

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Waterford 3 Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 2	PAGE (3) 1 OF 0 3
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TITLE (4)
Fire Door Technical Specification Surveillances

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
1	2	26	8	4	8	4	0	0	NA		0 5 0 0 0
									NA		0 5 0 0 0

OPERATING MODE (8) 6	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)											
POWER LEVEL (10) 0 0 0	20.402(b)			20.405(e)			50.73(a)(2)(iv)			73.71(b)		
	20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)		
	20.405(a)(1)(ii)			<input checked="" type="checkbox"/> 50.36(c)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
	20.405(a)(1)(iii)			<input checked="" type="checkbox"/> 50.73(a)(2)(i)			50.73(a)(2)(viii)(A)					
	20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)					
	20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)					

LICENSEE CONTACT FOR THIS LER (12)									
NAME O.D. Hayes, Operations Superintendent								TELEPHONE NUMBER 5 0 4 4 6 4 - 3 1 1 8	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)								EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

On December 26, 1984 Waterford 3 Steam Electric Station was in mode 6 for the previous seven (7) days when Operations personnel discovered the fire door surveillance, as defined in Technical Specification 4.7.11.2c,d, for eleven (11) fire doors had not been performed since the issuance of a Low Power License on December 18, 1984. Upon further investigation it was discovered that four (4) additional fire doors were not verified to be operable as required by the above referenced Technical Specification. Since Operations personnel were unaware that the operability requirements for the fifteen (15) fire doors were not met, a fire watch was not established within the proper time interval as required by Technical Specification 3.7.11a.

An immediate notification, via the Emergency Notification System, was made to the Commission pursuant to 10CFR50.36(c)(2).

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

On December 26, 1984 Waterford 3 Steam Electric Station was in mode 6 for the previous seven (7) days when Operations personnel discovered the fire door surveillance, as defined in Technical Specification 4.7.11.2c,d, for eleven (11) fire doors had not been performed since the issuance of a Low Power License on December 18, 1984. An immediate review, by shift personnel, of the controlling procedure, OP-903-001, Technical Specification Surveillance Logs, revealed that the fire doors in question were listed on page 3 of Attachment 10.12, which was inadvertently omitted from OP-903-001 during a recent change made to the procedure on December 6, 1984. In accordance with UNT-1-003, Plant Operating Manual Procedure Development, Review and Approval: Change and Revision: and Deletion, and Technical Specification 6.8.3, shift personnel initiated a temporary change to OP-903-001 in order to incorporate the subject page.

As part of the corrective action, a Senior Reactor Operator and a Shift Technical Advisor, in conjunction with a Fire Protection Engineer, were asked to perform a complete technical review of OP-903-001 in an effort to ensure the above incident did not occur again. Upon review of OP-903-001 it was discovered that four (4) additional fire doors were also omitted from Attachment 10.12. Operations personnel, in accordance with procedure UNT-1-003 and Technical Specifications, made a temporary change to OP-903-001 in order to include the additional doors.

Since Operations personnel were unaware that the operability requirements for the fifteen (15) fire doors were not met, a fire watch was not established within the proper time interval as required by Technical Specification 3.7.11a.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

SAFETY CONSEQUENCE AND IMPLICATIONS:

Since the required surveillance was not performed from December 18, 1984 to December 26, 1984 and January 10, 1985 respectively, Waterford 3 personnel could not ensure the fire doors in question were closed as required by Technical Specification 4.7.11.2c,d. However, since each subject fire door has an automatic closing device, and Operations personnel, immediately upon discovery of the noncompliance, performed the required surveillance, and proved each fire door was closed, it is felt the fire doors were in fact operable throughout the above time interval.

CORRECTIVE ACTIONS:

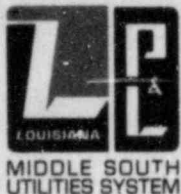
On January 11, 1985 a Senior Reactor Operator and a Shift Technical Advisor satisfactorily completed the technical review of OP-903-001. All supporting documentation, along with a complete revision of OP-903-001, has been reviewed by the Plant Operations Review Committee per Technical Specification 6.5.16.

SIMILAR EVENTS:

None

PLANT CONTACT:

O. D. Hayes, Operations Superintendent, 504/464-3118



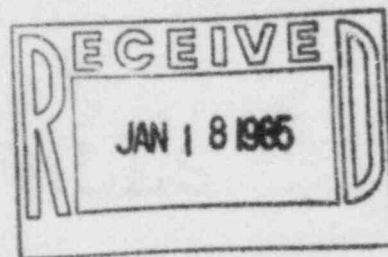
LOUISIANA
POWER & LIGHT

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NEW ORLEANS, LOUISIANA 70174-8008 • (504) 366-2345

January 17, 1985

W3P85-0123
3-A1.01.04
C14.03
A4.05

Mr. Robert D. Martin
Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011



Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-26
Reporting of Licensee Event Report

Dear Mr. Martin:

Attached is Licensee Event Report Number LER-84-002-00 for the Waterford 3 Steam Electric Station. This Licensee Event Report is submitted per 10CFR50.36(c)(2) and 10CFR50.73(a)(2)(i).

Very truly yours,

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: (NRC) Document Control Desk, G.W. Knighton, D.M. Crutchfield,
NRC Resident Inspectors Office, INPO Records Center (D.L. Gillispie),
E.L. Blake, W.M. Stevenson

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W3P85-0123

bcc: R.S. Leddick, R.P. Barkhurst, F.J. Drummond, K.W. Cook, D.E. Dobson,
T.F. Gerrets, S.A. Alleman, L.F. Storz, R.F. Burski, O.D. Hayes.
R.B. Willis, W. Perks, D. Packer, R.A. Legere, R.M. Nelson,
K.R. Iyengar, R.A. Savoie, R.J. Murillo, W.A. Cross, B.G. Morrison,
P.R. Pellette, Project Files, Administrative Support (3), Licensing
Library, Onsite Licensing Files, F.J. Englebracht