

NRC Form 308
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/86

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Grand Gulf Nuclear Station - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 1 6	PAGE (3) 1 OF 0 2
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TITLE (4)
Reactor Scram

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)		
1	2	2	4	8	4	8	4	8	N/A			0 5 0 0 0 0		
1	2	2	4	8	4	8	4	8				0 5 0 0 0 0		

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 20.2017 (Check one or more of the following) (11)

OPERATING MODE (9)	20.402(b)	20.406(a)	<input checked="" type="checkbox"/>	80.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	20.406(a)(1)(i)	80.36(a)(1)	<input type="checkbox"/>	80.73(a)(2)(v)	73.71(c)
0 1 0 1 2	20.406(a)(1)(ii)	80.36(a)(2)	<input type="checkbox"/>	80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 305A)
	20.406(a)(1)(iii)	80.73(a)(2)(i)	<input type="checkbox"/>	80.73(a)(2)(vii)(A)	
	20.406(a)(1)(iv)	80.73(a)(2)(ii)	<input type="checkbox"/>	80.73(a)(2)(vii)(B)	
	20.406(a)(1)(v)	80.73(a)(2)(iii)	<input type="checkbox"/>	80.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Angela H. Horton/License Engineer	TELEPHONE NUMBER AREA CODE: 6 0 1 4 3 7 - 2 1 4 9
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

The reactor was being manually shutdown when the operator noticed a decrease in reactor water level. To increase the level, he increased the feedwater flow by taking manual control of the startup level controller. This eliminated the decrease but the injection of the cool feedwater into the vessel increased the reactor power level and caused a spike of IRM's C and F which resulted in a reactor scram.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 9/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Grand Gulf Nuclear Station - Unit 1	0 5 0 0 0 4 1 6	8 4	- 0 5 8	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description of Reportable Occurrence

On December 24, 1984, at 1232 hours, the Reactor scrambled on an IRM High-High flux trip.

Initial Conditions

The plant was being shutdown so that a possible tube leak in the intermediate pressure condenser could be repaired. The mode switch was in startup at less than 2 percent power.

Status of Redundant or Backup Systems

Not Applicable

Nature of Occurrence

The mode switch had been placed in startup from the run position and, according to procedure the turbine control valves and the bypass control valves were closed. Reactor power had decreased to below 2 percent, and as had been experienced in the past, the startup level control valve became sluggish. The operator noticed a decrease in the reactor water level so he increased the feedwater flow by taking manual control of the startup level controller. This eliminated the decrease but the injection of the cool feedwater into the vessel increased the reactor power level and caused a spike of IRM's "C" and "F".

Immediate Corrective Actions Taken

The scram Off Normal Event Procedure (ONEP) was carried out, parameters established, and the Main Steam Isolation Valves (MSIV's) closed to reduce the cooldown to less than 100°F per hour.

Apparent Cause

The operator failed to realize the magnitude of the affect that a change in feedwater flow could have on reactor power when the turbine control valves and the bypass control valves are closed.

Supplemental Corrective Action

The startup and shutdown operating instructions will be changed to include a warning to operators of the expected changes in reactor power level when changing feedwater flow, recirculation flow, or rod position with the turbine and bypass control valves closed.

Safety Assessment

The reactor protection system performed as designed.



MISSISSIPPI POWER & LIGHT COMPANY

Helping Build Mississippi

P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

January 23, 1985

NUCLEAR LICENSING & SAFETY DEPARTMENT

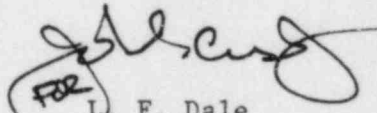
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U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

SUBJECT: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-29
File: 0260/L-835.0
Reactor Scram
LER 84-058-0
AECM-85/0025

Attached is Licensee Event Report (LER) 84-058-0 which is a final report.

Yours truly,


L. F. Dale
Director

EBS/SHH:rw
Attachment

cc: Mr. J. B. Richard (w/a)
Mr. R. B. McGehee (w/a)
Mr. N. S. Reynolds (w/a)
Mr. G. B. Taylor (w/o)

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