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UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
476 ALLENDALE ROAD
KING OF PRUSSIA, PENNSYLVANIA 19606

JUN 04 1991

Docket Nos. 50-336
File Number RI-91-A-0064

Northeast Nuclear Energy Company
ATTN: Mr. E. J. Mroczka
Senior Vice President - Nuclear
Engineering and Operations Group
P.O. Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Mroczka:

The U.S. Nuclear Regulatory Commission recently received information concerning activities at the Millstone Nuclear Power Facility, Unit 2. The details are enclosed for your review and followup.

We request that the results of your review and disposition of these matters be submitted to Region I within 30 days of the date of receipt of this letter. We request that your response contain no personal privacy, proprietary, or safeguards information so it can be released to the public and placed in the NRC Public Document Room. If necessary, such information shall be contained in a separate attachment which will be withheld from public disclosure. The affidavit required by 10 CFR 2.790(b) must accompany your response if proprietary information is included. Please refer to file number RI-91-A-0064 when providing your response.

The enclosure to this letter should be controlled and distribution limited to personnel with a "need to know" until your investigation of the concern has been completed and reviewed by NRC Region I. The enclosure to this letter is considered Exempt from Public Disclosure in accordance with Title 10, Code of Federal Regulations, Part 2.790(a). However, a copy of this letter excluding the enclosure will be placed in the NRC Public Document Room.

The response requested by this letter and the accompanying enclosure are not subject to the clearance procedures of the Office of Management and Budget as required by the Paperwork Reduction Act of 1980, PL 96-511.

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PDR FDIA
GUILD91-162 PDR
APP

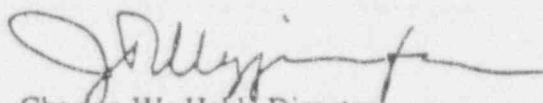
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Northeast Nuclear Energy Company

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Your cooperation with us is appreciated. We will gladly discuss any questions you have concerning this information.

Sincerely,



Charles W. Hehl, Director
Division of Reactor Projects

Enclosure: ~~(10 CFR 2.790(a) Information)~~

cc w/o encl:

Public Document Room (PDR)
Local Public Document Room (LPDR)
State of Connecticut

bcc w/encl:

W. Raymond, SRI, Millstone
Allegation File, RI-91-A-0064
J. Stewart

ENCLOSURE

Issue 1; On March 28, 1991 a QC inspector was called by Generation Construction to perform a final weld inspection of a prefabricated structure for the new steam jet air ejector monitor. The final inspection was part of procedure GWS-006. After the inspection, the QC inspector questioned if a "fit-up" inspection had been completed, as required by procedure. The fabrication and assembly of the structure were Non-QA, thus as required by GWS-006, the job supervisor performs the "fit-up" inspection. Based on indication that no "fit-up" inspection had been completed, the QC inspector initiated NCR 2-91-035. On April 1, Generation Construction completed the authorized work order and documented that a "fit-up" inspection had been completed; however, it had not been completed. This is the way non-QA inspection and documentation are normally accomplished.

Please discuss the validity of the above assertions. Please discuss the need for a "fit-up" inspection and the circumstances surrounding its completion, if required. Please discuss any corrective actions that you have taken or may take in response to any identified problems in procedural compliance or quality controls. Please discuss if the assertions are indicative of any generic problems.

Issue 2; On April 8, 1991, a NNECO technician (*) and a contractor were assigned fire protection surveillance, IC-2439A. The NNECO technician had never been trained on the surveillance procedure. On April 3, 1991, another technician was performing monthly function surveillance SP-2410A for the acoustic valve monitoring system associated with the pressurizer relief valves. This was the first time the technician had performed the surveillance since the last major revision and again, there had been no training on the conduct of the procedures with the incorporated revisions. In both cases, the technicians experienced difficulty in completing the procedures and numerous questions were raised.

Please discuss the validity of the above assertions. Please discuss employee training prior to performance of complex surveillances especially in cases involving first time assignment and performance subsequent to major revisions. Please discuss any corrective actions that you have taken or will take in these cases, to ensure that the surveillances are completed competently and safely.

(*) The identity may be obtained from the Senior Resident Inspector.

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Issue 3; During performance of modification package PDCE M2-90-032, drawing errors were noted for radiation monitor RM-9095. The errors involve the omission of a terminal board between the 120 VAC power supply and the radiation monitor solenoid valve on drawings 25203-39092 sheet 14a, 25203-31118 sheet 1, and 25203-32026 sheet 53. In addition to not identifying the drawing errors, the modification package did not identify the need for an equipment tagout.

Please discuss the validity of the above assertions. Please discuss actions that have been taken to correct any deficiencies with the modification package and with work control deficiencies, in general.

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