

JCP&L GPU

50-219
R
Jersey Central Power & Light Company
Madison Avenue at Punch Bowl Road
Morristown, New Jersey 07960
(201) 455-8200
4361

March 15, 1979

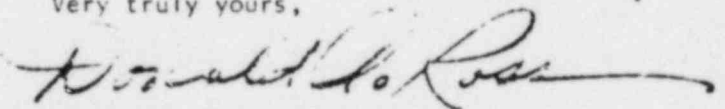
Mr. Boyce H. Grier, Director
Office of Inspection and Enforcement
Region I
United States Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

Subject: Oyster Creek Station
Docket No. 50-219
Monthly Operating Data

Enclosed are ten copies of the monthly operating data (Gray Book Information) regarding our Oyster Creek Nuclear Generating Station.

Very truly yours,



Donald A. Ross, Manager
Generating Stations-Nuclear

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Enclosures

cc: Mr. William G. McDonald, Director (2 copies)
Office of Management Information and Program Control
United States Nuclear Regulatory Commission
Washington, DC 20555

Director of Regulatory Operations (1 copy)
United States Nuclear Regulatory Commission
Washington, DC 20555

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AVERAGE DAILY POWER LEVEL

PROJECT #..... 50-214
 UNIT..... C. C. #1
 REPORT DATE... March 12, 1970
 COMPILED BY... C.N. McCLAIN
 TELEPHONE..... 201-455-8740

MONTH February 1970

| DAY | RM | DAY | RM |
|-----|------|-----|------|
| 1. | 649. | 17. | 649. |
| 2. | 649. | 18. | 649. |
| 3. | 649. | 19. | 647. |
| 4. | 649. | 20. | 648. |
| 5. | 648. | 21. | 648. |
| 6. | 649. | 22. | 648. |
| 7. | 628. | 23. | 648. |
| 8. | 629. | 24. | 647. |
| 9. | 604. | 25. | 647. |
| 10. | 647. | 26. | 647. |
| 11. | 648. | 27. | 647. |
| 12. | 649. | 28. | 646. |
| 13. | 643. | | |
| 14. | 581. | | |
| 15. | 650. | | |
| 16. | 647. | | |

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OPERATING DATA REPORT

OPERATING STATUS

UNIT NAME...OYSTER CREEK

DOCLET NUMBER...56-219

UTILITY DATA PREPARED BY...C.M. HOCLAIR 201-855-8748

REPORTING PERIOD... February 1979

LICENSED THERMAL POWER(MWT)...1930

NAMEPLATE RATING(GROSS MWE)...650

DESIGN ELECTRICAL RATING(NET MWE)...650

MAXIMUM DEPENDABLE CAPACITY(GROSS MWE)...650

1 HOUR DEPENDABLE CAPACITY(NET MWE)...625

IF CHANGES OCCUR IN CAPACITY RATING SINCE LAST REPORT, GIVE REASON FOR CHANGE

LOWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE) ... NO RESTRICTION

REASON FOR RESTRICTION, IF ANY ... NO RESTRICTION

| | MONTH | PERCENT | CUMULATIVE |
|--------------------------|---------|---------|------------|
| HOURS IN PERIOD | 720 | 100 | 38720 |
| HOURS AT CRITICAL | 66 | 9 | 2054 |
| 1 HOUR DEPENDABLE MWT | 625 | 93 | 397 |
| DESIGN CAPACITY MWT | 650 | 100 | 420 |
| DESIGN THERMAL POWER MWT | 1930 | 100 | 38720 |
| GROSS THERMAL ENERGY | 1369200 | 100 | 1369200 |
| GROSS ELEC ENERGY | 400960 | 100 | 1000000 |
| NET ELEC ENERGY | 310394 | 100 | 1074841 |
| UT SERVICE FACTOR | 92.1 | 100 | 75.4 |
| UT AVAILABILITY FACTOR | 92.5 | 100 | 76.0 |
| UT CAPACITY FACTOR MWT | 89.7 | 100 | 39.5 |
| UT CAPACITY FACTOR PER | 85.7 | 100 | 43.5 |
| FORCED OUTAGE FACTOR | 1.3 | 1.2 | 1.2 |

NO OUTAGE IS SCHEDULED OVER THE NEXT 210 MONTHS

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February 1979

DOCKET NO. 50-219
 UNIT NAME Oyster Creek #1
 DATE March 12, 1979
 COMPLETED BY C. M. McClain
 TELEPHONE 201-455-8748

| No | Date | Type ¹ | Duration (Hours) | Reason ² | Method of Shutting Down Reactor ³ | Licensee Event Report # | System Code ⁴ | Component Code ⁵ | Cause & Corrective Action to Prevent Recurrence |
|----|--------|-------------------|------------------|---------------------|--|-------------------------|--------------------------|-----------------------------|---|
| 2 | 790206 | F | 16.5 | G | 3 | NA | CH | RELAYX | "C" feedwater pump tripped when the breaker cubicle door was closed, shaking the "C" differential relay. The scram occurred from a low water level after the pump trip. |

¹
 F Forced
 S Scheduled

²
 Reason
 A Equipment Failure (Explain)
 B Maintenance or Test
 C Refueling
 D Regulatory Restriction
 E Operator Training & License Examination
 F Administrative
 G Operational Error (Explain)
 H Other (Explain)

³
 Method
 1 Manual
 2 Manual Scram.
 3 Automatic Scram.
 4 Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LIR) File (NURIG-0161)

⁵
 Exhibit I - Same Source

(9/77)

OPERATIONS SUMMARY - FEBRUARY 1979

The unit was operating at full load at the beginning of this report period.

On February 6, 1979, "C" feedwater pump tripped when the breaker cubicle door was closed, shaking the "C" differential relay. A reactor scram occurred at 11:15 a.m. from low water level following the feedwater pump trip. All systems functioned normally. The unit was returned to service at 3:41 a.m. on February 7.

On February 9, power level was reduced from rated load to approximately 35% when condenser vacuum rapidly decreased. Steam jet air ejector 1C1 was shutdown and 1C2 was started. Load was increased to rated.

On February 13, power was reduced to approximately 65% to insert control rod 34-39 and replace a leaking scram pilot solenoid valve. Power was increased to rated following withdrawal of rod 34-39.

Two (2) reportable occurrences were identified during the month:

RO 79-2 occurred on February 6 when one (1) reactor high pressure switch for isolation condenser initiation tripped above the allowable setpoint during routine surveillance testing.

RO 79-3 occurred on February 26, 1979 when the drywell to torus differential pressure dropped below the technical specification limit for 10 minutes during a reactor building closed cooling water temperature transient.

CORRECTIVE INSTRUMENT MAINTENANCE ON QASL ITEMS FOR THE MONTH OF FEBRUARY 1979

| Item # | Equipment | Malfunction | Corrective Action |
|--------|--------------------------|-------------------------------------|---|
| 1 | APRM Ch. 6 | Upscale spiking causing half-scrams | Bypassed 12-41B in APRM drawer and aux. drawer |
| 2 | CRD 34-39 | Scram solenoid air leak | Rebuilt scram pilot valves with new parts and tested |
| 3 | APRM Ch. 4 | Red block from faulty diode | Replaced diode and conducted APRM surveillance test & cal. |
| 4 | SIM Front Panel | ICR meter out of spec | Adjusted trip module and zeroed DC amp for 10^{-1} on zero position |
| 5 | 'D' Recirc Pump AP trans | Leaking water at RK03 | Tightened packing on root valve |
| 6 | Core Spray Sys I | Alarms did not flash | Replaced "B" acknowledge button |
| 7 | MFG Front Panel | Potential faulty indications | Adjusted zero and set alarm on recorder and checked recirc. valves |

CORRECTIVE ELECTRICAL MAINTENANCE ON QASL ITEMS FOR THE MONTH OF FEBRUARY 1979

| Item # | Equipment | Malfunction | Corrective Action |
|--------|-----------------------|--------------------------------|--|
| 1 | Service Water Pump #1 | Breaker trip unable to reclose | Changed undervoltage coil and performed PM on breaker |

CORRECTIVE MECHANICAL MAINTENANCE ON QASL ITEMS FOR THE MONTH OF FEBRUARY 1979

| Item # | Equipment | Malfunction | Corrective Action |
|--------|---|---|---|
| 1 | CRD 46-27 HCU | Nipple on fill connection leaking around threads | Removed connector & retapped |
| 2 | Core Spray System | V-20-41 leaking | Adjusted packing to stop leakage |
| 3 | Condensate Transfer Isolation to Isol. Cond. | V-11-41 leaking | Tightened bonnet bolts as necessary to stop leak |
| 4 | Fire Protection System | Packing leak on the pond pump at the fire pump house | Repacked the pump as necessary to stop leak |
| 5 | CRD accumulator 14-11 | V-111 valve malfunction | Replaced valve bonnet with a rebuilt spare |
| 6 | 1-1 RIXCW Pump | Leak from inboard seal | Replaced seals |

REFUELING INFORMATION - FEBRUARY 1979

Name of facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: September 15, 1979

Scheduled date for restart following refueling: November 10, 1979

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

No Technical Specification change relative to the refueling is anticipated.

Scheduled date(s) for submitting proposed licensing action and supporting information:

1. July 1979 - Cycle independent General Electric fuel design information and safety analysis for future use.
2. No submittal is scheduled for the use of Exxon fuel.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - Fuel design and performance analysis methods have been approved by NRC. New operating procedures, if necessary, will be submitted at a later date.
2. Exxon Fuel Assemblies - No major changes have been made, nor are there any anticipated.

The number of fuel assemblies (a) in the core - 560
(b) in the spent fuel storage pool - 620

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

1,800

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

The Fall 1986 Outage.

END