u>58.19</u>	<u>80.97</u>	<u>64.14</u>
18. UNIT FORCED OUTAGE RATE .....	<u>12.06</u>	<u>8.99</u>	<u>5.85</u>

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX C  
OPERATING DATA REPORT

DOCKET NO 50-265  
UNIT Two  
DATE September 4, 1992  
COMPLETED BY Matt Benson  
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 080192 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 2511 MAX. DEPEND. CAPACITY: 769  
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL .....	<u>744.00</u>	<u>2763.55</u>	<u>136246.25</u>
6. REACTOR RESERVE SHUTDOWN HOURS .....	<u>0.0</u>	<u>0.0</u>	<u>2985.8</u>
7. HOURS GENERATOR ON LINE .....	<u>744.00</u>	<u>2692.55</u>	<u>132712.45</u>
8. UNIT RESERVE SHUTDOWN HOURS .....	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. GROSS THERMAL ENERGY GENERATED (MWH).....	<u>1671345.60</u>	<u>5703616.80</u>	<u>285787502.80</u>
10. GROSS ELECTRICAL ENERGY GENERATED (MWH).....	<u>542083.00</u>	<u>1843561.00</u>	<u>91773751.00</u>
11. NET ELECTRICAL ENERGY GENERATED (MWH).....	<u>516722.00</u>	<u>1733546.00</u>	<u>86867721.00</u>
12. REACTOR SERVICE FACTOR.....	<u>100.00</u>	<u>47.20</u>	<u>76.90</u>
13. REACTOR AVAILABILITY FACTOR.....	<u>100.00</u>	<u>47.20</u>	<u>78.59</u>
14. UNIT SERVICE FACTOR .....	<u>100.00</u>	<u>45.99</u>	<u>74.91</u>
15. UNIT AVAILABILITY FACTOR .....	<u>100.00</u>	<u>45.99</u>	<u>75.31</u>
16. UNIT CAPACITY FACTOR (Using MDC) .....	<u>90.31</u>	<u>38.50</u>	<u>63.76</u>
17. UNIT CAPACITY FACTOR (Using Design MWe) .....	<u>08.03</u>	<u>37.53</u>	<u>62.14</u>
18. UNIT FORCED OUTAGE RATE .....	<u>0.0</u>	<u>0.0</u>	<u>7.94</u>

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254  
UNIT One  
DATE September 4, 1992  
COMPLETED BY Matt Benson  
TELEPHONE (309) 654-2241

MONTH August 1992

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	<u>525</u>
2.	<u>538</u>
3.	<u>576</u>
4.	<u>556</u>
5.	<u>555</u>
6.	<u>564</u>
7.	<u>543</u>
8.	<u>553</u>
9.	<u>547</u>
10.	<u>552</u>
11.	<u>546</u>
12.	<u>515</u>
13.	<u>536</u>
14.	<u>499</u>
15.	<u>40</u>
16.	<u>-8</u>

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17.	<u>-8</u>
18.	<u>4</u>
19.	<u>429</u>
20.	<u>549</u>
21.	<u>503</u>
22.	<u>501</u>
23.	<u>520</u>
24.	<u>532</u>
25.	<u>547</u>
26.	<u>516</u>
27.	<u>515</u>
28.	<u>486</u>
29.	<u>468</u>
30.	<u>504</u>
31.	<u>511</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.



APPENDIX B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265  
UNIT Two  
DATE September 4, 1992  
COMPLETED BY Matt Benson  
TELEPHONE (309) 654-2241

MONTH August 1992

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1. <u>644</u>	17. <u>776</u>
2. <u>334</u>	18. <u>732</u>
3. <u>646</u>	19. <u>777</u>
4. <u>774</u>	20. <u>694</u>
5. <u>775</u>	21. <u>711</u>
6. <u>695</u>	22. <u>733</u>
7. <u>686</u>	23. <u>736</u>
8. <u>701</u>	24. <u>773</u>
9. <u>342</u>	25. <u>750</u>
10. <u>769</u>	26. <u>715</u>
11. <u>749</u>	27. <u>700</u>
12. <u>719</u>	28. <u>696</u>
13. <u>572</u>	29. <u>763</u>
14. <u>730</u>	30. <u>742</u>
15. <u>715</u>	31. <u>703</u>
16. <u>663</u>	

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.



APPENDIX D  
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME One

COMPLETED BY Matthew Benson

DATE September 4, 1992

REPORT MONTH August 1992

TELEPHONE

309-654-2241

NO.	DATE	TYPE T OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-31	8-15-92	F	77.7	A	2	- - - -	- - - -	- - - -	EHC System Leak

APPENDIX E  
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Two

DATE September 4, 1992

REPORT MONTH August 1992

COMPLETED BY Matthew Benson

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-42	8-01-92	S	7.7	F	5	- - - -	- -	- - - -	Load Drop Per Chicago Load Dispatcher
92-43	8-02-92	F	8.6	B	5	- - - -	- -	- - - -	MSIV Repairs
92-44	8-06-92	S	6.7	F	5	- - - -	- -	- - - -	Load Drop Per Chicago Load Dispatcher
92-45	8-06-92	S	7.1	F	5	- - - -	- -	- - - -	" " "
92-46	8-08-92	S	7.3	F	5	- - - -	- -	- - - -	" " "
92-47	8-09-92	F	17.0	B	5	- - - -	- -	- - - -	MSIV Room Work
92-48	8-11-92	S	4.4	F	5	- - - -	- -	- - - -	Load Drop Per Chicago Load Dispatcher
92-49	8-12-92	S	4.9	F	5	- - - -	- -	- - - -	" " "
92-50	8-13-92	F	8.6	B	5	- - - -	- -	- - - -	MSIV Room Maintenance
92-51	8-14-92	S	5.7	F	5	- - - -	- -	- - - -	Load Drop Per Chicago Load Dispatcher
92-52	8-15-92	S	5.8	F	5	- - - -	- -	- - - -	" " "
92-53	8-16-92	S	8.1	F	5	- - - -	- -	- - - -	" " "
92-54	8-18-92	S	5.5	F	5	- - - -	- -	- - - -	" " "
92-55	8-20-92	S	8.3	F	5	- - - -	- -	- - - -	" " "
92-56	8-21-92	S	6.4	F	5	- - - -	- -	- - - -	" " "
92-57	8-22-92	S	6.7	F	5	- - - -	- -	- - - -	Turbine Weekly Surveillance
92-58	8-23-92	S	2.9	F	5	- - - -	- -	- - - -	Load Drop Per Chicago Load Dispatcher
92-59	8-25-92	S	2.5	F	5	- - - -	- -	- - - -	" " "
92-60	8-26-92	S	5.0	F	5	- - - -	- -	- - - -	" " "
92-61	8-27-92	S	7.0	F	5	- - - -	- -	- - - -	" " "
92-62	8-28-92	S	6.0	F	5	- - - -	- -	- - - -	" " "
92-63	8-31-92	S	6.9	F	5	- - - -	- -	- - - -	" " "

## VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

### A. Main Steam Relief Valve Operations

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

Unit: One

Date: 08/18/92

Valves Actuated

No. & Type of Actuation

1-203-3A  
1-203-3B  
1-203-3C  
1-203-3D  
1-203-3E

All manual actuations.

Plant Conditions: Rx. pressure: 922 psig.

Description of Events: Routine surveillance and Post Maintenance following adjustment of 3B pilot. (QCOS 203-3)

### B. Control Rod Drive Scram Timing Data for Units One and Two

There was no Control Rod Drive scram timing data for Units One and Two for the reporting period.

## VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD CITIES REFUELING  
INFORMATION REQUEST

1. Unit: Q1 Reload: 11 Cycle: 12
2. Scheduled date for next refueling shutdown: 9-20-92
3. Scheduled date for restart following refueling: 12-12-92
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:
  1. Modification to HPCI turbine exhaust steam line.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
  1. 06/30/92
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

NONE AT PRESENT TIME.
7. The number of fuel assemblies.
  - a. Number of assemblies in core: 724
  - b. Number of assemblies in spent fuel pool: 1405
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
  - a. Licensed storage capacity for spent fuel: 3657
  - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009



QUAD CITIES REFUELING  
INFORMATION REQUEST

1. Unit: Q2 Reload: 11 Cycle: 12
2. Scheduled date for next refueling shutdown: 03/06/93
3. Scheduled date for restart following refueling: 06/05/93
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:  
NOT AS YET DETERMINED.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:  
NOT AS YET DETERMINED.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:  
NONE AT PRESENT TIME.
7. The number of fuel assemblies.
  - a. Number of assemblies in core: 724
  - b. Number of assemblies in spent fuel pool: 2439
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
  - a. Licensed storage capacity for spent fuel: 3897
  - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009



## VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	- Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	- American National Standards Institute
APRM	- Average Power Range Monitor
ATWS	- Anticipated Transient Without Scram
BWR	- Boiling Water Reactor
CRD	- Control Rod Drive
EHC	- Electro-Hydraulic Control System
EOF	- Emergency Operations Facility
GSEP	- Generating Stations Emergency Plan
HEPA	- High-Efficiency Particulate Filter
HPCI	- High Pressure Coolant Injection System
HRSS	- High Radiation Sampling System
IPCLRT	- Integrated Primary Containment Leak Rate Test
IRM	- Intermediate Range Monitor
ISI	- Inservice Inspection
LER	- Licensee Event Report
LLRT	- Local Leak Rate Test
LPCI	- Low Pressure Coolant Injection Mode of RHRs
LPRM	- Local Power Range Monitor
MAPLHGR	- Maximum Average Planar Linear Heat Generation Rate
MCPR	- Minimum Critical Power Ratio
MFLCPR	- Maximum Fraction Limiting Critical Power Ratio
MPC	- Maximum Permissible Concentration
MSIV	- Main Steam Isolation Valve
NIOSH	- National Institute for Occupational Safety and Health
PCI	- Primary Containment Isolation
PCIOMR	- Preconditioning Interim Operating Management Recommendations
RBCCW	- Reactor Building Closed Cooling Water System
RBM	- Rod Block Monitor
RCIC	- Reactor Core Isolation Cooling System
RHRS	- Residual Heat Removal System
RFS	- Reactor Protection System
RWM	- Rod Worth Minimizer
SBGTS	- Standby Gas Treatment System
SBLC	- Standby Liquid Control
SDC	- Shutdown Cooling Mode of RHRS
SDV	- Scram Discharge Volume
SRM	- Source Range Monitor
TBCCW	- Turbine Building Closed Cooling Water System
TIP	- Traversing Incore Probe
TSC	- Technical Support Center