

September 9, 1992

Director of Nuclear Reactor Regulation United States Nuclear Regulatory Commission Mail Station P1-137 Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for August 1992.

Very truly yours.

G. J. Diederich to/Station Manager

LaSalle County Station

GJD/MJC/djf

Enclosure

xc: A. B. Davis, NRC, Region III

D. E. Hills, NRC Resident Inspector LaSalle

J. L. Roman, IL Dept. of Nuclear Safety

B. Siegel, NRR Project Manager

D. P. Galle, CECo D. L. Farrar, CECo INPO Records Center

D. R. Eggett, NED P. D. Doverspike, GE Resident

T. J. Kovach, Manager of Nuclear Licensing T. A. Rieck, Nuclear Fuel Services Manager

J. E. Lockwood, Regulatory Assurance Supervisor

W. P. Pietryga, QA/NS Off Site Review

Station File

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UNIT 1

MONTHLY PERFORMANCE REPORT

AUGUST 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373 LICENSE NO. NPF-11

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#### INTRODUCTION (Unit 1)

The LaSalle County Nuclear Prwer Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lurdy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial critically was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

## II. MONTHLY REPORT

## A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
1	0000	Reactor critical, Generator on-line at 780 Mwe due to system load (Unit in coastdown).
	1100	Increased power level to 1015 Mwe.
	2000	Reduced power level to 780 Mwe due to system load.
2	1000	Increased power level to 1010 Mwe.
3	0200	Reduced power level to 950 Mwe due to system load.
	0600	Increased power level to 1010 Mwe.
4	0000	Reduced power level to 740 Mwe due to system load.
	1000	Increased power level to 1005 Mwe.
5	0130	Reduced power level to 740 Mwe due to system load.
	0900	Increased power level to 1005 Mwe.
6	0200	Reduced power level to 740 Mwe due to system load.
	0800	Increased power level to 1000 Mwe.
8	0000	Reduced power level to 835 Mwe .a to system load.
	0900	Increased power level to 920 Mwe.
	2330	Reduced power level to 850 Mwe due to system load.
9	0930	Increased power level to 980 Mwe.
	2330	Reduced power level to 840 Mwe due to system load.
11	1000	Increased power level to 980 Mwe.
	1930	Reduced power level to 900 Mwe due +- stem load.
	2300	Reduced power level to 840 Mwe due to system load.
12	0730	Increased power level to 980 Mwe.
	2330	Reduced power level to 850 Mwe due to system load.
13	0800	Increased power level to 975 Mwe.
14	2300	Reduced power level to 850 Mwe due to system load.
15	0700	Increased power level to 975 Mwe.

# II. MONTHLY REPORT (CONTINUED)

# A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
	1900	Reduced power level to 880 Mwe due to system load.
16	0000	Reduced power level to 850 Mwe due to system load.
	0400	Reduced power level to 750 Mwe due to system load.
	1000	Increased power level to 975 Mwe.
	2330	Reduced power level to 850 Mwe due to system load.
17	0700	Increased power level to 960 Mwe.
18	0100	Reduced power level to 350 Mwe due to system load.
	0900	Increased power level to 950 Mwe.
19	2300	Reduced power level to 850 Mwe due to system load.
20	0900	Increased power level to 950 Mwe.
	1930	Reduced power level to 850 Mwe due to system load.
21	0800	Increased power level to 945 Mwe.
	2330	Reduced power level to 850 Mwe due to system load.
23	0200	Reduced power level to 700 Mwe due to system load.
	1400	Increased power level to 940 Mwe.
24	0030	Reduced power level to 800 Mwe due to system load.
	1000	Increased power level to 930 Mwe.
26	0230	Reduced power level to 740 Mwe due to system load.
	1000	Increased power level to 930 Mwe.
	2330	Reduced power level to 740 Mwe due to system load.
27	1000	Increased power level to 930 Mwe.
28	0100	Reduced power level to 700 Mwe due to system load.
	0800	Increased power level to 930 Mwe.
29	0200	Reduced power level to 750 Mwe due to system load.
	1000	Increased power level to 930 Mwe.

# . II. MONTHLY REPORT (CONTINUED)

# A. SUMMARY OF GPERATING EXPERIENCE (Unit 1)

Time	Event
0030	Reduced power level to 740 Mwe due to system load.
1200	Increased power level to 930 Mwe.
0100	Reduced power level to 800 Mwe due to system load.
0700	Increased power level to 930 Mwe.
2300	Reduced power level to 740 Mwe due to system load.
2400	Reactor critical, Generator on-line at 740 Mwe, continuing coastdown.
	0030 1200 0100 0700 2300

- B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

  Revised reference from General Electric performing core operating limits analysis to Commonwealth Edison and General Electric.
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).

  (See Table 1)
- D. LICENSEE EVENT REPORTS (Unit 1) (None.)
- E. DATA TABULATIONS (Unit 1)
  - 1. Operating Data Report (See Table 2)
  - 2. Average Daily Unit Power Level (See Table 3)
  - 3. Unit Shutdowns and Significant Power Reductions (See Table 4)

### C. TABLE 1 (Unit 1)

# MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L16302	Division II Post-LOCA Containment Monitoring Reagent Gas Bottle ICM12T	Bottle regulator leaking.	None.	Replaced regulator.
L16591	Main Control Room Ammonia Detector OYY-VC125B	Alarm lamp.	Spurious alarms.	Replaced bulb.
L17068	Local Power Range Monitor 24-25B	LPRM card.	Inaccurate indication.	Replaced diode on LPRM card.
L17128	Standby Gas Treatment Wide Range Noble Gas Monitor Recorder OD18-R519	Chart drive motor.	Chart paper not advancing properly.	Replaced chart drive motor.

(No SOR Failures this month.)

#### TABLE 2 E.1 OPERATING DATA REPORT

DOCKET NO. 050-373

UNIT LASALLE ONE

DATE September 10, 1992

COMPLETED BY M.J.CIALKOWSKI

TELEPHONE (815) 357-6761

#### OPERATING STATUS

1. REPORTING PERIOD: August 1992
GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323

MAX DEPENDABLE CAPACITY (MWe-Net): 1,036

DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASON FOR RESTRICTION (IF ANY):

			YEAR TO DATE	CUMULATIVE
5. R	EACTOR CRITICAL TIME (HOURS)	744.0		53,150.0
6 . R	EACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. 0	ENERATOR ON-LINE TIME (KOURS)	744.0	5,760.8	52,139.1
8. U	NIT RESERV SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. T	HERMAL ENERGY GENERATED (MWHt)	2,061,488	17,992,356	153,149,335
10. E	LECTRICAL ENERGY GENERATED (MWHe-Gross)	683,526	6,6 5,619	51,155,391
11. E	LECTRIC.1 ENERGY GENERATED (MWHe-Net)	656,988	5,865,824	49,060,969
12. R	DACTOR SERVICE FACTOR (%)	100.0	98.9	€9.9
13. R	EACTOR AVAILABILITY FACTOR (%)	100.0	98.9	72.1
14. U	NIT SERVICE FACTOR (%)	100.0	98.4	68.6
15. U	NIT AVAILIBILITY FACTOR (%)	100.0	98.4	68.6
16. U	NIT -\PACTAY FACTOR (USING MDC) (%)	85.2	96.7	52.3
17. U	NIT CAPA'.ITY FACTOR (USING DESIGN MWe)	81,9	92.9	59.9
18. 0	NIT FOSCED OUTAGE FACTOR (%)	0.0	1.6	7.0

<sup>19.</sup> SHUTDOWNS SCHEDULED OVER THE MEXT 6 MONTHS (TYPL, DATE, AND DURATION OF EACH):
Refuel Outage, 09/26/92, 13 Weeks

<sup>20.</sup> IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

DOCKET NO. 050-373

UNIT LASALLE ONE

DATE September 10, 1992

COMPLETED BY M.J. CIALKOWSKI

TELEPHONE (815)-357-6761

#### REPORT PERIOD: August 1992

DAY	POWER	DAY	POWER
1	852	17	905
2	900	1.8	903
3	960	19	909
4	930	20	868
5	888	21	889
6	900	22	825
7	949	23	803
В	878	24	877
	907	25	901
10	937	26	846
-11	892	27	837
12	906	2.8	107
13	914	29	833
14	915	30	825
15	897	31	853
16	874		

#### TABLE 4

# E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20% (Unit 1)

YEARLY SEQUENTIAL DATE

WUMBER

TYPE

DATE F: FORCED
(YYMMDD) S: SCHEDULED

DURATION (HOURS)

REASON

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POMER CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)

(None.)

#### SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system loading and maintenance activities.

### F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

- Safety/Relief valve operations (None.)
- ECCS System Outages (See Table 5)
- Changes to the Off-Site Dose Calculation Manual (None.)
- Major changes to Radioactive Waste "reatment Systems. (None.)
- Indications of Failed Fuel Elements. (None.)

# F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

OUTAGE NO.	EQUIPMENT	PURPOSE
(U-O)		
0617	ODG08CA	Repair the compressor discharge check valve.
(U-1)		
0626	1E22-S001	Varmeter replacement and megger cooling water pump.
0629	1E22-S001	HACR relay inspection.
2631	1E22-S001	Change soakback oil filter.
0632	1E22-S001	Install cylinder test valves.
0642 0643 0644	1E12-F047B	Refurbish limitorque.
0645 0652	1E12-C002B	Inspection and oil change.
0646	1E12-C002C	Inspection and oil change.
0647	1DG049A	Check Valve inspection.
0651	1DG049B	Check valve inspection.
0689	1E12-C300A 1E12-C300B	Megger motors.
0690	1E21-C001	Megger motor and change oil.
0691	1E12-C002A	Megger motor and change oil.
0713	1E12-C001	Relay calibrations.
0714	1E12-C002A	Relay calibrations.
0741	1512-F085A	Admin control during alarm testing.
0755	1E51-C003 1E51-F025 1E51-F026	EQ test on water leg pump, replace solenoid coils on drain valves.
0783	1E22-C003	Water leg pump inspection.
0784	1E22-C003	Coupling lubrication.

UNIT 2

MONTHLY PERFORMANCE REPORT

AUGUST 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374 LICENSE NO. NPF-18

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- F. UNIQUE REPORTING REQUIREMENTS
  - 1. Safety/Relief Valve Operations
  - 2. ECCS System Outages
  - 3. Off-Site Dose Calculation Manual Changes
  - 4. Major Changes to Radioactive Waste Treatment System
  - 5. Indications of Failed Fuel Elements

#### INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

# . II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

Day	Time	Event
1.	0000	Reactor critical, Generator on-line at 1080 Mwe (Power level held due to high Main Turbine vibrations).
	0200	Reduced power level to 840 Mwe due to system load.
	1200	Increased power level to 1050 Mwe.
	2230	Reduced power level to 850 Mwe due to system load.
2	1030	Increased power level to 1070 Mwe.
	1600	Reduced power level to 1000 Mwe due to system load.
3	0200	Reduced power level to 950 Mwe due to system load.
	1000	Increased power level to 1050 Mwe.
4	0100	Reduced power level to 850 Mwe due to system load.
	1100	Increased power level to 1100 Mwe.
	1700	Reduced power level to 1000 Mwe due to APRM high alarms.
	2300	Reduced power level to 800 Mwe to perform scram timing and rod set.
5	1100	Increased power level to 1070 Mwe.
	2330	Reduced power level to 830 Mwe due to system load.
6	1100	Increased power level to 1120 Mwe.
8	0030	Reduced power level to 890 Mwe due to system load.
	1600	Increased power level to 1115 Mwe.
9	0130	Reduced power level to 940 Mwe due to system load.
	1400	Increased power level to 1115 Mwe.
11	0030	Reduced power level to 850 Mwe due to system load.
	1300	Increased power level to 1115 Mwe.
12	0230	Reduced power level to 850 Mwe due to system load.
	1000	Increased power level to 1010 Mwe.
13	0200	Reduced power level to 800 Mwe due to system load.
	1100	Increased power level to 1120 Mwe.

# II. MONTHLY REPORT

# A. SUMMARY OF OPERATING EXPERIENCE (Unit 2) (CONTINUED)

Day	Time	Event
14	0130	Reduced power level to 840 Mwe due to system load.
	1000	Increased power level to 1110 Mwe.
	1830	Reduced power level to 940 Mwe due to system load.
	2300	Reduced power level to 850 Mwe due to system load.
15	0830	Increased power level to 1050 Mwe.
16	0030	Reduced power level to 850 Mwe due to system load.
	1200	Increased power level to 1075 Mwe.
17	0400	Reduced power level to 900 Mwe due to system load.
	1000	Increased power level to 1100 Mwe.
18	0130	Reduced power level to 1010 Mwe due to system load.
	0600	Increased power level to 1085 Mwe.
19	0300	Reduced power level to 840 Mwe due to system load.
	1000	Increased power level to 1100 Mwe.
27	0310	Reactor scram due to main turbine trip. The turbine trip was caused by a thrust bearing wear detector.
31	2400	Reactor subcritical, Generator off-line, Forced outage in progress.

- B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

  Revised reference from General Electric performing core operating limits analysis to Commonwealth Edison and General Electric.
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports). (See Table 1)
- LICENSEE EVENT REPORTS (Unit 2) D. LER Number Description Date 08/10/92 Failure of the Reactor Core Isolation 92-010-00 Cooling steam line outboard isolation valve 2E51-F008 during performance of the steam line high flow calibration. 92-011-00 08/09/92 Spurious automatic start of the Control Room ventilation emergency makeup train due to a high radiation spike. 92-012-00 08/27/92 Main Turbine trip due to thrust bearing wear detector which resulted in a Reactor scram.

#### E. DATA TABULATIONS (Unit 2)

- Operating Data Report. (See Table 2)
- Average Daily Unit Power Level. (See Table 3)
- Unit Shutdowns and Significant Power Reductions. (See Table 4)

### C. TABLE 1 (Unit 2)

#### MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT

WCPK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L15544	High Pressure Core Spray Diesel Generator "B" Air Receiver	Air receiver pressure relief valve 2E22-F3698 leaking.	None.	Replaced valve.
L17093	Reactor Core Isolation Cooling Turbine Isolation Valve 2E51-F008	Valve motor burned up due to the torque switch contacts and the valve being closed prior to initiation of Isolation signal.	Valve breaker tripped on magnetics during surveillance testing.	Replaced valve motor.

(No SOR Failures this month.)

#### TABLE 2 E.1 OPERATING DATA REPORT

DOCKET NO. 050-373

UNIT LASALLE TWO

DATE September 10, 1992

COMPLETED BY M.J.CIALKOWSKI TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD: August 1992
GROSS HOURS IN REPORTING PERIOD: 744

- 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323

  MAX DEPENDABLE CAPACITY (MWe-Net): 1,036

  DESIGN ELECTRICAL RATING (MWe-Net): 1,078
- 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
- 4. REASON FOR RESTRICTION (IF ANY):

		THIS MONTH	YEAR TO DATE	CUMULATIVE
5.	REACTOR CRITICAL TIME (HOURS)	627.2	3,388.4	48,324.3
6.	REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7.	GENERATOR ON-LINE TIME (HOURS)	627.2	3,226.6	47,431.1
8.	UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
0,	THERMAL ENERGY GENERATED (MWHt)	1,941,742	9,676,399	141,918,798
10.	ELECTRICAL ENERGY GENERATED (MWHe-Gross)	652,068	3,274,700	47,164,450
11.	ELECTRICAL ENERGY GENERATED (MWHe-Net)	627,160	3,129,740	45,261,244
12.	REACTOR SERVICE FACTOR (%)	84.3	57.9	70.0
13.	REACTOR AVAILABILITY FACTOR (%)	84.3	57.9	72.5
14.	UNIT SERVICE FACTOR (%)	84.3	55.1	68.7
15.	UNIT AVAILIBILITY FACTOR (%)	84.3	55.1	68.7
16,	UNIT CAPACITY FACTOR (USDIG MDC) (%)	81,4	51.6	63.3
17.	UNIT CAPACITY FACTOR (USING DESIGN MWO)	78,2	49.6	60.9
18.	UNIT FORCED OUTAGE FACTOR (%)	15.7	6.8	12.7

<sup>19.</sup> SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AN' DURATION OF EACH):

<sup>20.</sup> IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 09/10/92

DOCKET NO. 050-373

UNIT LASALLE TWO

DATE September 10, 1992

COMPLETED BY M.J. CIALKOWSKI

TELEPHONE (815)-357-6761

REPORT PERIOD: August 1992

DAY	POWER	DAY	POWER	
1	916	17	1,018	
2	937	18	1,026	
3.	979	19	991	
4	935	20	1,054	
	915	21	1,061	
6	993	22	1,051	
7	1.061	23	1,043	
8	991	24	1,051	
9	1,010	25	1,059	
10	1,054	26	1,053	
11	982	27	117	
12	1,004	28	-12	
13	1,009	29	-12	
14	960	30	-12	
15	958	31	-12	
16	953			

#### TABLE 4

# E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20% (UNIT 2)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR RELUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
07	920827	F	116.8	λ	3	Main turbine trip due to turbine thrust bearing wear detector (LER# 92-912-00/DVR# 1-2-92-067).

#### SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system loading and maintenance activities. The unit experienced a forced outage on 08/27/92 due to a main turbine thrust bearing wear detector. The unit is expected to return to service on 09/10/92.

### F. UNIQUE REPORTING REQUIREMENTS (Unit 2)

Safety/Relief Valve Operations

DATE	VALVES	NO & TYPE	PLANT	DESCRIPTION
	ACTUATED	ACTUATIONS	CONDITION	OF EVENY
08/27/92	2B21-F013U	Automatic	3	Scram(LER# 92-012-00)
08/27/92	2B21-F013A	Manual	3	Scram(LER# 92-012-00)
08/27/92	2B21-F013B	Manual	3	Scram(LER# 92-012-00)

- ECCS System Outages (See Table 5.)
- Changes to the Off-Site Dose Calculation Manual. (None.)
- Major changes to Radioactive Waste Treatment Systems. (None.)
- Indications of Failed Fuel Elements. (None.)

## (Unit 2) Table 5

# F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

OUTAGE NO.	EQUIPMENT	PURPOSE
1928	2E22-F369B	Reset relief valve setting.
1929	2022-5001	HACR relay inspection
1931	3E22-S001	Change soakback oil filters.
1936	2E51-F331	Lubricated turbine trip and throttle valve, repair flange on drain pot.
1938 1939 1995 1999 2000	2E51-F008	Breaker inspection and lalve repair.
1950	2E12-C002A	Lubrication.
1967	2E12-C002B	Lubrication.
1974	2DG08CB	Repair check valve.
1979	2E51-F091	Admin control.
1980	2E12-F024A	Admin control during full flow test.
2025	2251-F066	Check valve repair.
2028	2E12-F050B	Valve repack.
2034	2DG08CB	Repair compressor.

ZCADTS/7