

EDWIN I. HATCH NUCLEAR PLANT  
UNIT 1

CONTAINMENT LEAK RATE TEST PROGRAM

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## I. INTRODUCTION

Contained in this document is a reevaluation of the Plant Hatch Unit 1 Containment Leak Rate Test Program. The original program for Unit 1 was prepared with the intent of meeting the requirements of 10CFR50, Appendix J, "Reactor Containment Leakage Testing for Water Cooled Power Reactors", dated February 14, 1973. Since that time the Plant Hatch Unit 2 Containment Leak Rate Test Program has been reviewed and approved by the NRC. In order to maintain a continuity of test procedures and interpretation of 10CFR50, Appendix J between the two units, a rereview of the Unit 1 program was conducted. Each primary containment penetration on Unit 1 was compared to its similar penetration on Unit 2 and evaluated under the same guidelines used to develop the Unit 2 program.

## II. PROPOSED PLANT MODIFICATIONS

Leakage testing capabilities for each penetration were examined in the as-built condition. Several penetrations have primary containment isolation valves which are presently capable of being leakage rate tested only by pressurizing in a direction opposite to the primary containment pressure. However, their designs do not offer conservatism or justification for testing in the reverse direction. Also identified in the review was an additional penetration whose associated system piping cannot be considered a closed system unless a specific portion of connecting piping is removed. Therefore, modifications are required to enable leakage tests which are in compliance with the requirements of 10CFR50, Appendix J.

A detailed discussion for each of the affected penetrations is provided in this section including proposed plant modifications. It should be noted that the proposed modifications, when compared to Unit 2, were part of the Unit 2 initial design or were made to conform with the requirements of 10CFR50, Appendix J during the Unit 2 licensing process. Therefore, they are needed on Unit 1 to standardize the leakage testing program between the two units.

Penetrations Requiring Modification  
To Conform to 10CFR50, Appendix J

Penetration X25 - Vent Purge Supply:

Valve T48-F118A is a globe valve pressurized in the reverse direction during leakage testing. The valve cannot be conservatively tested in the reverse direction. A blocking valve and test connection installed as shown on Figure 1 will allow testing of valve T48-F118A in the proper direction. The modification shown in Figure 1 will also allow pressurizing of valves T48-F114 and T48-F322 in the correct direction. This arrangement is beneficial, although the two control valves may be conservatively tested as installed. This modification requires one 2" and one 3/4" ASME Section III Class 2 valves.

Penetration X26 - Vent Purge Return/H<sub>2</sub> & O<sub>2</sub> Analyzer:

This penetration has three control valves not conservatively tested in the reverse direction: T48-F335A and B, and P33-F002. As shown on Figure 2, T48-F335A and B can be tested in the correct direction by installing one blocking valve and a single valve test connection in the common line. P33-F002 has a blocking valve upstream and needs only the addition of a single valve test connection. Two 3/4" and one 2" ASME Section III Class 2 valves are required for these modifications.

Penetration X28A - Recirculation Sample

Valve B31-F019 is a globe valve that cannot be conservatively tested in the reverse direction. As shown on Figure 3, a blocking valve is presently installed but a test connection is required between B31-F019 and the blocking valve. Two 3/4" ASME Section III Class 1 valves are required for this modification.

Penetration X28F - H<sub>2</sub> & O<sub>2</sub> Analyzer:

P33-F003 is a control valve non-conservatively pressurized in the reverse direction. As shown on Figure 4, a blocking valve is presently installed, but a test connection is required between P33-F003 and the blocking valve. One 3/4" ASME Section III Class 2 valve is required for this modification.

Penetration X31F - Recirculation Pump Seal Water:

The present system design does not provide testing capabilities for check valves B31-F013A or B31-F017A. Test connections are required as shown in Figure 5. This modification requires four 3/4" ASME Section III Class 2 valves.

Penetration X45F - ILRT Verification Flow:

The inboard isolation globe valve, T23-F004, cannot be conservatively leakage rate tested by applying pressure in the reverse direction. T23-F004 can be tested in the correct direction by installing a flange on the pipe termination inside the drywell and testing through a blind flange with an installed test connection.

Penetration X46 - Demineralized Water:

The present system design has check valve P21-F372 installed between locked closed manual isolation valves P21-F406 and P21-F353. This arrangement results in the utilization of check valve P21-F372 as an isolation valve. The relative position of valves P21-F372 and P21-F353 should be interchanged and the test connection should be between P21-F353 and P21-F406 as shown in Figure 6. P21-F406 should be tested with pressure in the correct direction by pressurizing through a drywell hose connection. No additional valves are required for this modification.

Penetration X59A - Recirculation Pump Seal Water:

This penetration design is identical to X31F (see Figure 5). Testing capabilities are not provided for check valves B31-F013B or B31-F017A. Test connections are required as shown in Figure 5. This modification requires four 3/4" ASME Section III Class 2 valves.

Penetration X205 - Containment Purge and Inerting:

Globe isolation valve T48-F118B cannot be tested conservatively in the reverse direction. The modification shown in Figure 7 is similar to the one discussed for penetration X25.

Penetration X210 - Radwaste Connection:

The radwaste (G11) connection should be removed from Core Spray Loop B as shown in Figure 8. The G11 tie-in consists of quality group D piping and valves and does not meet the quality group B requirement for closed systems. Consequently, the Core Spray piping cannot be considered a closed system unless the G11 connection is removed. An identical situation existed on Unit 2 and was corrected by removing and capping the tie-in after it was determined that the line was not required. The same modification is required on Unit 1.

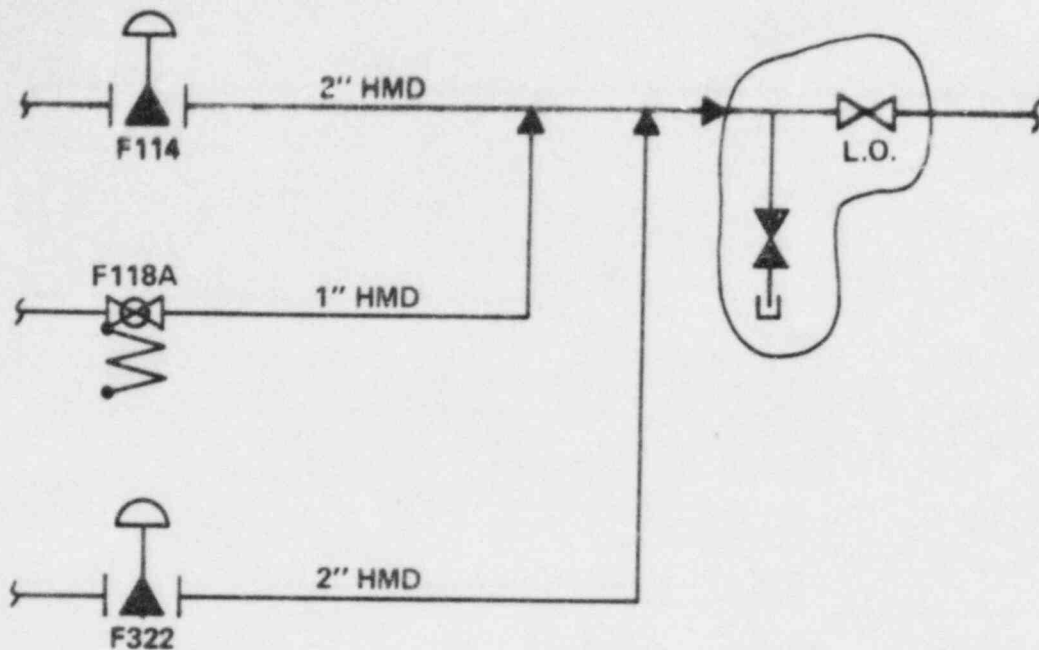
Penetration X217 - H<sub>2</sub> & O<sub>2</sub> Analyzer:

P33-F007 cannot be leakage tested in the correct direction and a reverse pressure test is not conservative. A blocking valve is part of the present system design. Therefore, an installed test connection will provide the required leakage testing capability for this valve (see Figure 9). This modification requires one 3/4" ASME Section III Class 2 valve.

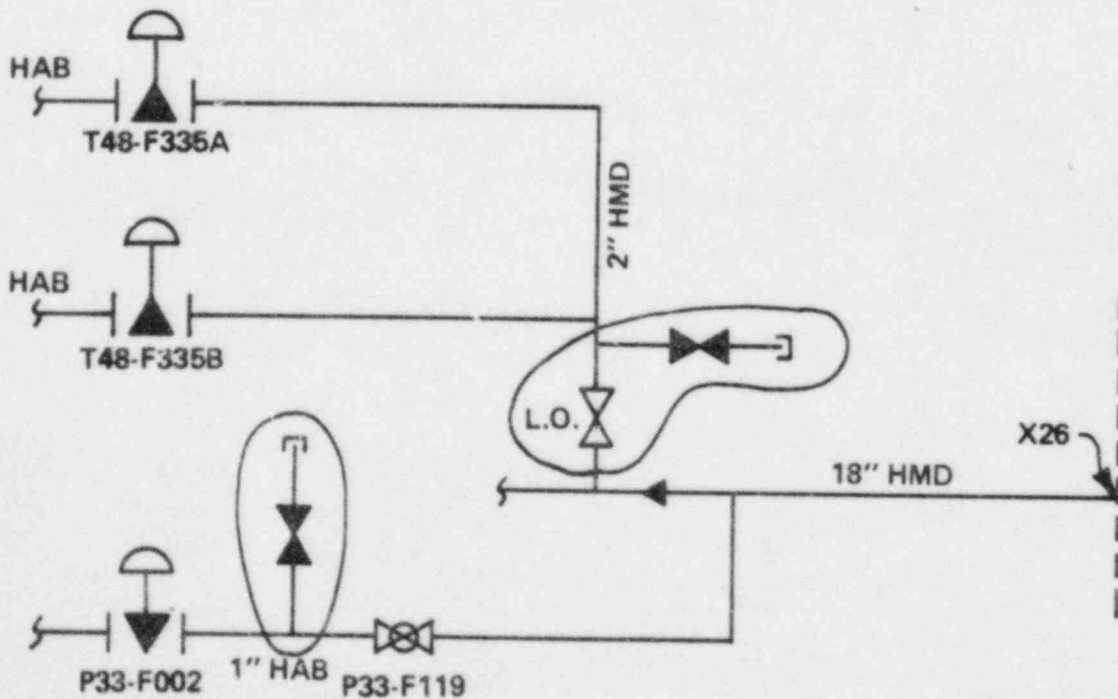
Penetration X220 - Vent Purge Outlet/H<sub>2</sub> & O<sub>2</sub> Analyzer:

Valves T48-F333A/B are control valves that are utilized as primary containment isolation valves. The valves cannot be conservatively tested in the reverse direction and are not presently capable of being tested from the containment direction. The addition of a blocking valve and a test connection as shown in Figure 10 will enable acceptable leakage rate testing of these valves. Isolation valve P33-F006 has a blocking valve installed, but requires the installation of a test connection for leakage rate testing. This modification requires one 2" and two 3/4" ASME Section III, Class 2 valves.

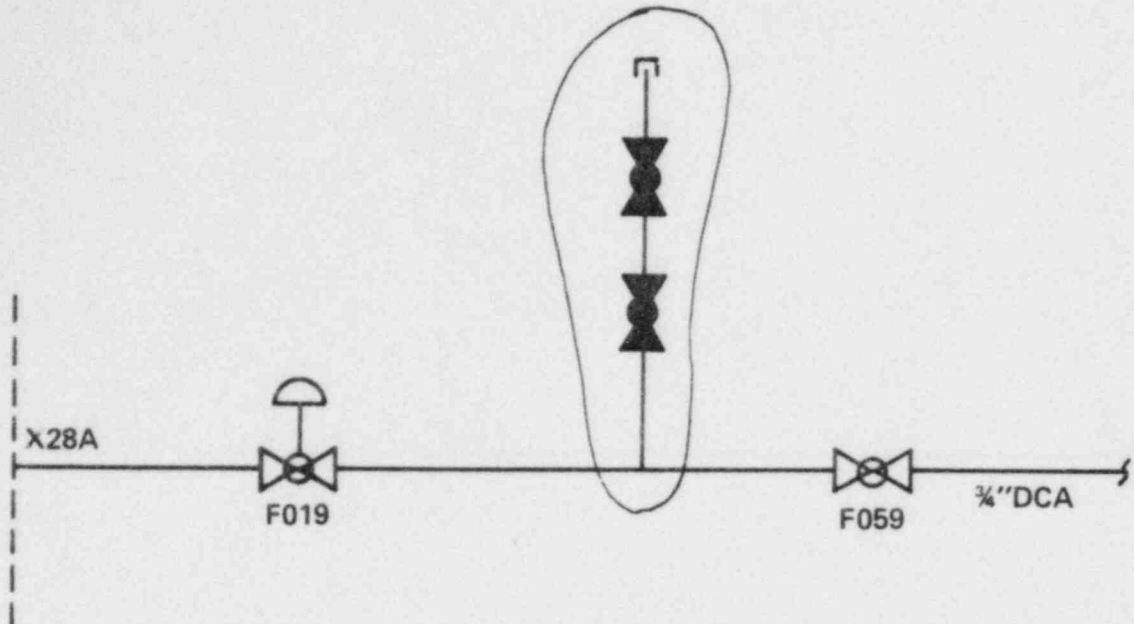




**PENETRATION Y25: ADD A BLOCKING VALVE AND TEST CONNECTION BETWEEN VALVE T48-F118A AND THE PENETRATION**  
 Figure 1 (Reference H-16000)

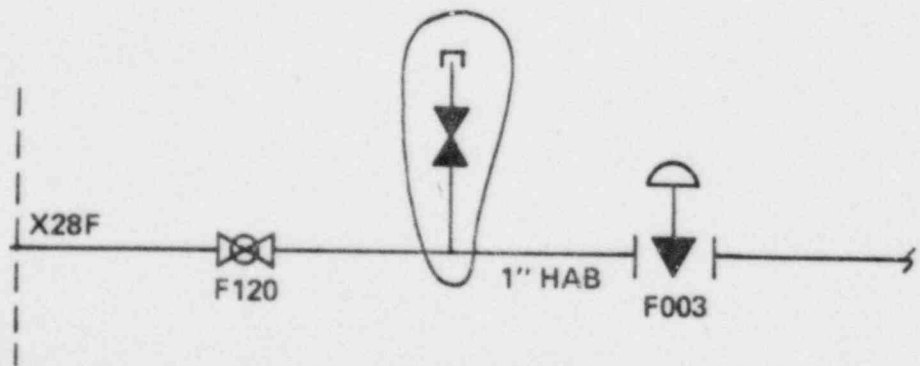


**PENETRATION X26: ADD TEST CONNECTIONS AND A BLOCKING VALVE**  
 Figure 2 (Reference H-16024 and H-16276)



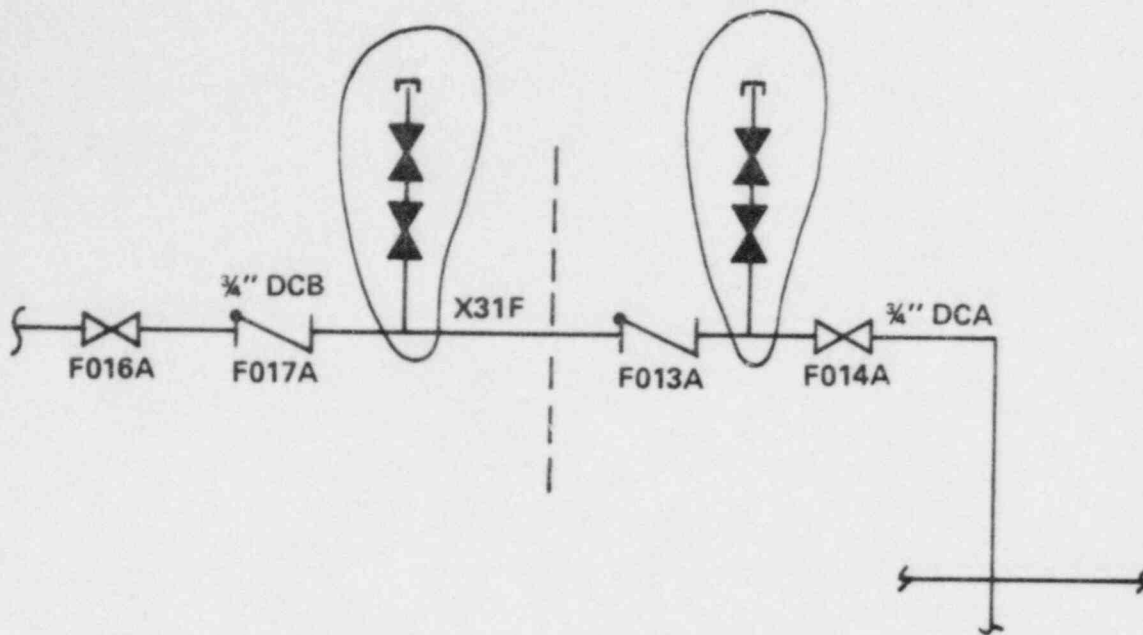
PENETRATION X28A: ADD A TEST CONNECTION

Figure 3 (Reference H-16066)



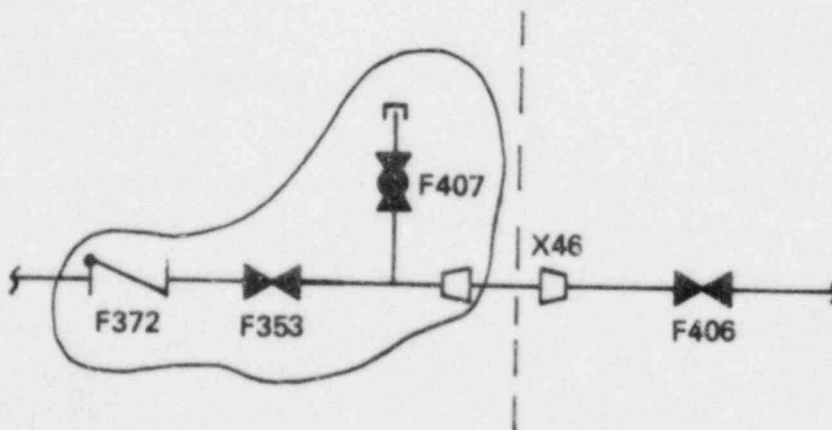
PENETRATION X28F: ADD A TEST CONNECTION

Figure 4 (Reference H-16276)



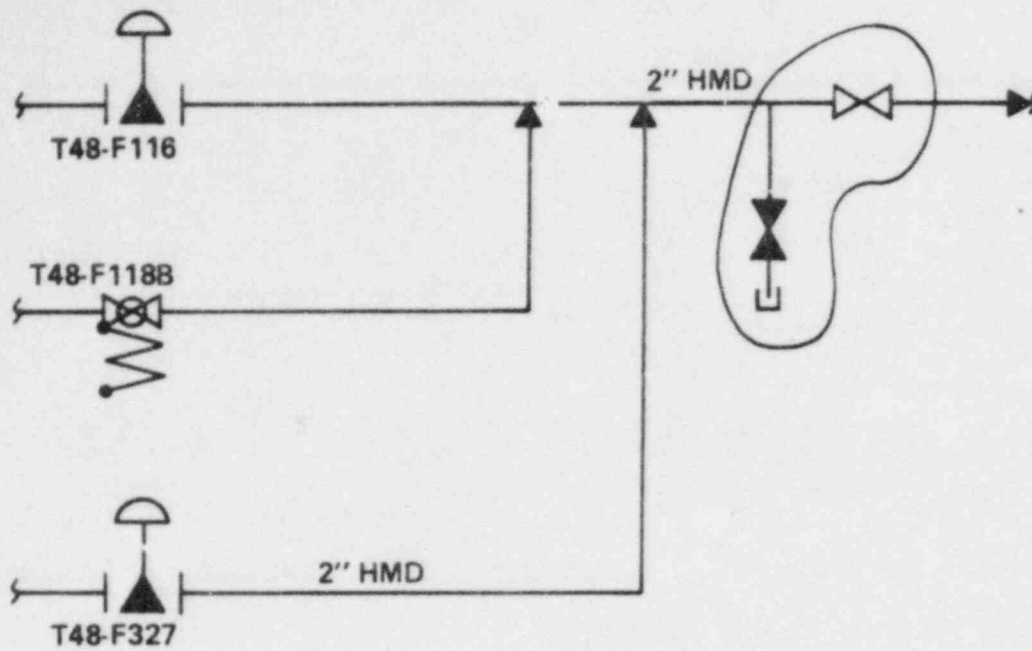
**PENETRATION X31F: ADD TEST CONNECTIONS BETWEEN VALVES B31-F017A, B31-F013A, AND B31-F014A. PENETRATION X59A IS IDENTICAL.**

Figure 5 (Reference H-16066)



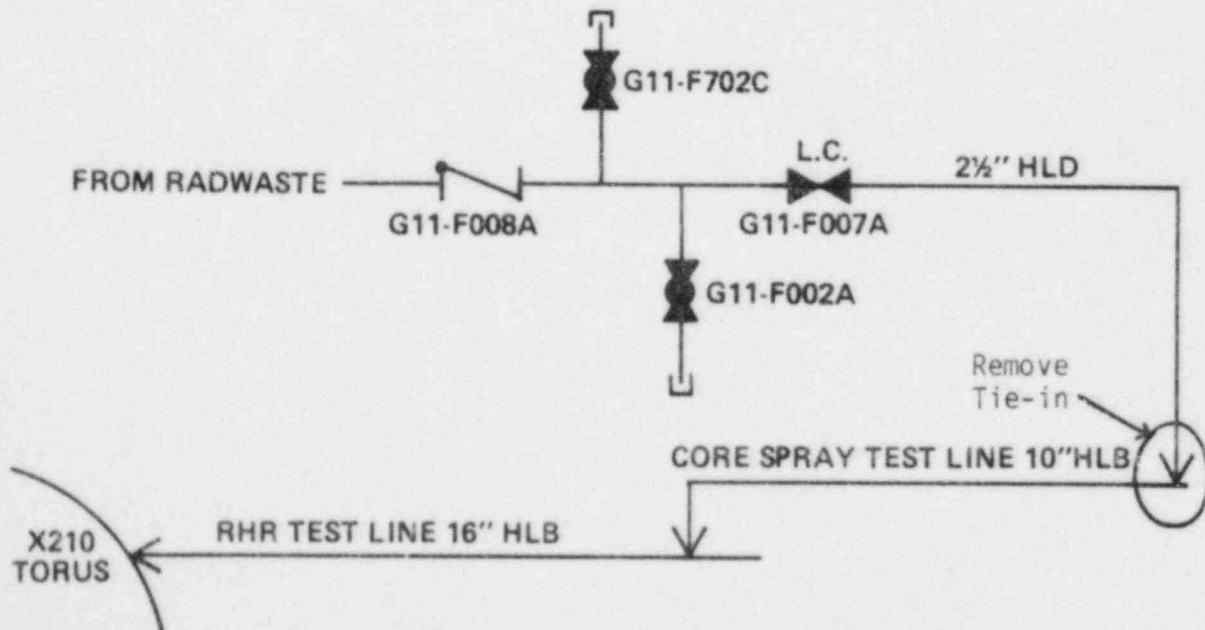
**PENETRATION X46: EXCHANGE POSITION OF P21-F353 AND P21-F372 ENSURE TEST CONNECTION IS BETWEEN P21-F353 AND P21-F406.**

Figure 6 (Reference H-16015)



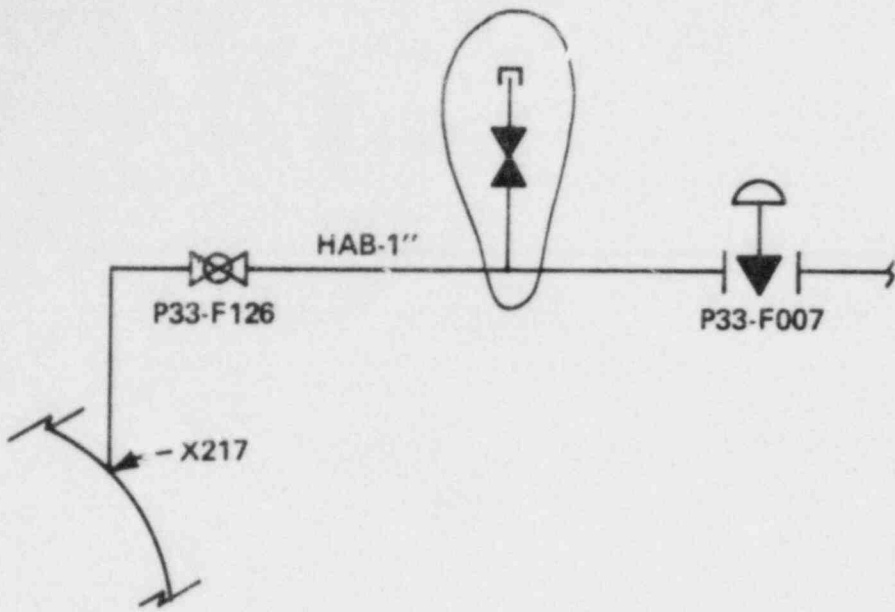
**PENETRATION X205: INSTALL A BLOCKING VALVE AND TEST CONNECTION BETWEEN T48-118B AND THE PENETRATION**

Figure 7 (Reference H-16000)



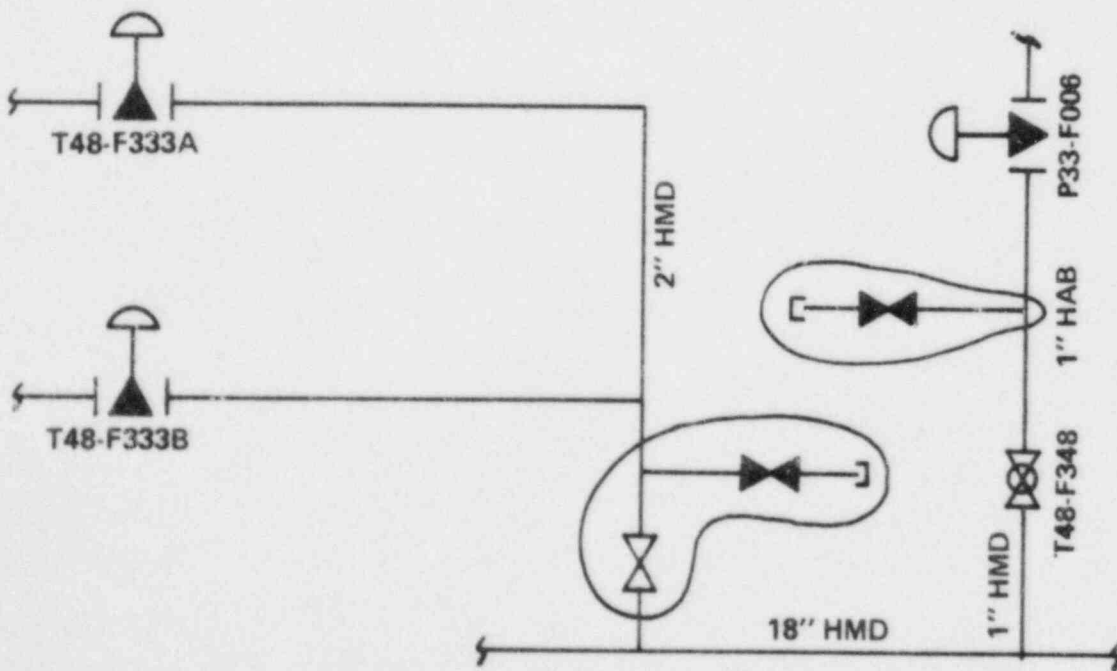
**PENETRATION X210: REMOVE AND CAP G11 TIE-IN TO CORE SPRAY**

Figure 8 (Reference H-16176 and H-16331)



**PENETRATION X217: ADD A TEST CONNECTION BETWEEN P33-F007 AND P33-F126**

Figure 9 (Reference H-16276)



**PENETRATION X220: ADD A BLOCKING VALVE AND TEST CONNECTION BETWEEN T48-F333A/B AND THE PENETRATION, ADD A TEST CONNECTION BETWEEN T48-F348 AND P33-F006.**

Figure 10 (Reference H-16024 and H-16276)

### III. PROGRAM DESCRIPTION:

Provided in this section is a penetration leakage rate test list which describes the inboard and outboard isolation barrier for each of the Unit 1 primary containment penetrations. It has been compiled in tabular form similar to Table 3.8-12 in the Unit 2 FSAR. It has also been formulated assuming all of the proposed plant modifications described in Section II have received NRC approval and have been completed. Piping and Instrumentation Drawings are also provided as referenced in the test schedule.

The basis used to establish testing requirements and acceptance criteria for the Unit 1 program is identical to that used on Unit 2. This includes the use of closed systems outside the primary containment as isolation barriers. In most instances, the valve chosen as the primary containment isolation valve for the closed system is the outboard isolation valve.

CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
1A	Equipment Hatch	B	Double O Rings	16			
1B	Equipment Hatch	B	Double O Rings	16			
2	Personnel Lock Inner Door Outer Door Barrel	B	Double O Rings	16, 20	Outer Door		
		B	Double O Rings	16, 20			
		B	Inner Door				
4	Head Access Hatch	B	Double O Rings	16			
5A-H	Vent Line	B	Expansion Bellows	15			
6	Control Rod Drive Removal Hatch	B	Double O Rings	16			
7A	Main Steam	C	B21-F022A	5,13,14	B21-F028A	13, 14	H-16062
		B	Expansion Bellows	15			
7B	Main Steam	C	B21-F022B	5,13,14	B21-F028B	13, 14	H-16062
		B	Expansion Bellows	15			
7C	Main Steam	C	B21-F022C	5,13,14	B21-F028C	13, 14	H-16062
		B	Expansion Bellows	15			
7D	Main Steam	C	B21-F022D	5,13,14	B21-F028D	13, 14	H-16062
		B	Expansion Bellows	15			
8	Condensate Drain	C	B21-F016	24	B21-F019		H-16062
		B	Expansion Bellows	15			
9A	Primary Feedwater	C	B21-F010B	12	B21-F032B E41-F006	12	H-16062 H-16332
		B	Expansion Bellows	15		12	

## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
9B	Primary Feedwater	C	B21-F010A	12	B21-F032A E51-F013 G31-F039	12 12 12	H-16062 H-16334 H-16188
		B	Expansion Bellows	15			
10	Steam to RCIC Turbine	C	E51-F007	24	E51-F008		H-16334
		B	Expansion Bellows	15			
11	Steam to HPCI Turbine	C	E41-F002	24	E41-F003		H-16332
		B	Expansion Bellows	15			
12	RHR Suction	C	E11-F008		Closed System	21	H-16329
		B	Expansion Bellows	15			
13A	RHR Return to Recirculation	C	E11-F015A		Closed System	21	H-16330
		B	Expansion Bellows	15			
13B	RHR Return to Recirculation	C	E11-F015B		Closed System	21	H-16329
		B	Expansion Bellows	15			
14	Reactor Water Cleanup Supply	C	G31-F001		G31-F004		H-16188
		B	Expansion Bellows	15			
15	Spare	A					
16A	Core Spray	C	E21-F005A		Closed System	21	H-16331
		B	Expansion Bellows	15			H-16328
16B	Core Spray	C	E21-F005B		Closed System	21	H-16331
		B	Expansion Bellows	15			H-16328
17	RPV Head Spray	C	E11-F023		Closed System	21	H-16329
		B	Expansion Bellows	15			H-16328
			III-3				



## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
18	Equipment Drain Pump Discharge	C	G11-F019		G11-F020		H-16176
19	Floor Drain Pump Discharge	C	G11-F003		G11-F004		H-16176
20	Service Water Supply	C	Closed System	19	P41-F049	25, 12	H-16011
21	Service Air	C	P51-F514		P51-F513		H-16013
22	Drywell Pneumatic Supply	C	P70-F004, P70-F005		P70-F020		H-16286
23	RBCCW Supply	C	Closed System	19	P42-F051	25, 12	H-16009
24	RBCCW Return	C	Closed System	19	P42-F052	25, 12	H-16009
25	Vent Purge Supply	C	T48-F307	7	T48-F308, T48-F324, T48-F108		H-16024
		C	T48-F114		T48-F113		H-16000
		C	T48-F118A		T48-F104		
		C	T48-F322		T48-F321		
		B	Butterfly Valve O Rings				
26	Vent Purge Return/H <sub>2</sub> & O <sub>2</sub> Analyzer	C	T48-F319	7	T48-F320		H-16024
		C	T48-F341	5	T48-F340		
		C	T48-F335B		T48-F334B		
		C	T48-F335A		T48-F334A		
		C	P33-F002		P33-F010		H-16276
27A	Spare	A					
27B	Spare	A					

## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
27C	Drywell Pressure	A	E11-F043A, E11-F037A	2			H-16330
27D	Drywell Pressure	A	E11-F043C, E11-F037C	2			H-16330
27E	Drywell Pressure	A	T48-F304B	2			H-16024
27F	Spare	A					
28A	Recirculation Sample	C	B31-F019		B31-F020		H-16066
28B	RPV Instrumentation	A	B21-F047B	1			H-16063
28C	RPV Instrumentation	A	B21-F045B	1			H-16063
28D	RPV Instrumentation	A	B21-F065B	1			H-16063
		A	B21-F049B	1			H-16063
28E	RPV Instrumentation	A	B21-F043B	1			H-16063
28F	H <sub>2</sub> & O <sub>2</sub> Analyzer	C	P33-F003		P33-F011		H-16276
29A	Spare	A					
29B	RPV Instrumentation	A	B21-F047A	1			H-16063
29C	RPV Instrumentation	A	B21-F045A	1			H-16063
29D	RPV Instrumentation	A	B21-F065A	1			H-16063
		A	B21-F049A	1			H-16063
29E	RPV Instrumentation	A	B21-F043A	1			H-16063

## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
29F	RPV Instrumentation	A	B21-F041	1			H-16063
30A	RPV Instrumentation	A	B21-F055	1			H-16063
30B	RPV Instrumentation	A	B21-F057	1			H-16063
30C	Main Steam Flow	A	B21-F015G	1			H-16062
30D	Main Steam Flow	A	B21-F015H	1			H-16062
30E	HPCI Steam Flow	A	E41-F024B	1			H-16332
30F	HPCI Steam Flow	A	E41-F024D	1			H-16332
31A	Recirculation Loop Instrumentation	A	B31-F009A	1			H-16066
		A	B31-F009D	1			H-16066
31B	Recirculation Loop Instrumentation	A	B31-F010A	1			H-16066
		A	B31-F010D	1			H-16066
31C	Spare	A					
31D	H <sub>2</sub> & O <sub>2</sub> Analyzer	C	P33-F004	6	P33-F012		H-16276
31E	Spare	A					
31F	Recirculation Pump Seal Water	C	B31-F013A		B31-F017A		H-16066
32A	Recirculation Loop Instrumentation	A	B31-F040A	1			H-16066
32B	Recirculation Loop Instrumentation	A	B31-F040C	1			H-16066
32C	Recirculation Loop Instrumentation	A	B31-F057A	1			H-16066

## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
32D	Spare	A					
32E	Recirculation Loop Instrumentation	A	B31-F003A	1			H-16066
32F	Recirculation Loop Instrumentation	A	B31-F004A	1			H-16066
33A	Recirculation Loop Instrumentation	A	B31-F011A	1			H-16066
			B31-F011D	1			H-16066
33B	Recirculation Loop Instrumentation	A	B31-F012A	1			H-16066
			B31-F012D	1			H-16066
33C	Spare	A					
33D	Spare	A					
33E	Spare	A					
33F	Spare	A					
34A	Recirculation Loop Instrumentation	A	B31-F040B	1			H-16066
34B	Recirculation Loop Instrumentation	A	B31-F040D	1			H-16066
34C	Recirculation Loop Instrumentation	A	B31-F003B	1			H-16066
34D	Recirculation Loop Instrumentation	A	B31-F004B	1			H-16066
34E	Spare	A					
34F	Recirculation Loop Instrumentation	A	B31-F057B	1			H-16066
35A	Tip Drive	C	Ball Valve A		Shear Valve A	22	H-16070
		B	Double O Ring	16			H-16070
			III-7				

## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
35B	Tip Drive	C B	Ball Valve B Double O Ring	16	Shear Valve B	22	H-16070 H-16070
35C	Tip Drive	C B	Ball Valve C Double O Ring	16	Shear Valve C	22	H-16070 H-16070
35D	Tip Drive	C B	Ball Valve D Double O Ring	16	Shear Valve D	22	H-16070
35E	Tip N <sub>2</sub> Purge	C B	Check Valve Double O Ring	16	Solenoid Valve		H-16070
36	Spare	B	Welded Cap		Welded Cap		
37A-D	CRD Insert	A			Hydraulic Control Unit	23	
38A-D	CRD Withdraw	A			Scram Discharge Header	23	
39A	Containment Spray	C	E11-F016A		Closed System	21	H-16330 H-16328
39B	Containment Spray	C	E11-F016B		Closed System	21	H-16329 H-16328 H-16066
40A-A	Recirculation Loop Instrumentation	A	B31-F055B	1			H-16066
40A-B	Recirculation Loop Instrumentation	A	B31-F055D	1			H-16066
40A-C	RPV Instrumentation	A	E21-F018A	1			H-16331
40A-D	Spare	A					
40A-E	Recirculation Loop Instrumentation	A	B31-F055E	1			H-16066
40A-F	Recirculation Loop Instrumentation	A	B31-F055G	1			H-16066

## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier	Notes	Outboard Isolation Barrier	Notes	P & ID No.
408-A	Spare	A					
408-B	Spare	A					
408-C	Spare	A					
408-D	Spare	A					
408-E	RCIC Steam Flow	A	E51-F044B	1			H-16334
408-F	RCIC Steam Flow	A	E51-F044D	1			H-16334
40C-A	Spare	A					
40C-B	Spare	A					
40C-C	Spare	A					
40C-D	Spare	A					
40C-E	Spare	A					
40C-F	Drywell Pneumatic Outlet	C	P70-F002		P70-F003		H-16286
40D-A	RPV Instrumentation	A	E21-F018B	1			H-16331
40D-B	Spare	A					
40D-C	Recirculation Loop Instrumentation	A	B31-F055F	1			H-16066
40D-D	Recirculation Loop Instrumentation	A	B31-F055H	1			H-16066
40D-E	Recirculation Loop Instrumentation	A	B31-F05EA	1			H-16066
						III-9	

## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
40D-F	Recirculation Loop Instrumentation	A	B31-F055C	1			H-16066
41	Spare	A					
42	Standby Liquid Control	C	C41-F007		C41-F006		H-16061
43	Drywell Test and Fill	B	Double O Ring	16			
44	Service Water Return	C	Closed System	19	P41-F050	25, 12	H-16011
45A	HPCI Steam Instrumentation	A	E41-F024A	1			H-16332
45B	HPCI Steam Instrumentation	A	E41-F024C	1			H-16332
45C	Drywell Pressure	A	E11-F043D, E11-F037D	2			H-16329
45D	Drywell Pressure	A	E11-F043B, E11-F037B	2			H-16329
45E	Drywell Pressure	A	T48-F303B	2			H-16024
45F	ILRT Verification Flow	C	T23-F004		T23-F005		H-16060
46	Demineralized Water	C	P21-F406		P21-F353		H-16015
47	Spare	A					
49A	Jet Pump Instrumentation	A	B21-F058A	1			H-16063
49B	Jet Pump Instrumentation	A	B21-F059G	1			H-16063
49C	Jet Pump Instrumentation	A	B21-F059E	1			H-16063
49D	Jet Pump Instrumentation	A	B21-F059A	1			H-16063

## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
49E	Jet Pump Instrumentation	A	B21-F059C	1			H-16063
49F	Jet Pump Instrumentation	A	B21-F051A	1			H-16063
50A	Jet Pump Instrumentation	A	B21-F053B	1			H-16063
50B	Jet Pump Instrumentation	A	B21-F059H	1			H-16063
50C	Jet Pump Instrumentation	A	B21-F059F	1			H-16063
50D	Jet Pump Instrumentation	A	B21-F059B	1			H-16063
50E	Jet Pump Instrumentation	A	B21-F059D	1			H-16063
50F	Jet Pump Instrumentation	A	B21-F051B	1			H-16063
51A	Jet Pump Instrumentation	A	B21-F059M	1			H-16063
51B	Jet Pump Instrumentation	A	B21-F053D	1			H-16063
51C	Jet Pump Instrumentation	A	B21-F059U	1			H-16063
51D	Jet Pump Instrumentation	A	B21-F059P	1			H-16063
51E	Jet Pump Instrumentation	A	B21-F059S	1			H-16063
51F	Jet Pump Instrumentation	A	B21-F051D	1			H-16063
52A	Jet Pump Instrumentation	A	B21-F059L	1			H-16063
52B	Jet Pump Instrumentation	A	B21-F053C	1			H-16063



## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
52C	Jet Pump Instrumentation	A	B21-F059T	1			H-16063
52D	Jet Pump Instrumentation	A	B21-F059N	1			H-16063
52E	Jet Pump Instrumentation	A	B21-F059E	1			H-16063
52F	Jet Pump Instrumentation	A	B21-F051C	1			H-16063
53A-F	Power Test	A			Welded Cap		
54A	RPV Instrumentation	A	B21-F061	1			H-16063
54B	RPV Instrumentation	A	E21-F018C	1			H-16331
54C	Main Steam Instrumentation	A	B21-F015K	1			H-16062
54D	Main Steam Instrumentation	A	B21-F015J	1			H-16062
54E	RCIC Steam Instrumentation	A	E51-F044A	1			H016334
54F	RCIC Steam Instrumentation	A	E51-F044C	1			H-16334
59A	Recirculation Pump Seal Water	C	B31-F013B		B31-F017B		H-16066
59B	Recirculation Loop Instrumentation	A	B31-F009C	1			H-16066
		A	B31-F009B	1			H-16066
59C	Recirculation Loop Instrumentation	A	B31-F010B	1			H-16066
		A	B31-F010C	1			H-16066
59D	Spare	A					H-16066

CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
59E	Recirculation Loop Instrumentation	A	B31-F012B B31-F012C	1			H-16066 H-16066
59F		A		1			
60A	Recirculation Loop Instrumentation	A	B31-F011B B31-F011C	1			H-16066 H-16066
60B		A		1			
61A	Spare	A					
61B	Spare	A					
62	Spare	A					
100A	Neutron Monitoring	B	Canister	17			
100B	Neutron Monitoring	B	Canister	17			
100C	Spare	A					
100D	Neutron Monitoring	B	Canister	17			
100E	Neutron Monitoring	B	Canister	17			
100F-A	Main Steam Flow	A	B21-F015C	1			H-16062
100F-B	Main Steam Flow	A	B21-F015M	1			H-16062
100F-C	Main Steam Flow	A	B21-F015L	1			H-16062
			III-13				

## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
100F-D	Main Steam Flow	A	B21-F015S	1			H-16062
100F-E	Main Steam Flow	A	B21-F015R	1			H-16062
100F-F	Main Steam Flow	A	B21-F015D	1			H-16062
101A-F	Recirculation Pump Power	B	Canister	17			
102A	Indication and Control	B	Canister	17			
102B	Spare	A					
103A	Indication and Control	B	Canister	17			
103B-A	Main Steam Instrumentation	A	B21-F015A	1			H-16062
103B-B	Main Steam Instrumentation	A	B21-F015N	1			H-16062
103B-C	Main Steam Instrumentation	A	B21-F015P	1			H-16062
103B-D	Main Steam Instrumentation	A	B21-F015F	1			H016062
103B-E	Main Steam Instrumentation	A	B21-F015E	1			H016062
103B-F	Main Steam Instrumentation	A	B21-F015B	1			H016062
104A,B,C	CRD Rod Position Indication	B	Canister	17			
104D, E	Spare	A					
104F,G,H	CRD Rod Position Indication	B	Canister	17			
105A	600 Volt Power	B	Canister	17			

## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
105B	Spare	A					
105C	600 Volt Power	B	Canister	17			
105D	Spare	A					
106A	Spare	A					
106B	Thermocouples	B	Canister	17			
107A, B	Spare	A					
108A	Grounding Rod	A	Welded				
108B	Grounding Rod	A	Welded				
200A	Torus Access Hatch	B	Double O Rings	16			
200B	Torus Access Hatch	B	Double O Rings	16			
201A-H	Drywell to Torus Vent Lines	B	Expansion Bellows	15			
202	Control and Indication	B	Canister	17			
203	RCIC Pump Suction	C	E51-F003	7, 12	E51-F031	12	H-16334
204A	RHR Pump Suction	C	E11-F004A, F030A	12	Closed System	21	H-16330
204B	RHR Pump Suction	C	E11-F004B, F030B	12	Closed System	21	H-16329
204C	RHR Pump Suction	C	E11-F004C, F030C	12	Closed System	21	H-16330

## CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
204D	RHR Pump Suction	C	E11-F004D, F030D	12	Closed System	21	H-16329
205	Containment Purge and Inerting	A	T48-F302	2			H-16024
		A	T48-F303A	2			
		A	T48-F301	2			
		C	T48-F311	7	T48-F328B		
		C	T48-F310	7	T48-F328A		
		C	T48-F309	7	T48-F324		
		C	T48-F116		T48-F115		
		C	T48-F118B		T48-F104		
		C	T48-F327		T48-F325		
		B	Butterfly Valve O Rings	16			
206A	Torus Water Level	A	T48-F331B	2			H-16024
		A	E41-F109	2			H-16332
206B	Torus Water Level	A	T48-F331A	2			H-16024
		A	E41-F110	2			H-16332
206C	Torus Water Level	A	T48-F330B	2			H-16024
		A	E41-F107	2			H-16332
206D	Torus Water Level	A	T48-F330A	2			H-16024
		A	E41-F108	2			H-16332
206E-H	Spare	A					
207	HPCI Pump Suction	C	E41-F051	7, 12	E41-F042	12	H-16332
208A	Core Spray Pump Suction	C	E21-F001A	12	Closed System	21	H-16331
208B	Core Spray Pump Suction	C	E21-F001B	12	Closed System	21	H-16331

CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.		
				Notes		Notes			
209A-D	Torus Water Temperature	A	Welded Thermowells						
210A	RHR/Core Spray Test Line	C	E11-F028A	11	Closed System	21	H-16330		
		C	E11-F007A	3, 11	Closed System	21	H-16331		
		C	E11-F011A	3, 11	E11-F026A, Closed System	11, 21	H-16334		
		C	E11-F103A	10	Closed System	21			
		C	E11-F055A	9	Closed System	21			
		C	Thermal Relief	9	Closed System	21			
		C	E11-F029	9	Closed System	21			
		C	E11-F025A	9	Closed System	21			
		C	E21-F015A	5, 11	Closed System	21			
		C	E21-F036A	10, 11	Closed System	21			
		C	E51-F019	4, 11	E51-F021	11			
		210B	RHR/Core Spray Test Line	C	E11-F028B	11	Closed System	21	H-16329
				C	E11-F007B	3, 11	Closed System	21	H-16331
C	E11-F011B			3, 11	E11-F026B, Closed System	11, 21			
C	E11-F103B			10	Closed System	21	H-16176		
C	E11-F055B			9	Closed System	21	H-16182		
C	Thermal Relief			9	Closed System	21			
C	E11-F025B			9	Closed System	21	H-16332		
C	E21-F015B			5, 11	Closed System	21			
C	E11-F097			9	Closed System	21			
C	E21-F036B			10, 11	Closed System	21			
211A	Torus Spray	C	E41-F012	4, 11	E41-F046	11			
		C	E11-F028A		Closed System	21	H-16330		

CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier		Outboard Isolation Barrier		P & ID No.
				Notes		Notes	
211B	Torus Spray	C	E11-F028B		Closed System	21	H-16329
212	RCIC Turbine Exhaust	C	E51-F001	8, 11	E51-F040	11	H-16334
213	RCIC Turbine Vacuum Pump Discharge	C	E51-F002	8, 11	E51-F028	11	H-16334
214	HPCI Turbine Exhaust	C	E41-F021	8, 11	E41-F049	11	H-16332
215	HPCI Exhaust Drain	C	E41-F022	8, 11	E41-F040	11	H-16332
216A-D	Torus Air Temperature	A	Welded Thermowell				
217	H <sub>2</sub> & O <sub>2</sub> Analyzer	C	P33-F007		P33-F015		H-16276
218A, B	Construction Drain	B	Flange, Double O Rings	16			
220	Vent Purge Outlet	A	T48-F304A	2			
		C	T48-F333A		T48-F332A		H-16024
		C	T48-F333B		T48-F332B		
		C	T48-F318	7	T48-F326		
		C	T48-F339	5	T48-F338		
		C	P33-F006		P33-F014		H-16276
221A	Spare	A					
221B	Spare	A					
221C	RCIC Turbine Exhaust Vacuum Breaker	C	E51-F105	24	E51-F104		H-16334
222A	HPCI Turbine Exhaust Vacuum Breaker	C	E41-F111	24	E41-F104		H-16332
222B	Spare	A					
223A A-F	Control Air for Vacuum Breaker	C	Air Cylinder	18	T48-F342 G-L	4	H-16024

CONTAINMENT LEAK RATE TEST PROGRAM

Penetration No.	Description	Type Test	Inboard Isolation Barrier Notes	Outboard Isolation Barrier Notes	P & ID No.
223B A-F	Control Air for Vacuum Breaker	C	Air Cylinder 18	T48-F342 A-F 4	H-16024



## NOTES

1. Seismic Category I instrument line with an orifice and excess-flow check valve (EFCV). The EFCV is subjected to operability testing in accordance with the Technical Specifications. This line does not isolate during a LOCA and can leak only if the line or instrument should rupture.
2. Instrument line does not communicate with the reactor coolant pressure boundary. The isolation valve is manually operated and the design satisfies the requirements of Regulatory Guide 1.11, with backfit supplement, for plants which have had a Construction Permit hearing prior to December 30, 1969. Type C testing is not required by 10CFR50, Appendix J. The lines are subjected to pressure during the Type A test.
3. Gate valve tested in the reverse direction.
4. Globe valve tested in the reverse direction.
5. Globe valve tested in the reverse direction. Conservative test: test pressure tends to unseat the disc. (See Figures 11 and 12)
6. Control valve tested in the reverse direction. Conservative test: test pressure tends to unseat the disc. (See Figure 13)
7. Butterfly valve tested in the reverse direction. Same seating surface is tested when test pressure is applied from either direction. (See Figures 14 and 15)
8. Stop check valve pressurized in the reverse direction. Test pressure tends to lift the disc from the seat; therefore, the reverse pressure test is conservative. (See Figure 16)
9. Isolation barrier is the discharge side of a relief valve. The relief valve is tested in the inlet direction; conservative test since containment pressure tends to seat the disc and test pressure tends to unseat it.
10. Untestable globe or check valve; leakage prevented by a closed system.
11. Valve is sealed from the primary containment atmosphere because its line terminates below the water level of the torus. Leakage is not included in the  $0.60 L_a$  Types B and C tests local leakage totals.
12. System remains water filled post LOCA. Isolation valves are tested with water at a pressure of  $1.10 P_a$ . Leakage is not included in the  $0.60 L_a$  Types B and C tests local leakage totals.

13. Tested at one-half  $P_a$ .
14. MSIV leakage rate shall not exceed 11.5 scfh for any valve. Leakage is not included in 0.60  $L_a$  acceptance criteria for Type B and C tests.
15. Penetration has a double-ply, bellows-type seal which will be tested by pressurizing between the two plies through a test connection.
16. Penetration is sealed by a blind flange or door with double O-ring seals. These seals are leakage rate tested by pressurizing between the O-rings.
17. Electrical penetrations are tested by pressurizing between the seals through a valved test connection.
18. The inboard isolation barrier is the vacuum breaker exercising cylinder. The barrier is provided by seals on the air-operated piston. The exercising cylinder, although not Quality Group B, was specified by the vacuum breaker vendor to be qualified to the postulated post-LOCA environment. The cylinder is designed to operate with an air pressure of 95 to 100 psig, which is significantly higher than the post-LOCA containment pressure and is Type C leakage rate tested.
19. The inboard isolation barrier is a closed system inside primary containment. The closed system is subject to the in-service inspection requirements of ASME Section XI for Nuclear Class 3 piping. The system remains water filled post-LOCA and is, therefore, pressurized with water to 1.10 Pa during the Type C test. In accordance with 10CFR50, Appendix J, Paragraph III.C.3, the leakage is excluded from the 0.60  $L_a$  criteria. Leakage acceptance criteria are based upon maintaining a 30-day inventory of water.
20. The personnel air lock door seals shall be tested at 10 psig; the barrel shall be tested at  $P_a$ . The lock barrel test leakage rate shall not exceed 0.05  $L_a$ .
21. The outboard isolation barrier is a closed system outside primary containment. The closed system is subject to the in-service inspection requirements of the ASME Code Section XI for Nuclear Class 2 piping, which requires that any visible leakage be repaired. The system is filled with water and operating at a pressure greater than  $P_a$ , post-LOCA. Leakage is not included in the .60  $L_a$  Types B and C local leakage totals.
22. The operation of the TIP drive shear valve is described in FSAR Section 5.2.3.5.2. Since the shear valve isolates the TIP tubing by shearing the tube and drive cable and by jamming the sheared ends of the tubing into a teflon coating on the shear valve disc, the valve can not be Type C tested without destroying the drive tube. Therefore, the TIP shear valves are not Type C tested. However, each lot of valves are sample leakage tested by the manufacturer prior to delivery. Failure of a single valve to meet the  $10^{-2}$ cc/sec leakage criteria set for the leakage test results in the rejection of the entire

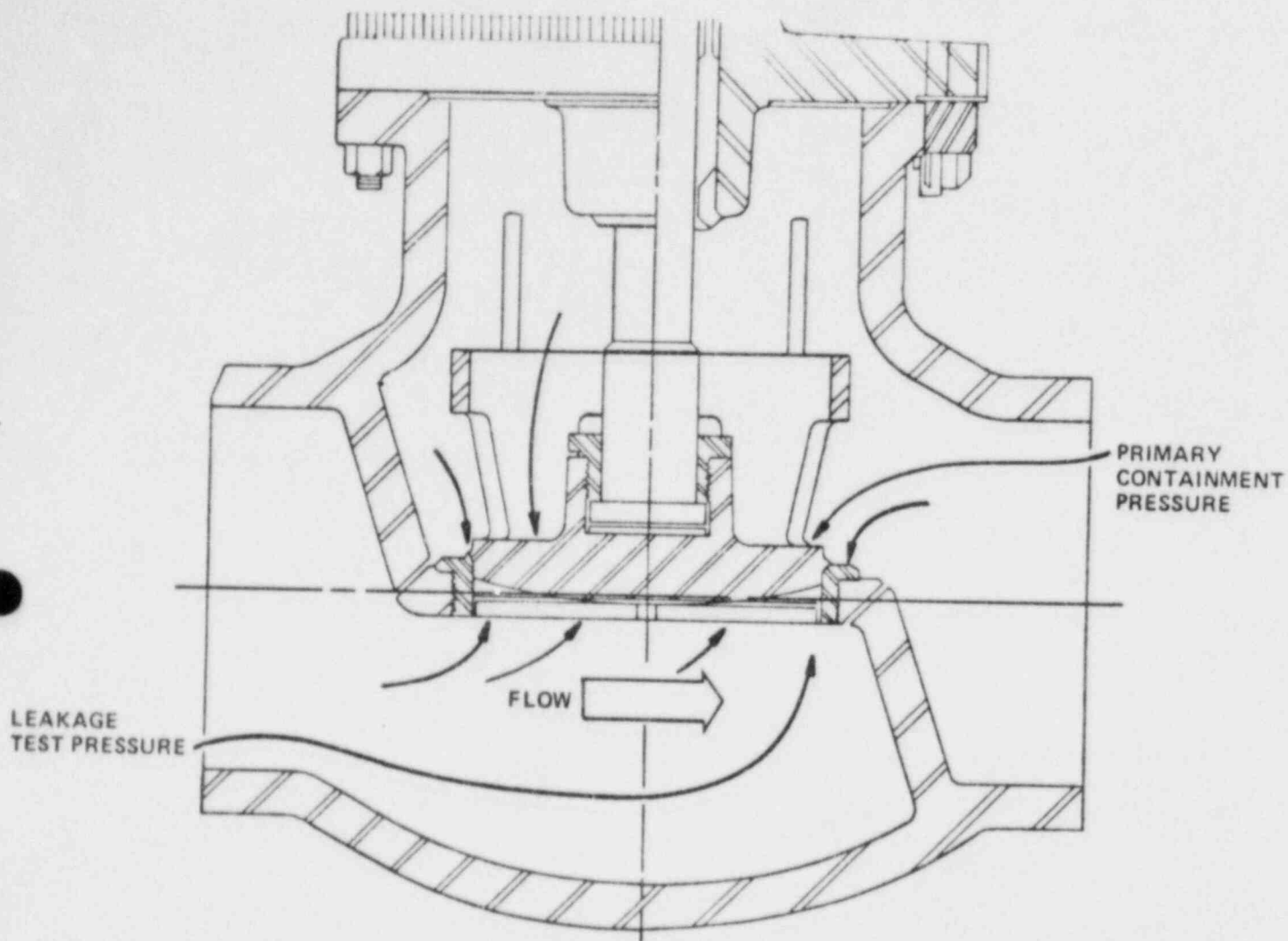
lot. Explosive charges, which operate the shear valves, are in-service inspected in accordance with the requirements of ASME Section XI.

23. The design of the CRD insert and withdraw lines is discussed in FSAR Section 5.2.3.5.1. The design of these lines does not facilitate Type C testing as described in 10CFR50, Appendix J. However, adequate leakage monitoring of the CRD lines is provided by normal plant operating procedures and the Type A leakage rate tests. Since the insert and withdraw lines are pressurized to at least reactor operating pressure by the cooling water flow during normal plant operation, leakage from these lines would be immediately evident.

The hydraulic control units are installed on E1, 130' of the reactor building, a relatively high traffic area. In addition, the HNP-2 Daily Rounds procedure requires that an operator make a visual inspection for leakage in the CRD hydraulic area of the reactor building at least once per shift and that he record the inspection.

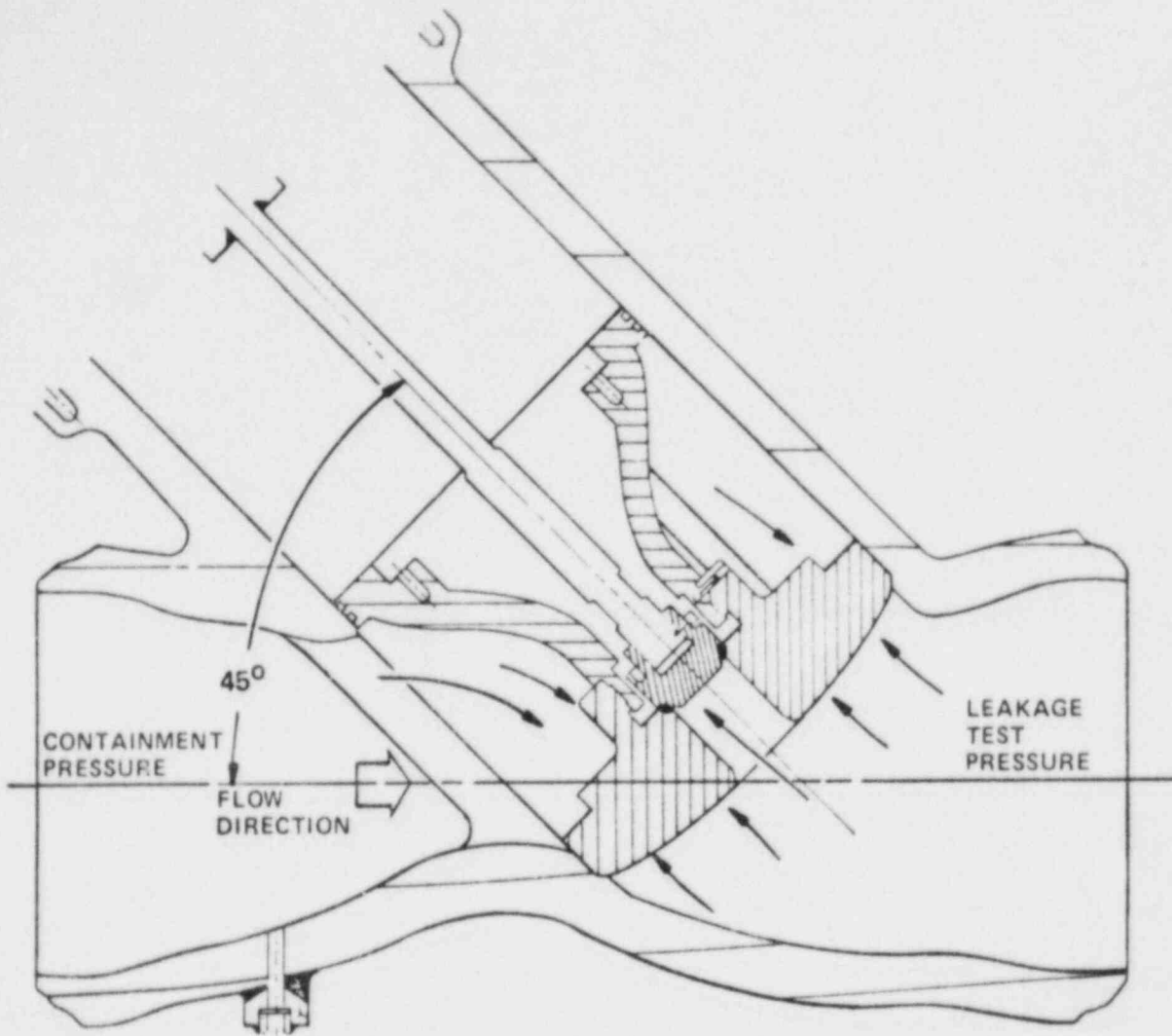
The RPV and the nonseismic portions of the CRD system are vented during the performance of the Type A test. Therefore, leakage from the insert and withdraw lines will be included in the total Type A test leakage.

24. Gate valve tested in the reverse direction. A generic leakage test is performed for this valve since the opposite seat is tested on the outboard valve. Both isolation valves are the same design and, therefore, have similar leakage characteristics. This valve is also subjected to Pa during the Type A test. Piping outboard of the second primary containment isolation valve is seismic Category I, Quality Group B.
25. Local leakage rate test required in accordance with Appendix J, Art. III.A.1.d. However, leakage is not included in the  $0.60 L_a$  Types B and C test acceptance criteria.



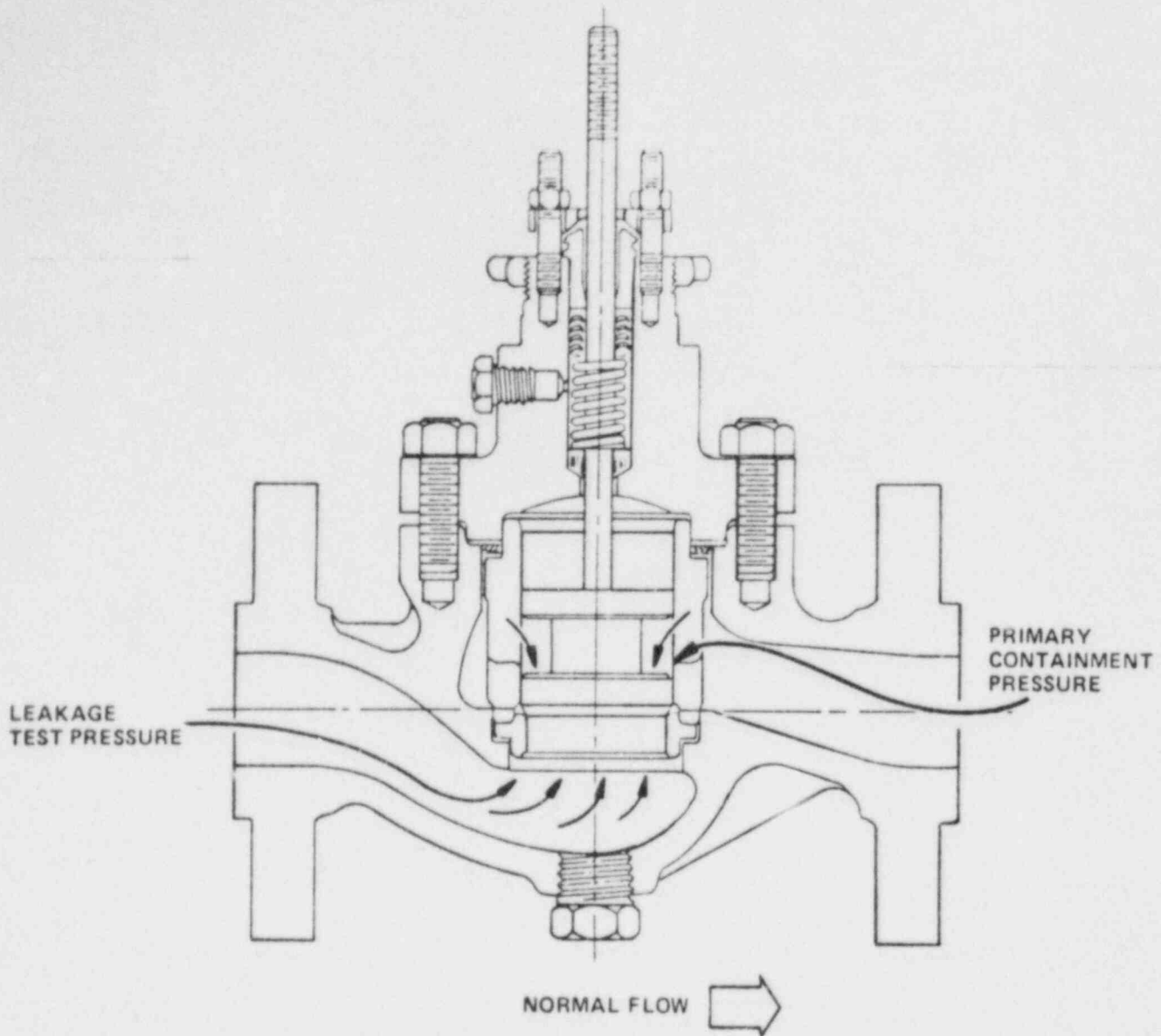
FORCES CAUSED BY THE APPLICATION OF LEAKAGE TEST PRESSURE UNDER THE VALVE DISC ACT AGAINST THE SEATING FORCE CREATED BY THE STEM ACTING ON THE DISC. FORCES DUE TO CONTAINMENT PRESSURE ACT ON TOP OF THE DISC AND ARE ADDITIVE TO THE SEATING FORCES OF THE STEM AGAINST THE DISC AND TEND TO SEAT THE VALVE MORE TIGHTLY.

Figure 11 Globe Valve: Applicable To Valves  
E21-F015A, B and T48-F339, 341



FORCES DUE TO THE LEAKAGE TEST PRESSURE ACT AGAINST THE SEATING FORCES OF THE VALVE. AS THE VALVE IS DESIGNED TO USE UPSTREAM PRESSURE TO PROVIDE A TIGHT SEAT, PRESSURE FORCES FROM THE CONTAINMENT DIRECTION WILL TEND TO SEAT THE VALVE.

Figure 12 Main Steam Isolation Globe Valve  
Applicable to Valves B21-F022 A,B,C,D



THE SUBJECT VALVES ARE OF THE UNBALANCED FLOW TO OPEN DESIGN; THEREFORE, WITH AN OBSERVED PRESSURE DROP IN THE REVERSE FLOW DIRECTION, AN ADDITIONAL SEATING LOAD WILL BE EXPERIENCED DUE TO THE HIGHER PRESSURE AT THE OUTLET OF THE VALVE BEING REGISTERED ON TOP OF THE VALVE PLUG, THUS SUPPLYING A FORCE IN THE DOWNWARD DIRECTION. THEREFORE, PRIMARY CONTAINMENT PRESSURE WILL TEND TO SEAT THE VALVE MORE TIGHTLY, WHEREAS TEST PRESSURE APPLIED ON THE SIDE OPPOSITE CONTAINMENT ACTS AGAINST THE SEATING FORCES.

Figure 13 Control Valve, Applicable to Valve P33-F004

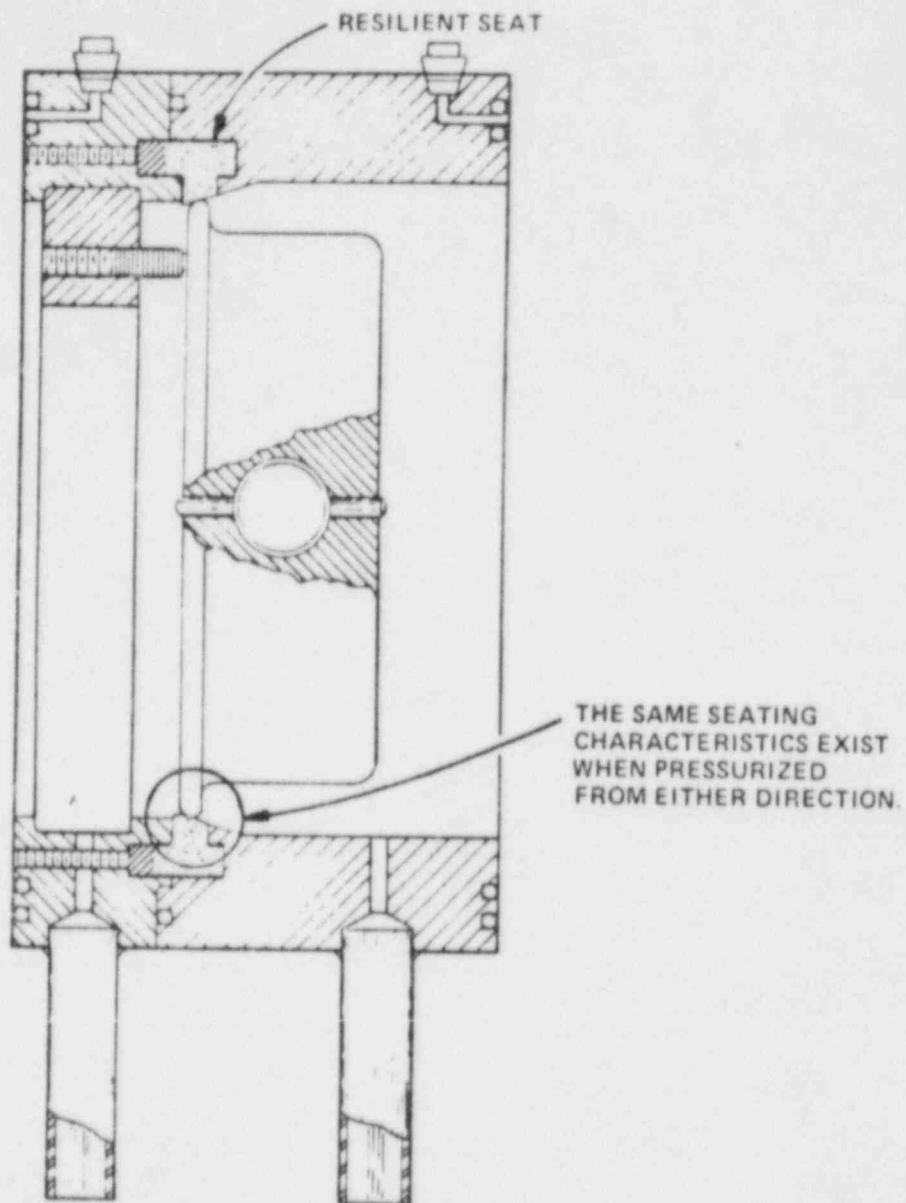


Figure 14 Butterfly Valve, Applicable to Valves T48-F307, F309, F310, F311 F318, and F319.

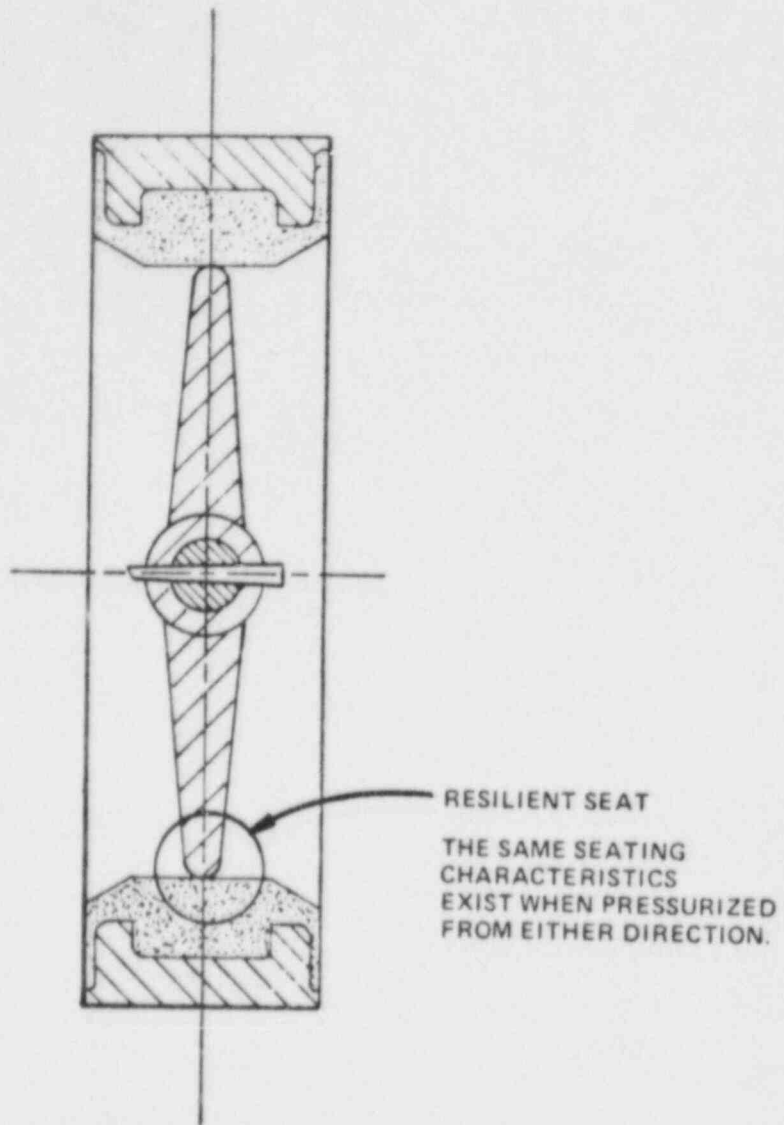


Figure 15 Butterfly Valve, Applicable To  
Valves E41-F051 and E51-F003



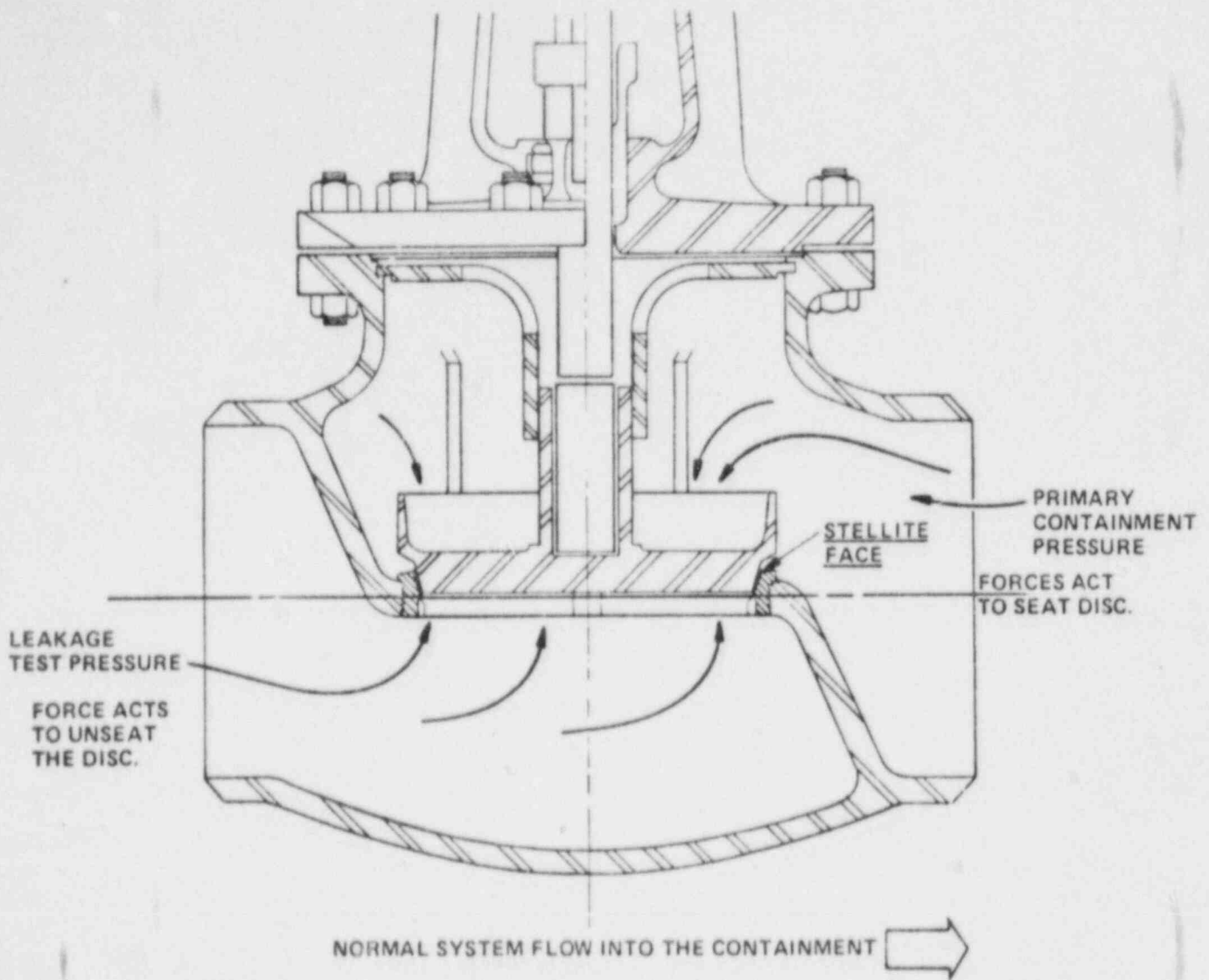
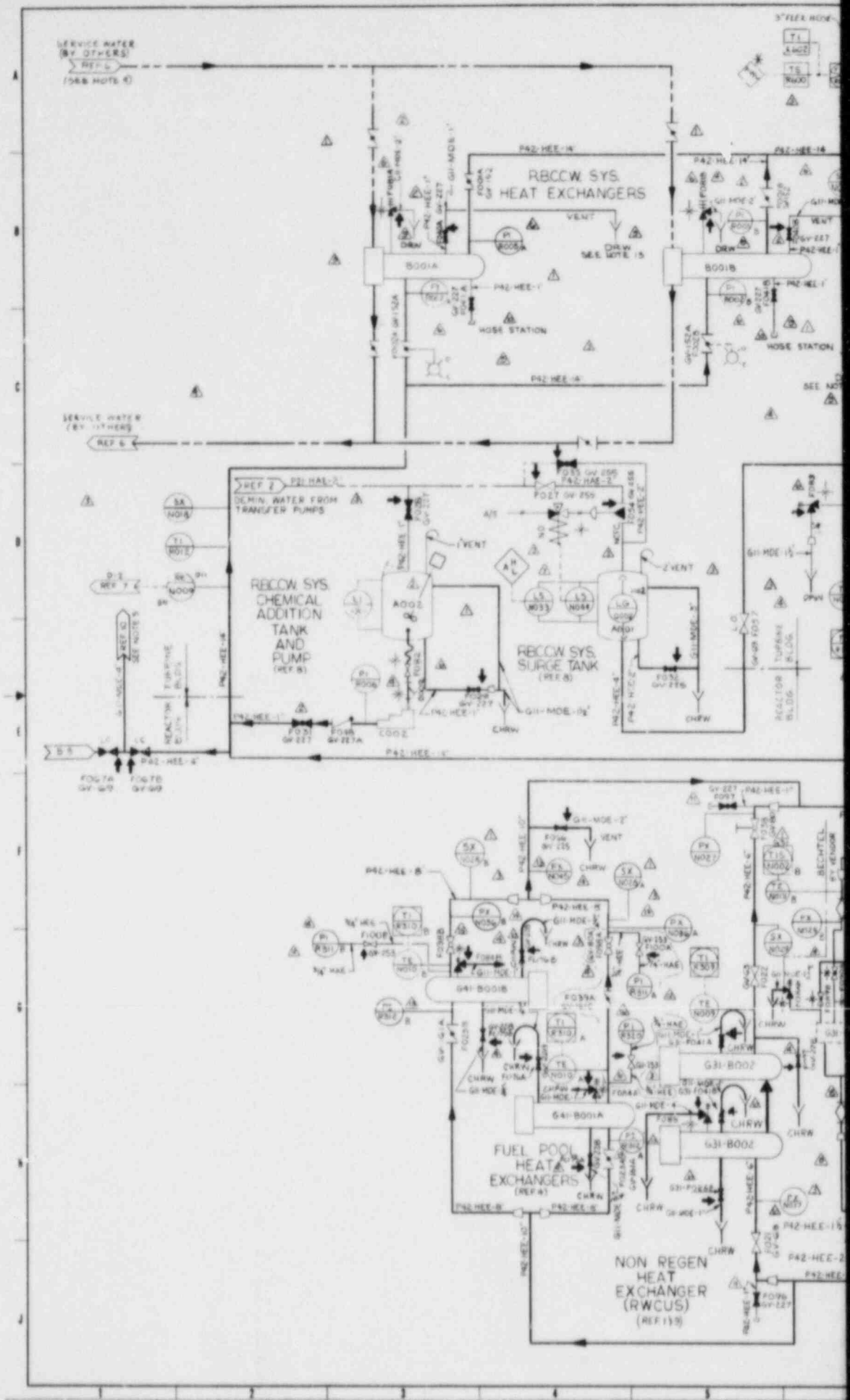


Figure 1b Stop Check Valve, Applicable To  
 Valves E51-F001, F002 and E41-F021, F022



ALL EQUIPMENT IS LOCATED INSIDE THE REACTOR BUILDING EXCEPT FOR THE RBCCW SYS HT EXCH. (ITEM NO. P42-BOO A,B) AND THE RECW SYS PUMPS (ITEM NO. P42-COOA,B).

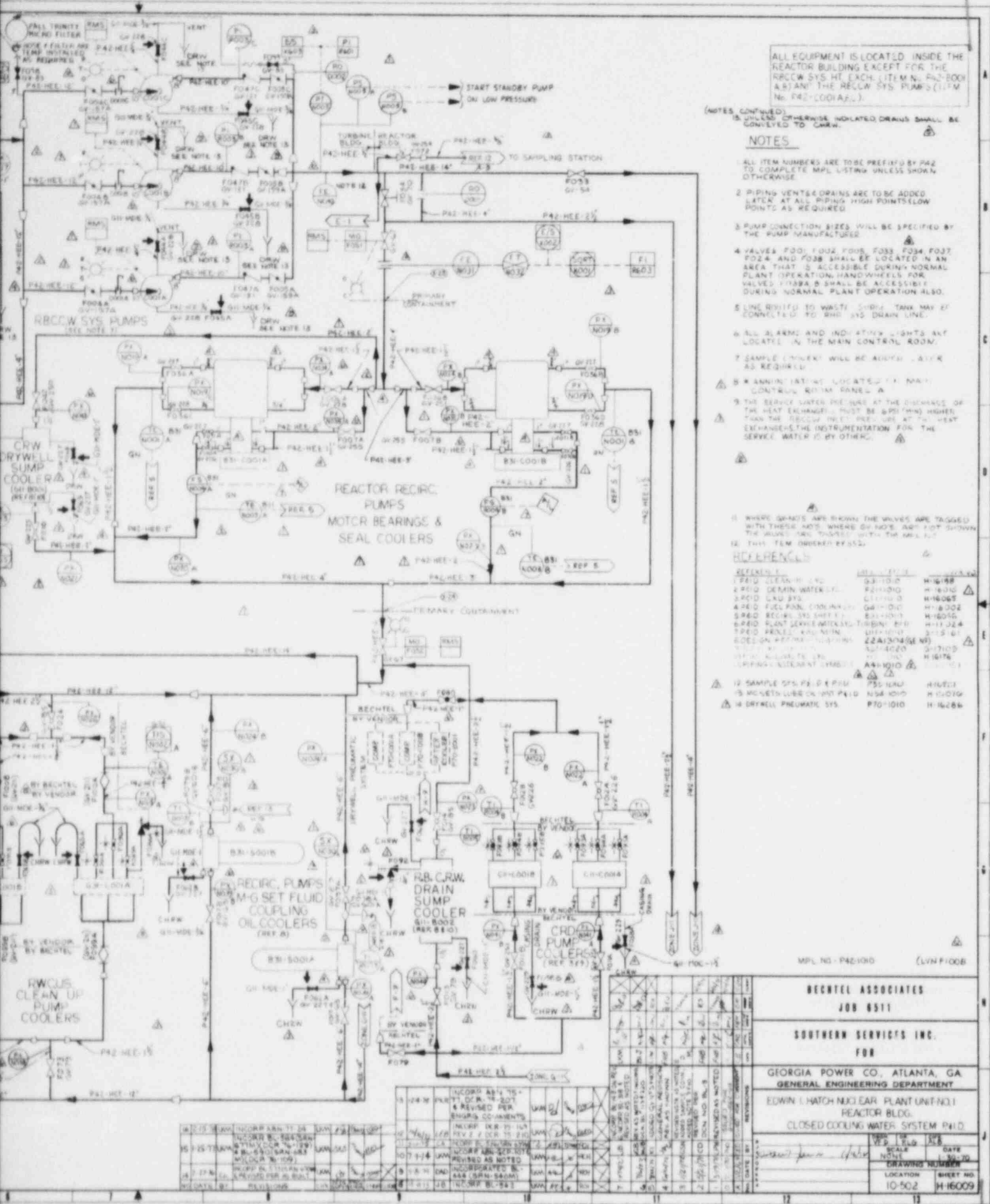
(NOTES CONTINUED)  
 15. UNLESS OTHERWISE INDICATED DRAINS SHALL BE CONVEYED TO CWRW.

**NOTES**

1. ALL ITEM NUMBERS ARE TO BE PREFIXED BY P42 TO COMPLETE MPL LISTING UNLESS SHOWN OTHERWISE.
2. PIPING VENT/S DRAINS ARE TO BE ADDED AT ALL HIGH POINTS AND LOW POINTS AS REQUIRED.
3. PUMP CONNECTION SIZES WILL BE SPECIFIED BY THE PUMP MANUFACTURER.
4. VALVES F001, F002, F003, F034, F037, F024 AND F028 SHALL BE LOCATED IN AN AREA THAT IS ACCESSIBLE DURING NORMAL PLANT OPERATION. HANDWHEELS FOR VALVES F034 & F028 SHALL BE ACCESSIBLE DURING NORMAL PLANT OPERATION ALSO.
5. LINE ROUTED TO WASTE SINK TANK MAY BE CONNECTED TO RND SYS DRAIN LINE.
6. ALL ALARMS AND INDICATING LIGHTS ARE LOCATED IN THE MAIN CONTROL ROOM.
7. SAMPLE POINTS WILL BE ADDED AS REQUIRED.
8. WARNING LIGHT LOCATED IN MAIN CONTROL ROOM PANEL A.
9. THE SERVICE WATER PRESSURE AT THE DISCHARGE OF THE HEAT EXCHANGER MUST BE 6 PSI (MIN) HIGHER THAN THE FLOW PRESSURE AT THE HEAT EXCHANGER. THE INSTRUMENTATION FOR THE SERVICE WATER IS BY OTHER.
10. WHERE DRAINS ARE SHOWN THE VALVES ARE TAGGED WITH THESE TAGS. WHERE DRAINS ARE NOT SHOWN THE VALVES ARE TAGGED WITH THE TAGS IN THIS ITEM CHECKED BY 552.

**REFERENCES**

REF. NO.	DESCRIPTION	DATE	REV.
1	FIELD CLEANING LOG	03-10-60	H-16188
2	FIELD DEMIN. WATER LOG	02-10-60	H-16010
3	FIELD OIL LOG	01-10-60	H-16065
4	FIELD FUEL OIL LOG	04-10-60	H-16002
5	FIELD RECORD SHEET	03-10-60	H-16015
6	FIELD PLANT SERVICE LOG	01-10-60	H-16024
7	FIELD PROJECT LOG	01-10-60	H-16011
8	FIELD ON-PUMP LOG	02-10-60	H-16012
9	FIELD ON-PUMP LOG	02-10-60	H-16013
10	FIELD ON-PUMP LOG	02-10-60	H-16014
11	FIELD ON-PUMP LOG	02-10-60	H-16015
12	SAMPLE SYS. P42-000	03-10-60	H-16021
13	MOBILE LOG ON P42-000	03-10-60	H-16022
14	DRYCELL PNEUMATIC SYS.	07-10-60	H-16286



NO.	DATE	BY	DESCRIPTION	APP.	REV.
1	10-27-59	WJW	ISSUED FOR CONSTRUCTION	WJW	1
2	11-10-59	WJW	REVISIONS	WJW	2
3	12-15-59	WJW	REVISIONS	WJW	3
4	01-20-60	WJW	REVISIONS	WJW	4
5	02-10-60	WJW	REVISIONS	WJW	5
6	03-10-60	WJW	REVISIONS	WJW	6
7	04-10-60	WJW	REVISIONS	WJW	7
8	05-10-60	WJW	REVISIONS	WJW	8
9	06-10-60	WJW	REVISIONS	WJW	9
10	07-10-60	WJW	REVISIONS	WJW	10
11	08-10-60	WJW	REVISIONS	WJW	11
12	09-10-60	WJW	REVISIONS	WJW	12
13	10-10-60	WJW	REVISIONS	WJW	13
14	11-10-60	WJW	REVISIONS	WJW	14
15	12-10-60	WJW	REVISIONS	WJW	15

MPL NO - P42-100 (LVN F100B)

**RECHTEL ASSOCIATED**  
 JOB 8511

**SOUTHERN SERVICES INC.**  
 FOR

**GEORGIA POWER CO., ATLANTA, GA.**  
 GENERAL ENGINEERING DEPARTMENT

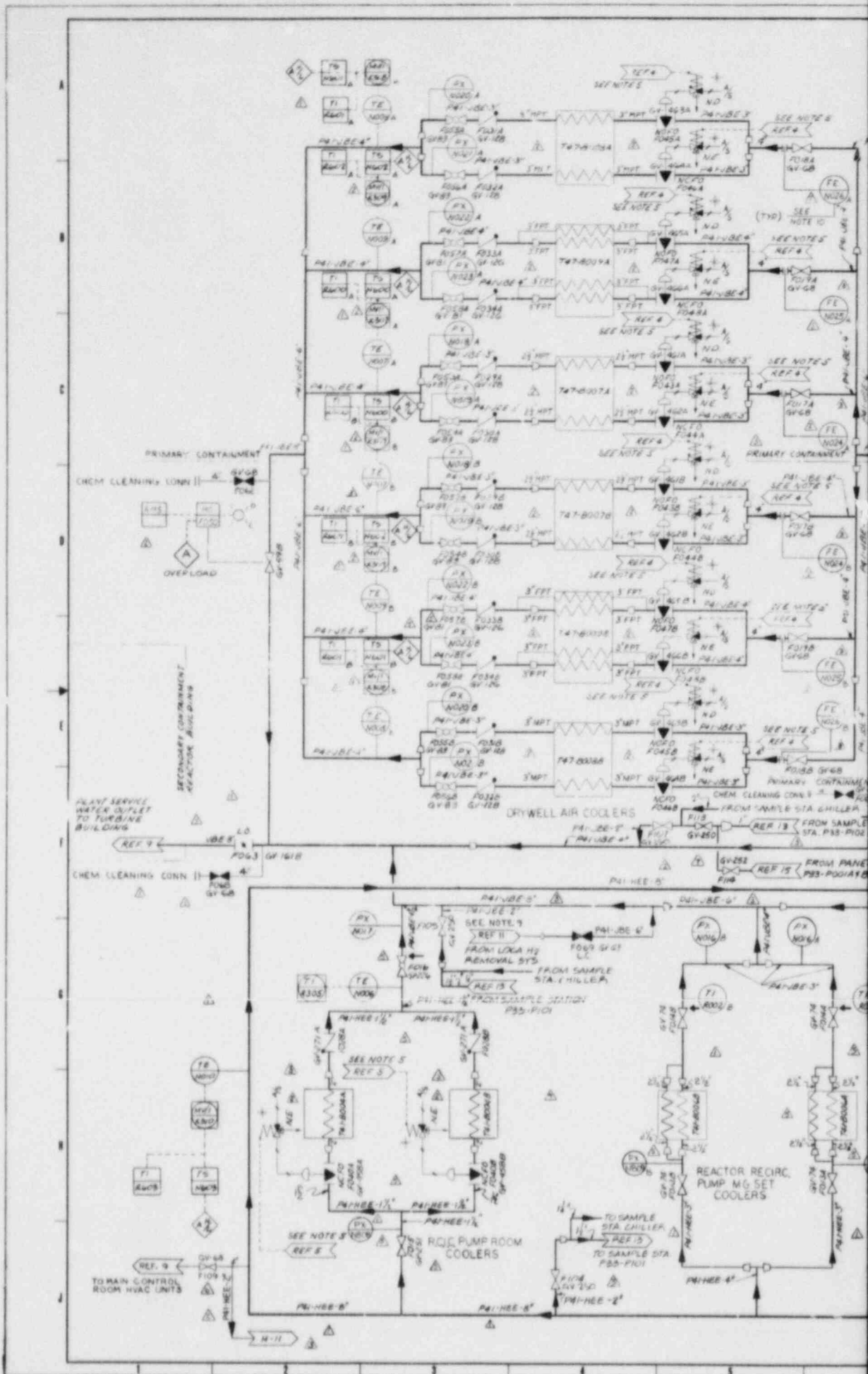
EDWIN HATCH NUCLEAR PLANT UNIT NO. 1  
 REACTOR BLDG.  
 CLOSED COOLING WATER SYSTEM P42.0

SCALE: 1" = 6'-0"

DATE: 10-27-59

DRAWING NUMBER: 10-502 H-16009

SHEET NO. 10 OF 12



ALL EQUIPMENT SHOWN IS LOCATED INSIDE THE REACTOR BUILDING

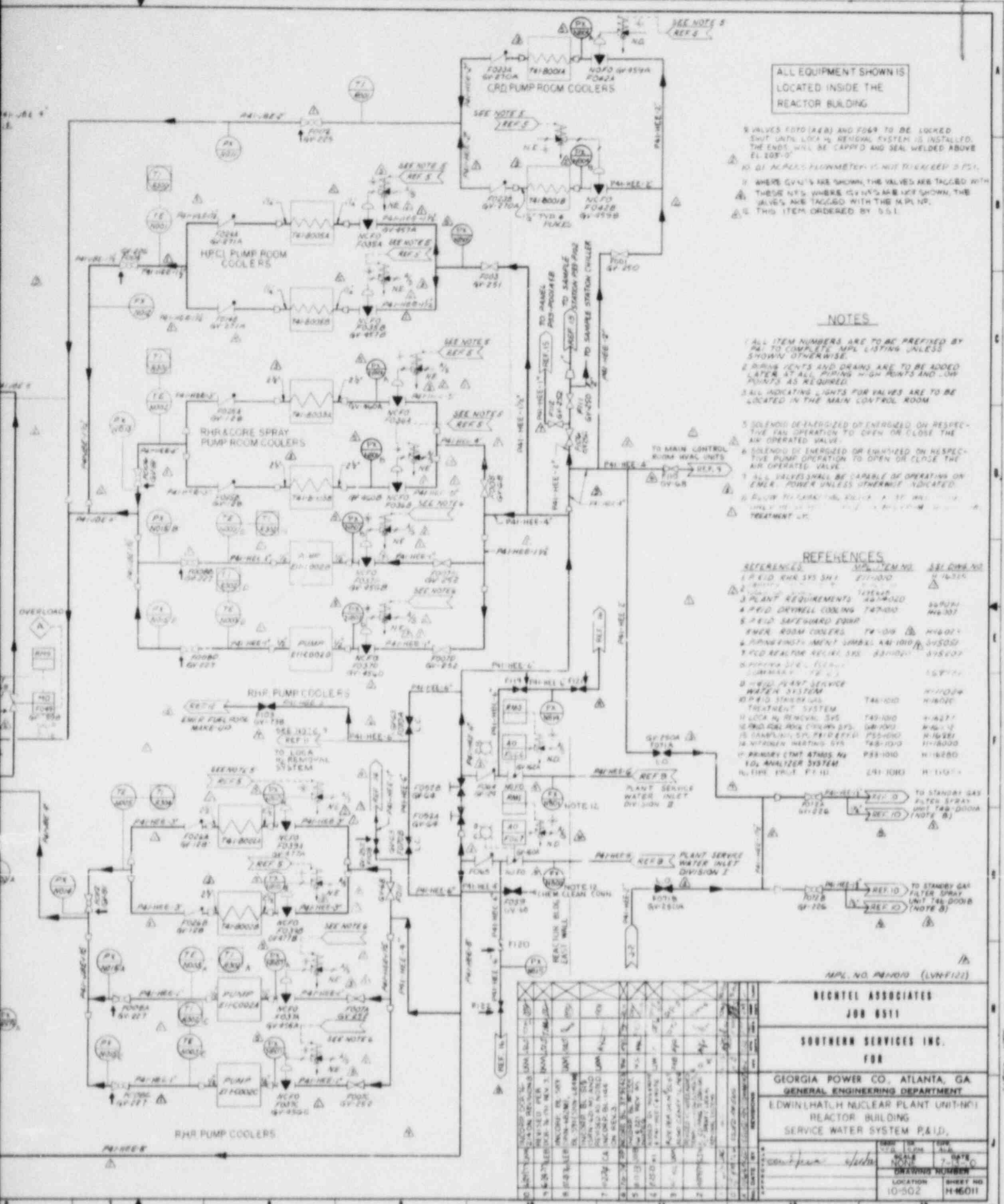
- VALVES F050 (A&B) AND F069 TO BE LOCKED SHUT UNTIL LOCA IS REMOVED FROM SYSTEM. THE ENDS WILL BE CAPPED AND SEAL WELDED ABOVE EL 200'-0".
- ON 21 ALL GAS FLOWMETERS TO NOT EXCEED 3 PSI.
- WHERE GVAL'S ARE SHOWN, THE VALVES ARE TAGGED WITH THESE NTS WHERE GVAL'S ARE NOT SHOWN, THE VALVES ARE TAGGED WITH THE M.P.L. NO.
- THIS ITEM ORDERED BY G.I.

NOTES

- ALL ITEM NUMBERS ARE TO BE PREFIXED BY RL TO COMPLETE MPL LISTING UNLESS OTHERWISE INDICATED.
- DRAIN POINTS AND DRAINS ARE TO BE ADDED LATER AT ALL PUMPING HIGH POINTS AND LOW POINTS AS REQUIRED.
- ALL INDICATING LIGHTS FOR VALVES ARE TO BE LOCATED IN THE MAIN CONTROL ROOM.
- SOLENOID DE-ENERGIZED OR ENERGIZED ON RESPECTIVE PUMP OPERATION TO OPEN OR CLOSE THE AIR OPERATED VALVE.
- SOLENOID DE-ENERGIZED OR ENERGIZED ON RESPECTIVE PUMP OPERATION TO OPEN OR CLOSE THE AIR OPERATED VALVE.
- ALL VALVES SHALL BE CAPABLE OF OPERATING ON EMERGENCY POWER UNLESS OTHERWISE INDICATED.
- FLOW TO STANDBY GAS FILTER SPRAY UNIT 746-0008 UNIT 746-0008A TREATMENT UNIT.

REFERENCES

REFERENCE	SYMBOL	REV. NO.	DATE
1. P.O. RMR 575 542	277-1000		7-2-52
2. P.O. RMR 575 542	277-1000		7-2-52
3. PLANT REQUIREMENTS	442-0000		5-20-51
4. P.O. DRYWELL COOLING	747-1000		4-6-52
5. P.O. SAFEGUARD ZONE			
6. P.O. ROOM COOLERS	747-1000		4-6-52
7. P.O. SAFEGUARD ZONE			
8. P.O. SAFEGUARD ZONE			
9. P.O. SAFEGUARD ZONE			
10. P.O. SAFEGUARD ZONE			
11. P.O. SAFEGUARD ZONE			
12. P.O. SAFEGUARD ZONE			
13. P.O. SAFEGUARD ZONE			
14. P.O. SAFEGUARD ZONE			
15. P.O. SAFEGUARD ZONE			
16. P.O. SAFEGUARD ZONE			
17. P.O. SAFEGUARD ZONE			
18. P.O. SAFEGUARD ZONE			
19. P.O. SAFEGUARD ZONE			
20. P.O. SAFEGUARD ZONE			



NO.	DATE	BY	CHKD.	APP'D.	REVISION
1	10-5-52	J. H. ...			...
2	10-10-52	J. H. ...			...
3	10-15-52	J. H. ...			...
4	10-20-52	J. H. ...			...
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8	11-10-52	J. H. ...			...
9	11-15-52	J. H. ...			...
10	11-20-52	J. H. ...			...
11	11-25-52	J. H. ...			...
12	11-30-52	J. H. ...			...
13	12-5-52	J. H. ...			...
14	12-10-52	J. H. ...			...
15	12-15-52	J. H. ...			...
16	12-20-52	J. H. ...			...
17	12-25-52	J. H. ...			...
18	12-30-52	J. H. ...			...
19	1-5-53	J. H. ...			...
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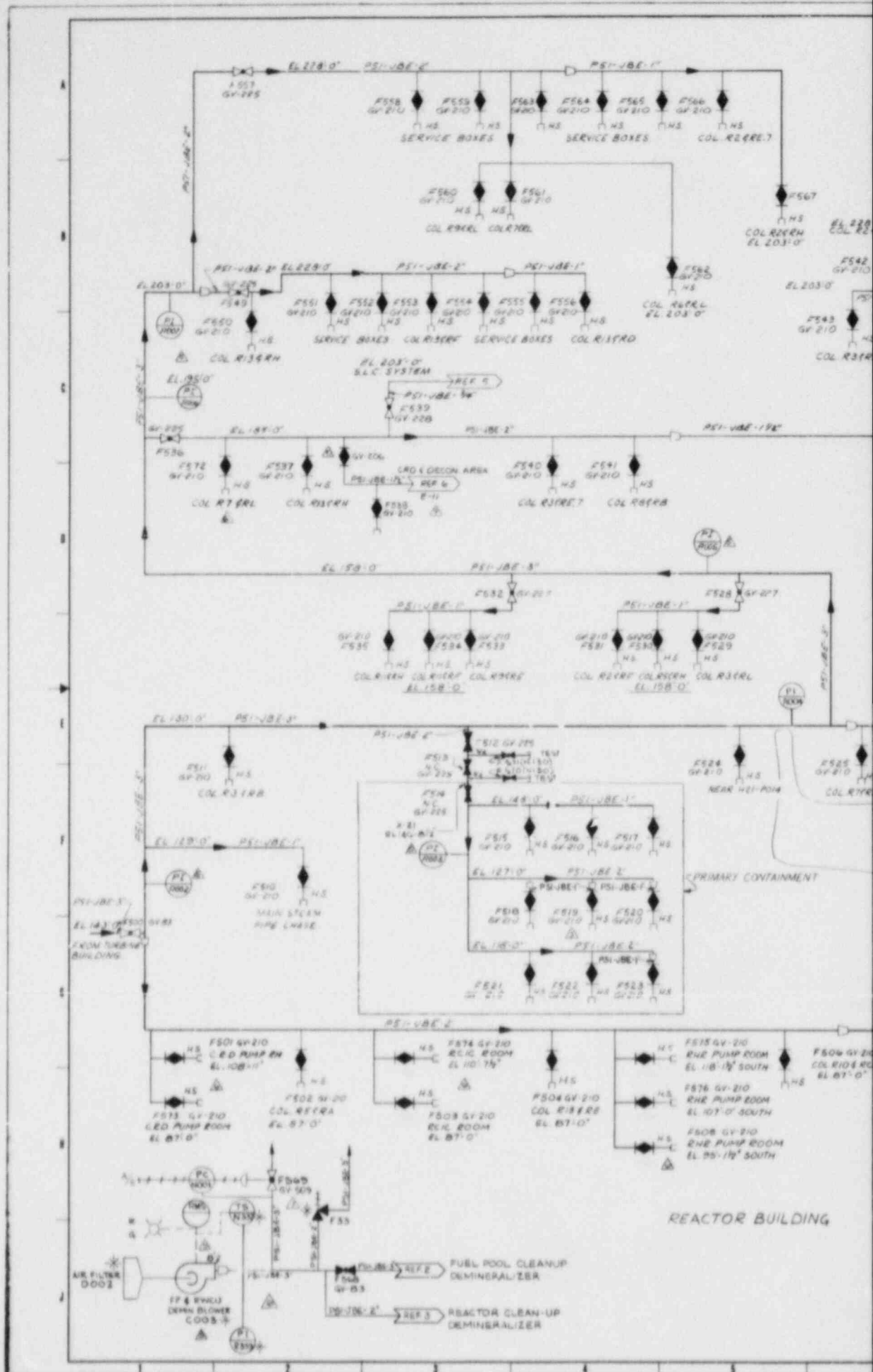
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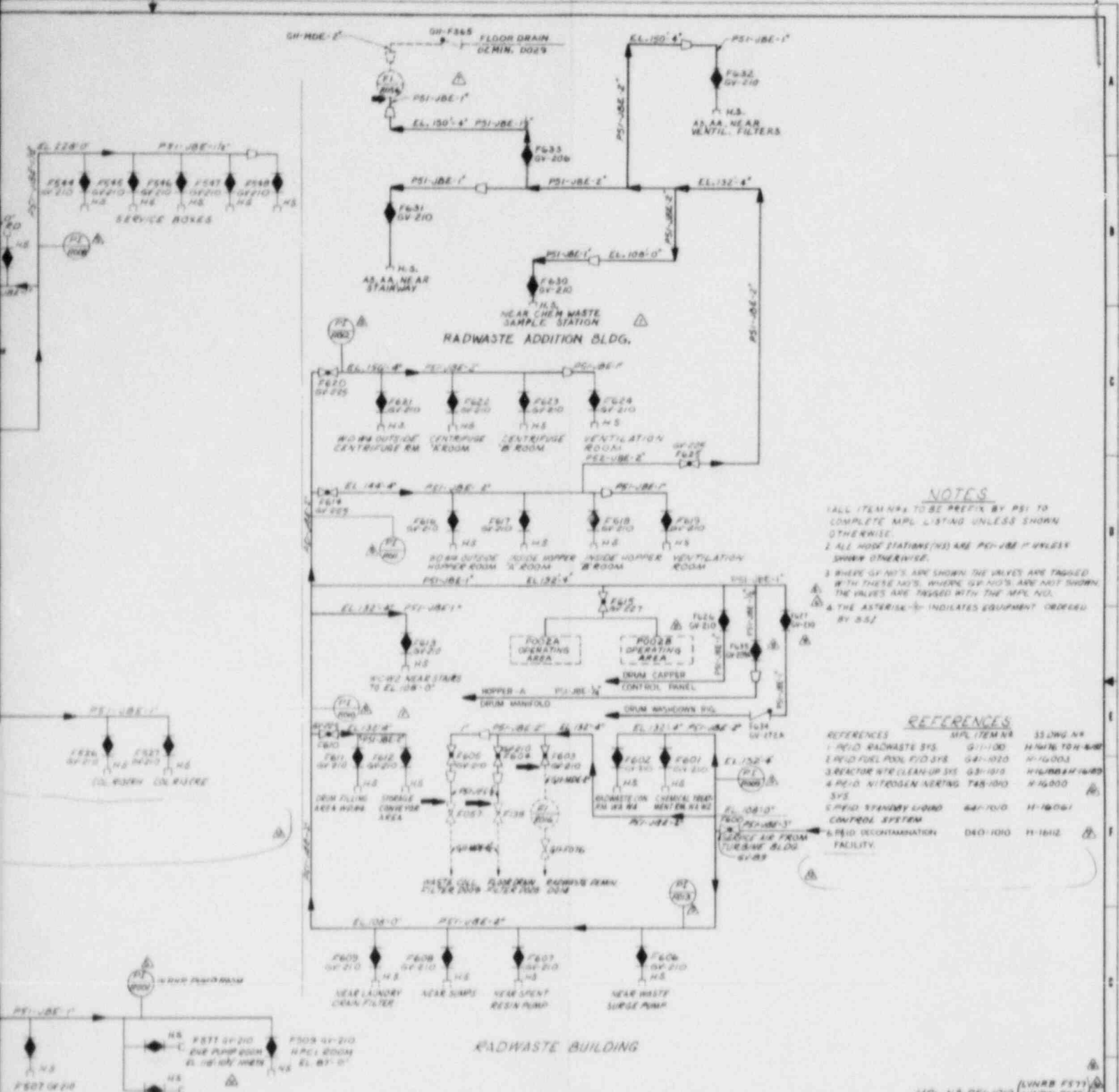
BECHTEL ASSOCIATES  
JOB 6511

SOUTHERN SERVICES INC.  
FOR

GEORGIA POWER CO., ATLANTA, GA  
GENERAL ENGINEERING DEPARTMENT  
EDWIN HATHI NUCLEAR PLANT UNIT-N01  
REACTOR BUILDING  
SERVICE WATER SYSTEM P&ID

DATE	10-5-52
SCALE	AS SHOWN
DRAWING NUMBER	10-502
SHEET NO.	H-46011





- NOTES**
1. ALL ITEM NOS TO BE PREFIX BY PSI TO COMPLETE MPL LISTING UNLESS SHOWN OTHERWISE.
  2. ALL NODE STATIONS (NS) ARE PSI-JBE-1 UNLESS SHOWN OTHERWISE.
  3. WHERE GF NO'S ARE SHOWN THE DRIVES ARE TAGGED WITH THESE NO'S. WHERE GF NO'S ARE NOT SHOWN THE VALVES ARE TAGGED WITH THE MPL NO.
  4. THE ASTERISK (\*) INDICATES EQUIPMENT ORDERED BY 552.

- REFERENCES**
- | REFERENCES                             | MPL ITEM NR | SS DWG NR      |
|----------------------------------------|-------------|----------------|
| 1. RADWASTE SYS.                       | 01-100      | H-1616 TOR-600 |
| 2. RAD FUEL POOL FID SYS               | 04-1020     | H-16003        |
| 3. REACTOR WTR CLEAN-UP SYS            | 03-1010     | H-16004/1000   |
| 4. RAD NITROGEN HEATING SYS            | 748-1010    | H-16000        |
| 5. SPENT SPENDRY G-DRUM CONTROL SYSTEM | 642-1010    | H-16061        |
| 6. RAD DECONTAMINATION FACILITY        | 040-1010    | H-16112        |

NO.	REVISION	DATE	BY	CHKD	APP'D	DESCRIPTION
1						ISSUED FOR CONSTRUCTION
2						REVISIONS AS NOTED
3						REVISIONS AS NOTED
4						REVISIONS AS NOTED
5						REVISIONS AS NOTED
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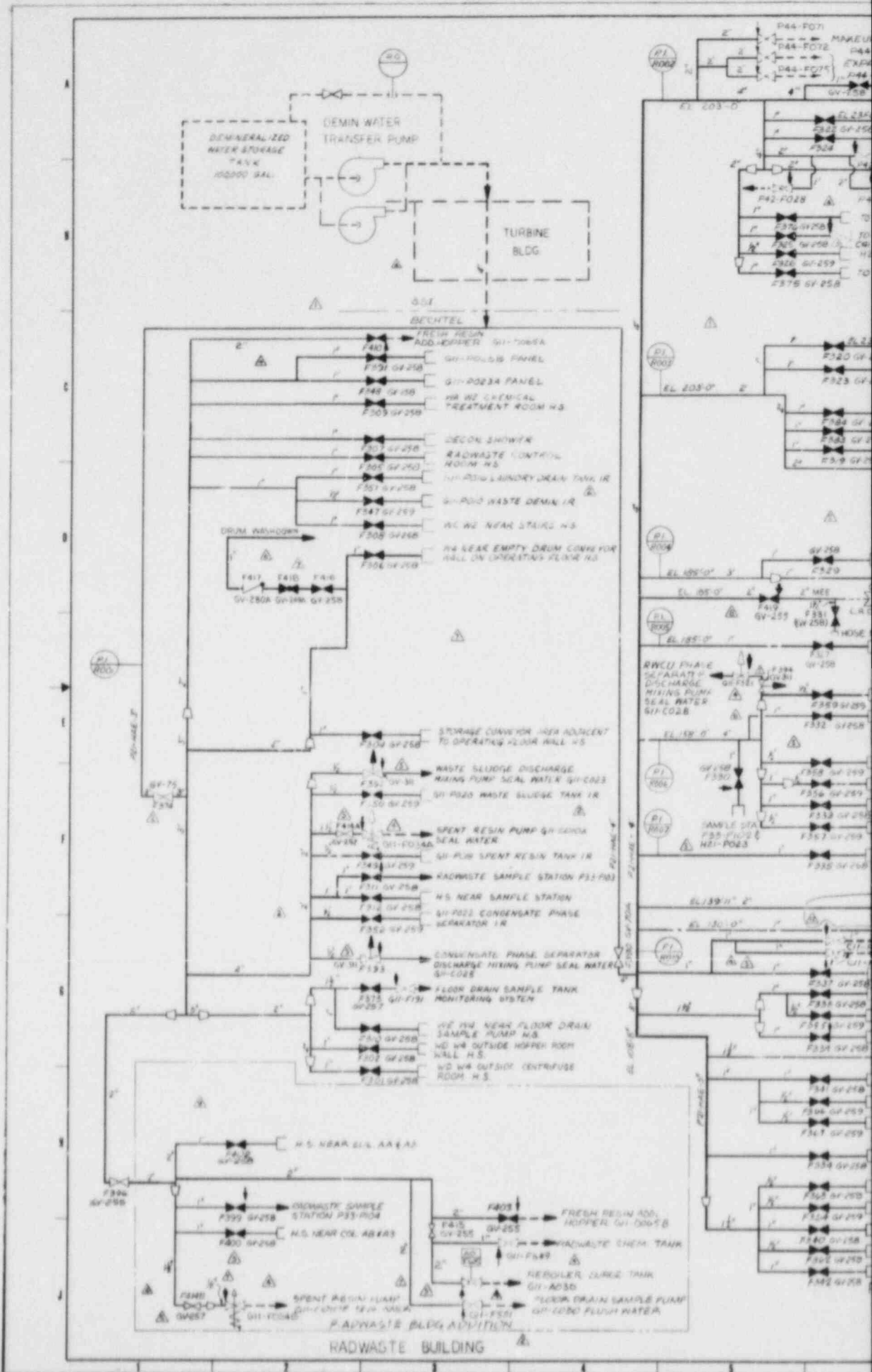
MPL NR 021-1010 (LVNR F577)  
 (LVNR F635)

**BECHTEL ASSOCIATES**  
 JOB 8511

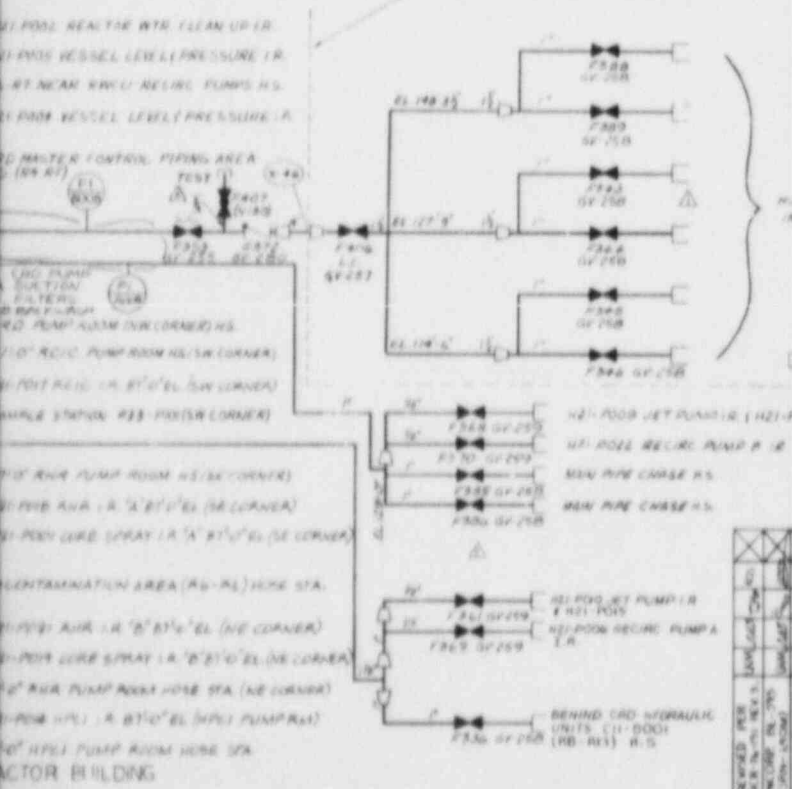
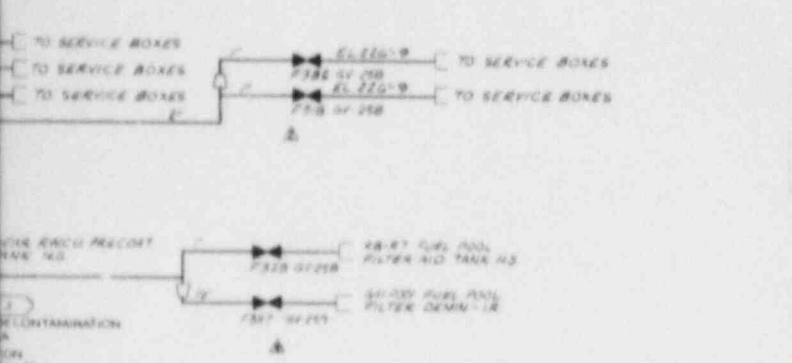
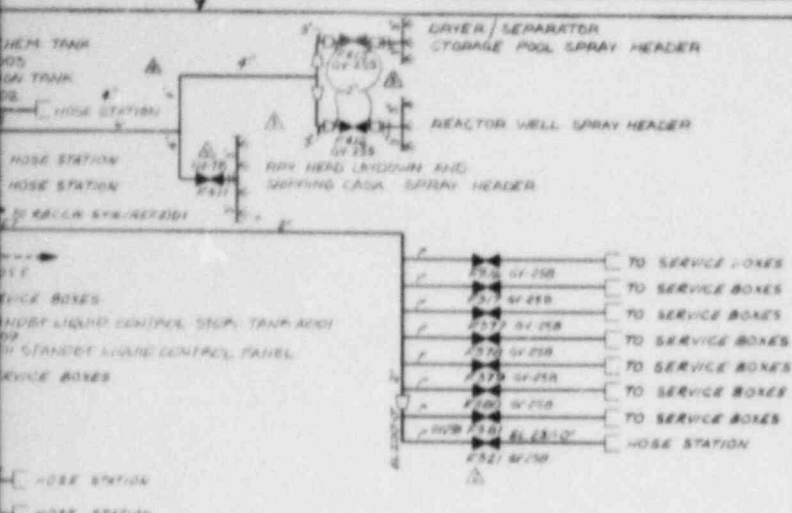
**SOUTHERN SERVICES INC.**  
 FOR

GEORGIA POWER CO., ATLANTA, GA  
 GENERAL ENGINEERING DEPARTMENT  
 EDWIN LATCH NUCLEAR PLANT UNIT NO.1  
 REACTOR & RADWASTE BUILDINGS  
 SERVICE AIR SYSTEM DIAG.

DATE: 11/15/61  
 DRAWING NUMBER: 10-502  
 SHEET NO: H-16013







- NOTES**
1. NS - HOSE STATION
  2. IRI - INSTRUMENT RACK
  3. UNLESS OTHERWISE NOTED ALL PIPE IN THIS SYSTEM IS NAC
  4. ALL EQUIPMENT AND INSTRUMENTS ON THIS DRAWING PRECEDED BY MFL NR P21 UNLESS OTHERWISE NOTED.
  5. LOCATIONS OF HIGHPOINT VENTS AND LOWPOINT DRAINS TO BE DETERMINED BY PHYSICAL PIPING ARRANGEMENT.
  6. MOMENTARY PUSH BUTTON SWITCH LOCATED IN CONTROL SYSTEM OPERATING AISLE NEAR PRIME PRE-CLOSE POSITION
  7. WHERE 1/2\"/>

**REFERENCES**

REFERENCE	NO. / TITLE	DATE
1	STANDBY LIQUID CONTROL SYS	11-16-61
2	REACTOR BLDG CLOSED COOLING WATER SYSTEM	11-16-61
3	DECONTAMINATION FACILITY	11-16-61

NO.	REVISION	DATE	BY	CHKD	APP'D	REVISION
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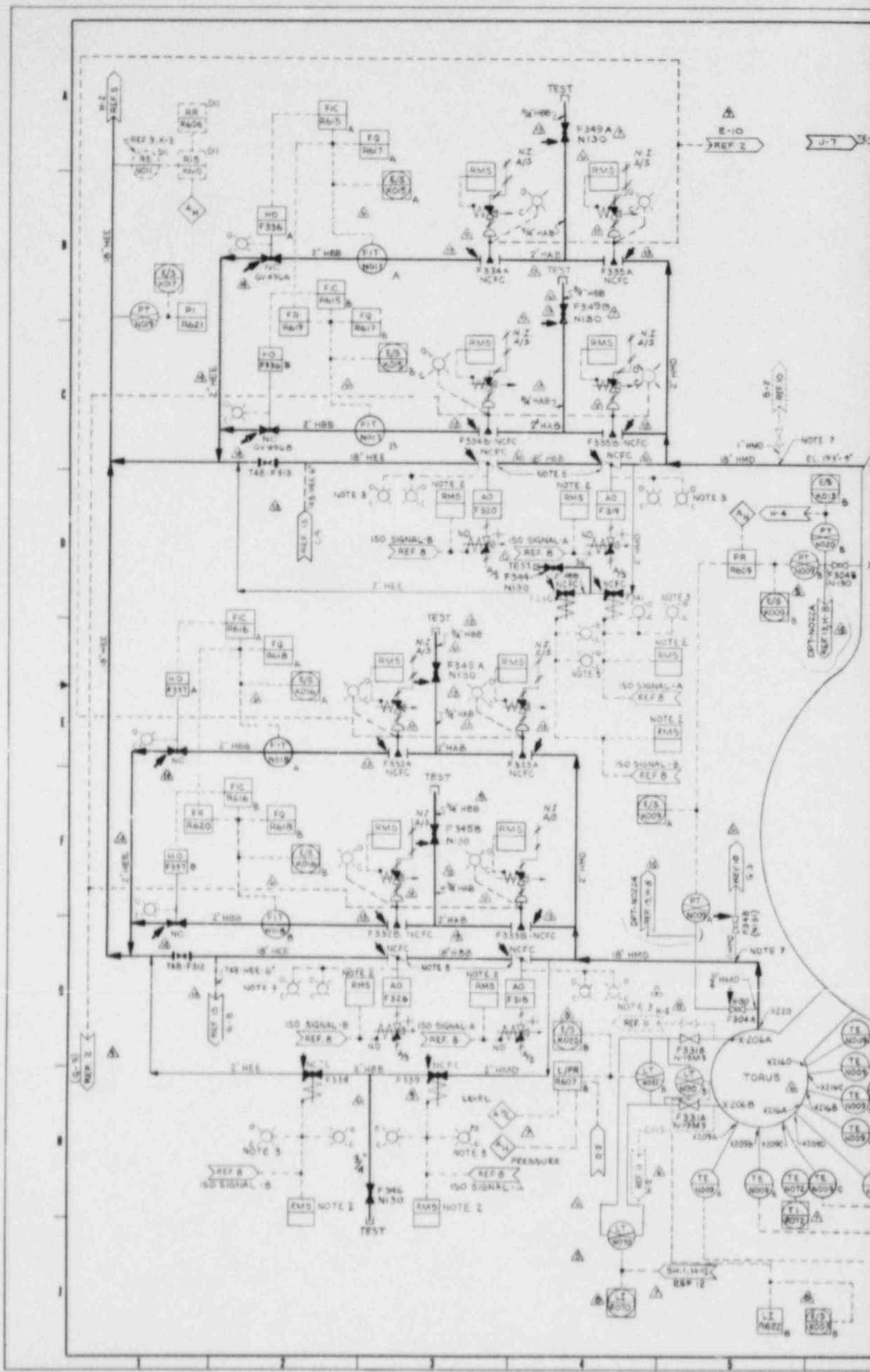
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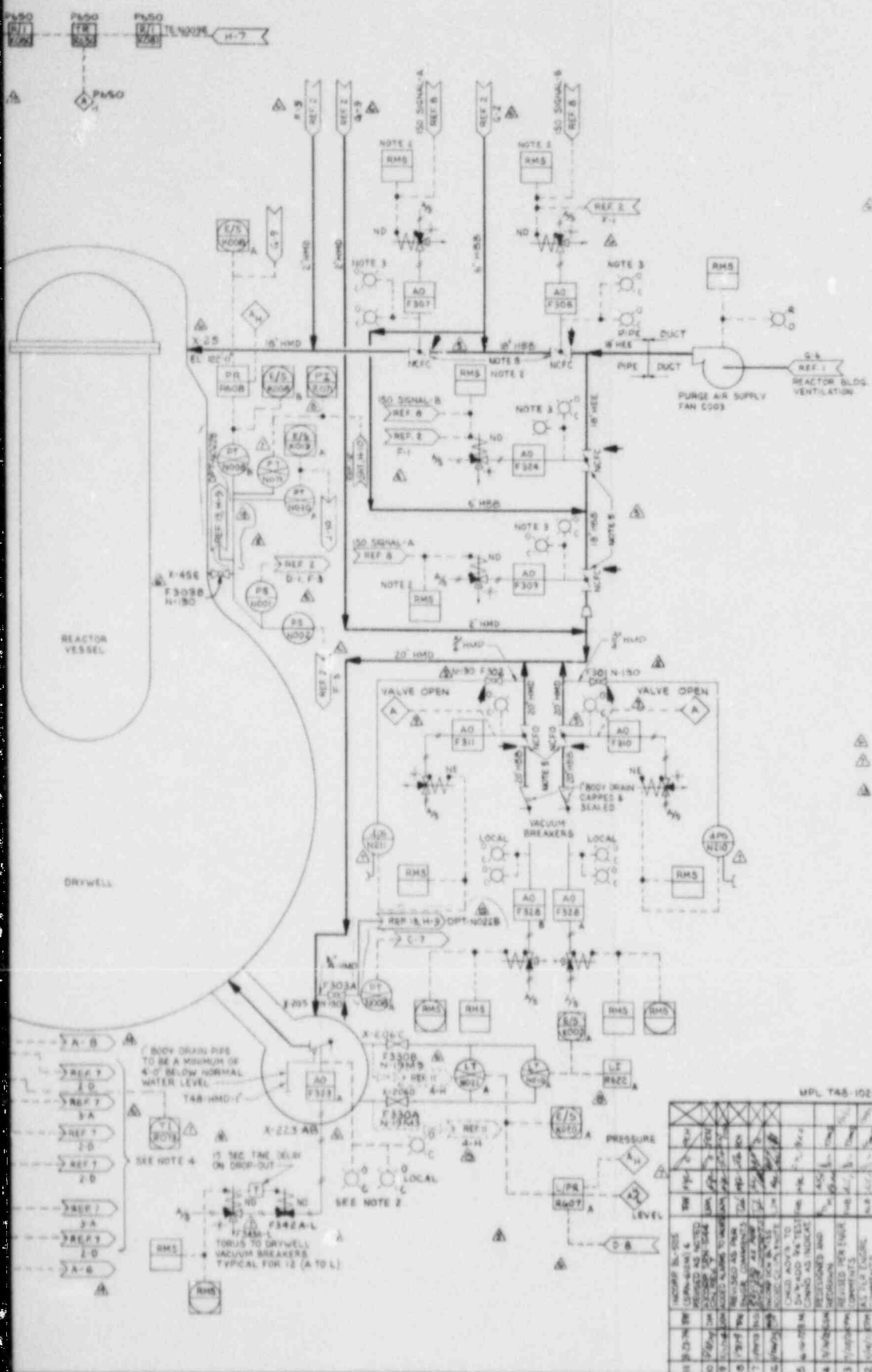
**BEUTEL ASSOCIATES**  
JOB 8311

**SOUTHERN SERVICES INC.**  
FOR

GEORGIA POWER CO., ATLANTA, GA  
GENERAL ENGINEERING DEPARTMENT  
EDWIN L HATCH, NUCLEAR PLANT UNIT#1  
REACTOR BUILDING DEMINERALIZED  
WATER SYSTEM DIAGRAM & P&ID.

John Taylor  
DATE: 4/28/70  
DRAWING NUMBER: 11-502  
SHEET NO: 11-502





**NOTES**

1. EQUIPMENT AND INSTRUMENT NOS. ON THIS DWG. PRECEDED BY T48, EXAMPLE: T48-1003 EXCEPT AS INDICATED. SEE NOTES 2 & 3.
2. SWITCHES AND LIGHTS INDICATED BY THIS NOTE ARE PART OF THE P.C.I.S. SYSTEM (PRIMARY CONTAINMENT ISOLATION SYSTEM) MPL 641 AND ARE MOUNTED ON BENCHBOARD HII-P601 & HII-P602.
3. INDICATING LIGHTS LOCATED ON GRAPHIC DISPLAY ON BENCHBOARD HII-P601.
4. TEMPERATURE ELEMENTS ARE INPUT TO A RECORDER SHOWN ON REF. 7.
5. 1" LEAKOFF CONNECTION ON VALVE TO BE USED FOR LOCAL LEAK RATE TESTING.
6. WHERE GW'S ARE SHOWN THE VALVES ARE TAGGED WITH THESE NOS. WHERE GW'S ARE NOT SHOWN, THE VALVES ARE TAGGED WITH THE MPL NO.
7. SAMPLE POINT CONNECTION ON VENT PURGE OUTLET LINE IS TO BE LOCATED AS CLOSE TO THE TORUS AND DRYWELL AS POSSIBLE, PROBABLY AT THE BOTTOM OF THE 20" ID REDUCER ON THE DRYWELL SIDE OF THE LINE.

**REFERENCES**

REFERENCE	MPL NO.	S.S. NO.
1. REACTOR BUILDING VENTILATION FLOW DIAG.	T41-1040	H-16006
2. NITROGEN INERTING SYSTEM P&ID.	T48-1010	H-16000
3. P&ID & INSTRUMENT SYMBOLS	A41-1010QA	S-15051
4. 3 HR SYS PAID	E11-1010	H-16529
5. STANDBY GAS TREATMENT SYSTEM	T46-1000	H-16020
6. REACTOR BUILDING VENTILATION P&ID	T41-1020	H-16005
7. DRYWELL COOLING SYSTEM P&ID	T47-1010	H-16007
8. PRIMARY CONTAINMENT ISOLATION SYSTEM	CM-1040	H-17801-805
9. PROCESS RADIATION MONITORING SYSTEM	PH-1010	S-17790
10. H2 ANAL. SYS. P&ID.	P33-1010	H-16276
11. WPC1 SYS. P&ID.	EX-1000	H-16332
12. ECLIC SYS. P&ID.	E51-1010	H-16234
13. DRYWELL TO TORUS DIFFERENTIAL PRESSURE SYSTEM P&ID.	T48-1050	H-16158

NO.	DATE	BY	DESCRIPTION	APPROVED
16	11/17/77	WJM	INCOMP. BL-6 (STERN-102C) (DGR-77-10)	
14	11/10/77	WJM	INCOMP. TELECON DATED 11/9 (DGR 74-110)	
13	11/10/77	WJM	ADDED TORUS TO DRYWELL OF SYS ISOLATION VALVES (RELOCATED 181" BYPASS) (CAD SYS. TR. AND REVISED PER ENGINEER COMMENTS)	
12	10/30/77	WJM	INCOMP. TRN-5 (REV. B) (DGR-77-10)	
11	10/27/77	WJM	REVISIONS	

MPL T48-1020

NO.	DATE	BY	DESCRIPTION	APPROVED
10	10/27/77	WJM	INCOMP. BL-6 (STERN-102C) (DGR-77-10)	
9	10/27/77	WJM	INCOMP. TELECON DATED 10/27/77	
8	10/27/77	WJM	ADDED TORUS TO DRYWELL OF SYS ISOLATION VALVES (RELOCATED 181" BYPASS) (CAD SYS. TR. AND REVISED PER ENGINEER COMMENTS)	
7	10/27/77	WJM	INCOMP. TRN-5 (REV. B) (DGR-77-10)	
6	10/27/77	WJM	REVISIONS	
5	10/27/77	WJM	INCOMP. TRN-5 (REV. B) (DGR-77-10)	
4	10/27/77	WJM	INCOMP. TRN-5 (REV. B) (DGR-77-10)	
3	10/27/77	WJM	INCOMP. TRN-5 (REV. B) (DGR-77-10)	
2	10/27/77	WJM	INCOMP. TRN-5 (REV. B) (DGR-77-10)	
1	10/27/77	WJM	INCOMP. TRN-5 (REV. B) (DGR-77-10)	

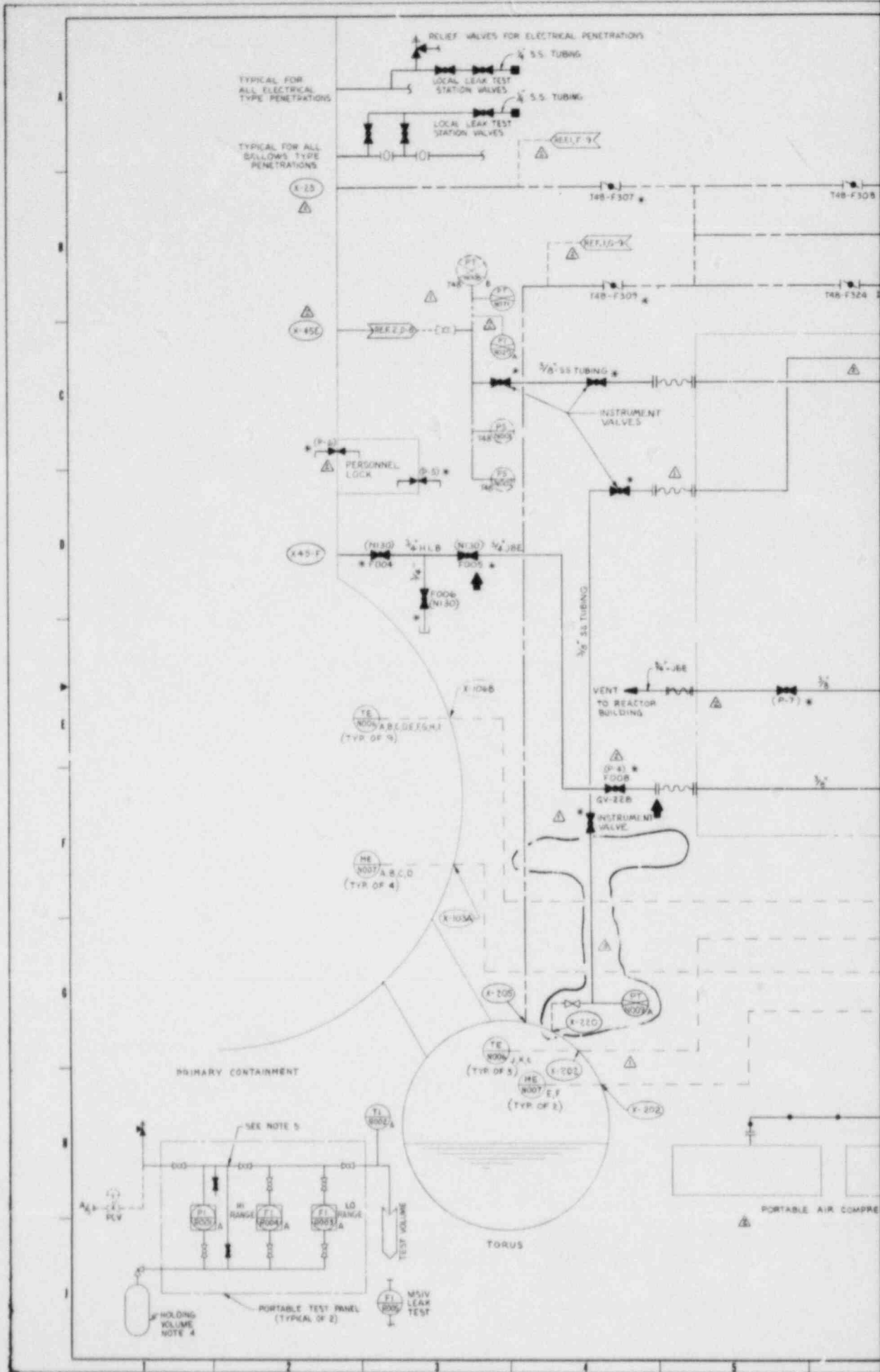
**BECHTEL ASSOCIATES**  
JOB 0511

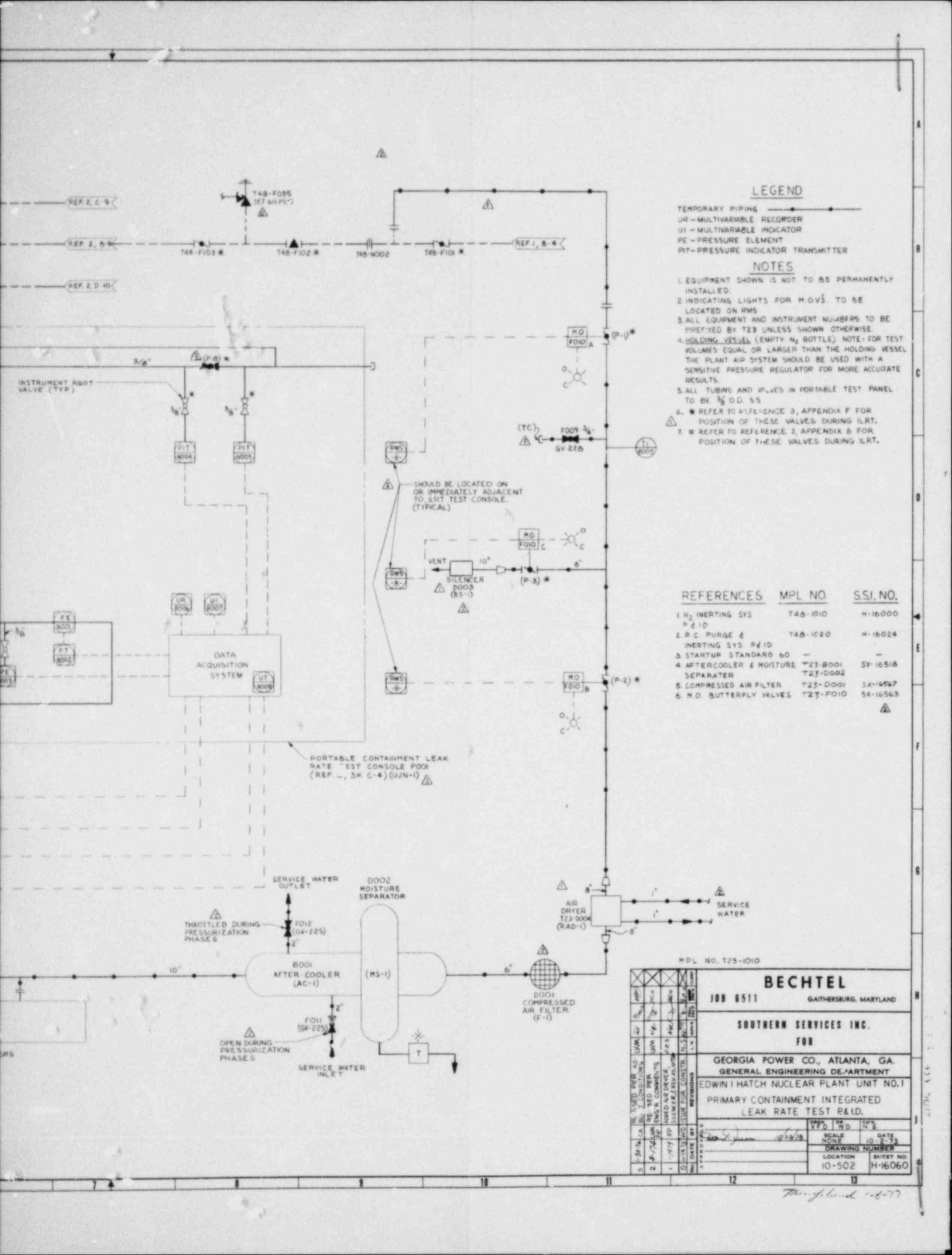
**SOUTHERN SERVICES INC.**  
FOR

**GEORGIA POWER CO., ATLANTA, GA.**  
GENERAL ENGINEERING DEPARTMENT  
EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1  
PRIMARY CONTAINMENT PURGE & INERTING SYSTEM P&ID

DATE: 11/17/77	SCALE: AS SHOWN	DATE: 11/17/77
DRAWING NUMBER		
LOCATION: 10-502	SHEET NO: H-6024	

LITING NEG.





**LEGEND**

- TEMPORARY PIPING
- UR - MULTIVARIABLE RECORDER
- UI - MULTIVARIABLE INDICATOR
- PE - PRESSURE ELEMENT
- PIT - PRESSURE INDICATOR TRANSMITTER

**NOTES**

1. EQUIPMENT SHOWN IS NOT TO BE PERMANENTLY INSTALLED.
2. INDICATING LIGHTS FOR MOV'S TO BE LOCATED ON RMS.
3. ALL EQUIPMENT AND INSTRUMENT NUMBERS TO BE PREFIXED BY T23 UNLESS SHOWN OTHERWISE.
4. HOLDING VESSEL (EMPTY N<sub>2</sub> BOTTLE). NOTE: FOR TEST VOLUMES EQUAL OR LARGER THAN THE HOLDING VESSEL THE PLANT AIR SYSTEM SHOULD BE USED WITH A SENSITIVE PRESSURE REGULATOR FOR MORE ACCURATE RESULTS.
5. ALL TUBING AND VALVES IN PORTABLE TEST PANEL TO BE 3/8" O.D. SS.
6. \* REFER TO REFERENCE 3, APPENDIX F FOR POSITION OF THESE VALVES DURING ILRT.
7. \* REFER TO REFERENCE 3, APPENDIX B FOR POSITION OF THESE VALVES DURING ILRT.

REFERENCES	MPL NO	SSI. NO.
1. N <sub>2</sub> INERTING SYS P&ID	T48-1010	H-16000
2. P.C. PURGE & INERTING SYS P&ID	T48-1020	H-16024
3. STARTUP STANDARD 60	-	-
4. AFTERCOOLER & MOISTURE SEPARATOR	T23-B001 T23-0002	5X-16518
5. COMPRESSED AIR FILTER	T23-D001	5X-16567
6. M.O. BUTTERFLY VALVES	T23-FO10	5X-16563

MPL NO. T23-1010

**BECHTEL**

JOB 6511 GAITHERSBURG, MARYLAND

**SOUTHERN SERVICES INC.**  
FOR

GEORGIA POWER CO., ATLANTA, GA.  
GENERAL ENGINEERING DEPARTMENT  
EDWIN HATCH NUCLEAR PLANT UNIT NO. 1  
PRIMARY CONTAINMENT INTEGRATED  
LEAK RATE TEST R&I.

DESIGNED BY	W. J. GIBSON
CHECKED BY	J. W. GIBSON
DATE	10/2/73
SCALE	AS SHOWN
LOCATION	10-502
SHEET NO.	H-16060

*Thompson 10-2-73*

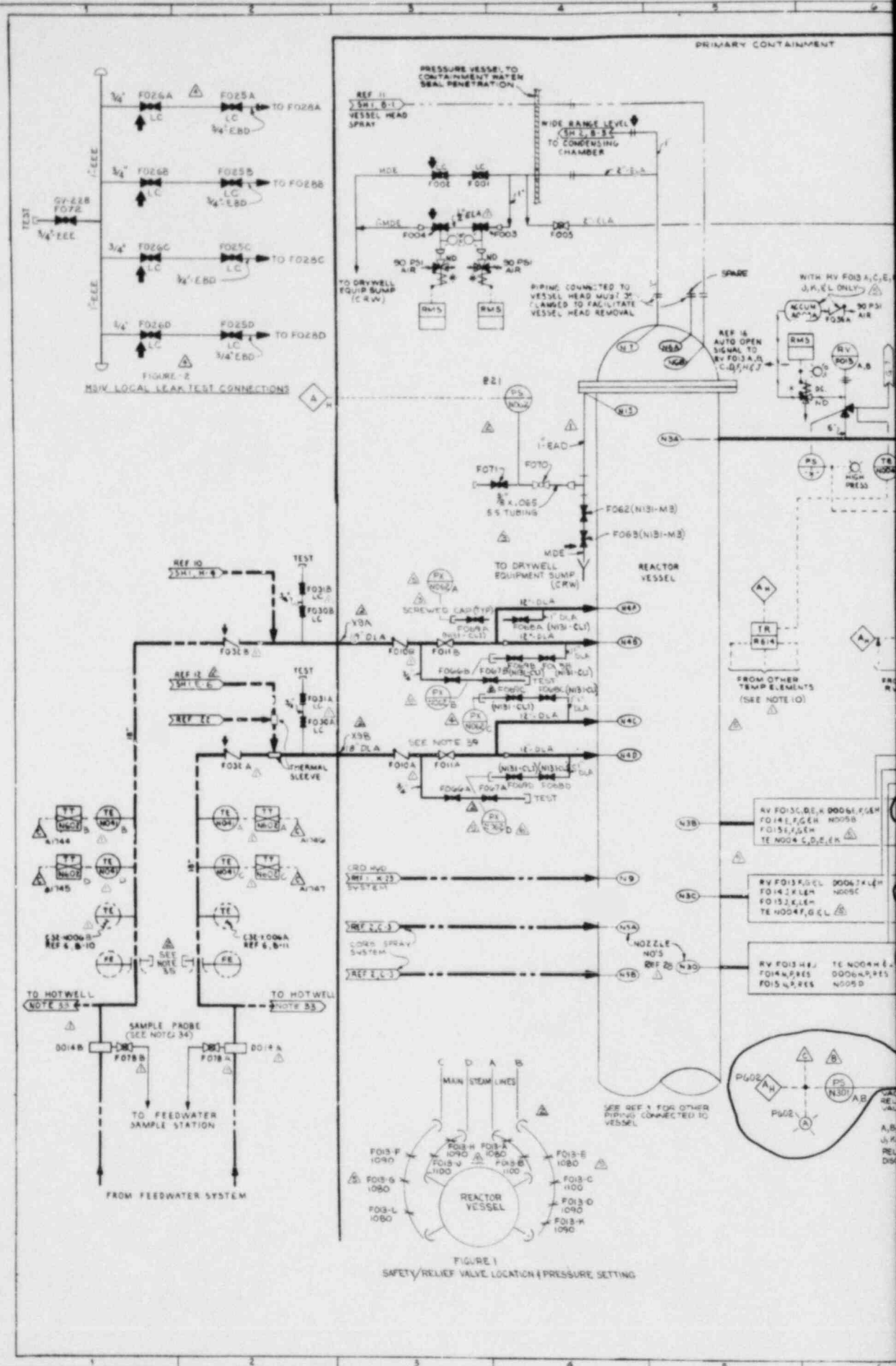
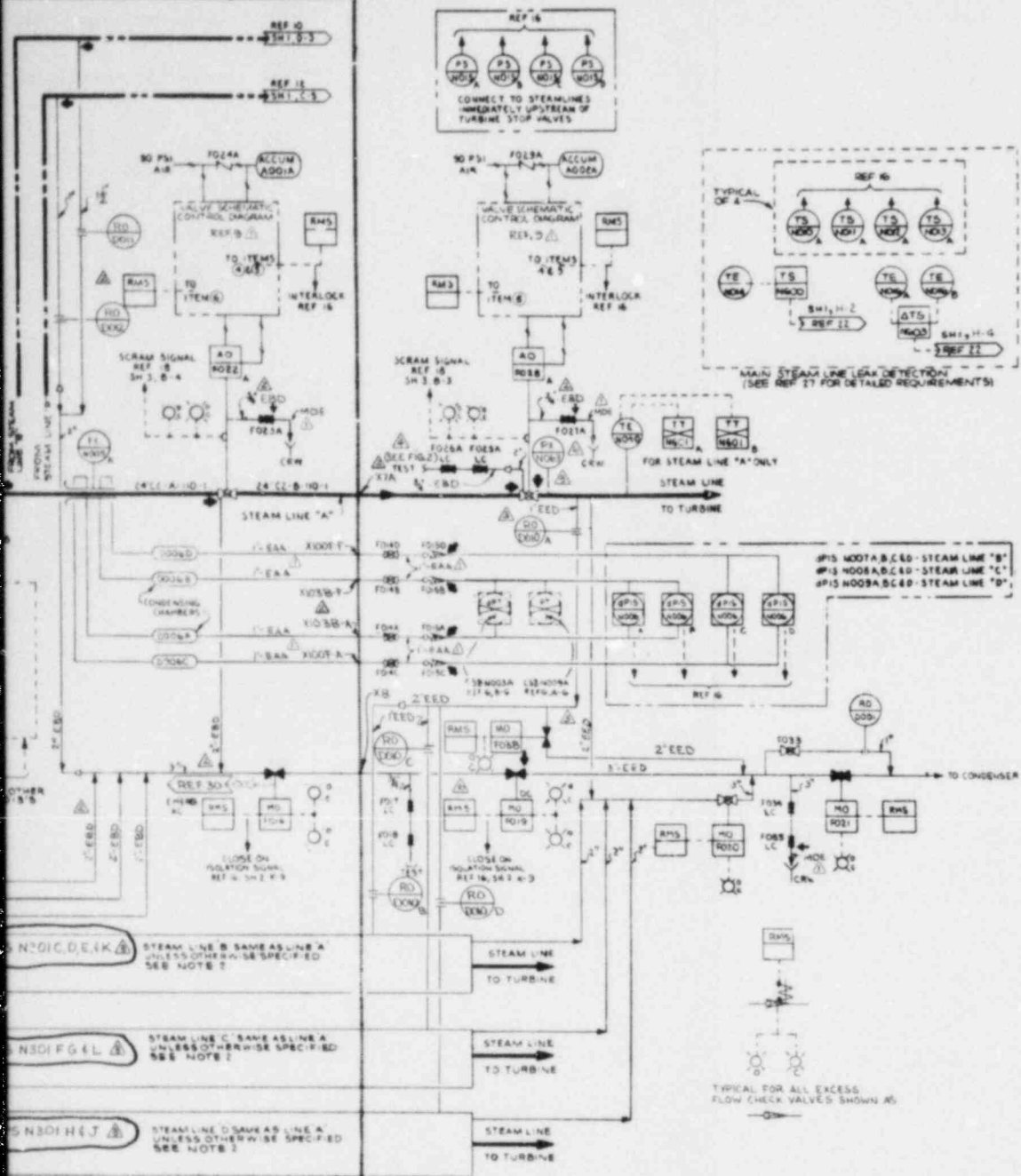


FIGURE 2  
MSW LOCAL LEAK TEST CONNECTIONS

FIGURE 1  
SAFETY/RELIEF VALVE LOCATION & PRESSURE SETTING



NOTES:

1. ALL EQUIPMENT & INSTRUMENTS ARE PRECEDED BY MPL NO. 521 UNLESS OTHERWISE NOTED.
2. STEAM LINES, ENCLOSED IN HOSES SHALL HAVE PART NOS. CORRESPONDING TO ITS RESPECTIVE LINE NO. UNLESS OTHERWISE NOTED.  
EXAMPLE: X338 IS ON LINE "B"  
X332 IS ON LINE "C"
3. WHERE SV NUMBERS ARE SHOWN THE VALVES ARE TAGGED WITH THESE NUMBERS, WHERE SV NUMBERS ARE NOT SHOWN THE VALVES ARE TAGGED WITH THE MPL NUMBER.
4. HIGH POINT VENTS AND LOW POINT DRAINS ARE TO BE PROVIDED WHERE NECESSARY AS PROVIDED BY PHYSICAL ROUTING OF PIPE.
5. INDICATED LEVEL TRIP SETTINGS VERSUS ACTUAL LEVEL INSIDE DRYER SEAL DRIFT IS BASED UPON:  
A. CALIBRATION OF DEVICES AT 1000 PSIG REACTOR DOME PRESSURE AND 133°F DRYER AMBIENT TEMPERATURE.  
B.  $\Delta P \times$  DRYER PRESSURE DROP AT RATED LOAD = 10" HOT WATER.  
C. CARRY-UNDER CORRECTION (BASED ON 0.25 BY WEIGHT CARRY-UNDER) = 0.5% READABILITY.  
D. FEEDWATER LEVEL SENSOR ERROR BAND 0.25 RANGE.  
E. SAFEGUARDS LEVEL SENSOR ERROR BAND 0.25 RANGE.
6. T/C JUNCTION BOX (LOCALLY MOUNTED BY OTHERS) EACH T/C JUNCTION BOX TO HAVE OWN SET OF TERMINALS.
7. AN EXPANSION LEAD SHALL BE PROVIDED IN INSTRUMENT SENSING LINE BETWEEN PRT PART 0002 AND THE WATER-TIGHT PENETRATION SEAL THROUGH BOTTOM OF REACTOR VESSEL. THE EXPANSION LEAD & PIPING IMMEDIATELY SHALL BE DESIGNED TO ALLOW FOR MAXIMUM CHANGE OF VESSEL LENGTH WITH TEMPERATURE TO AVOID OVERSTRESSING THE PIPING OR THE SEAL OR DAMAGE TO THE INSULATION AROUND THE VESSEL.
8. FOR LOCATION & IDENTIFICATION OF INSTRUMENTS SEE INSTRUMENT DATA SHEET LISTED IN MPL FOR EACH INSTRUMENT.

(NOTES CONTD ON SHT. 2)

REFERENCES

1. CONTROL ROD DRIVE HYDRAULIC SYS	D11-1010	8-16-60S
2. CORE SPRAY SYSTEM PAID	E21-1010	8-16-63S
3. REACTOR RECIRCULATION SYS PAID	SHT 1	8-16-60G
4. PIPING & INSTRUMENT SYMBOLS	REF 10, 12, 14, 16	8-16-60S
5. REACTOR VESSEL PURCHASE PART DWG	SHT 1-2	8-16-60S
6. FEEDWATER CONTROL SYSTEM IED	C32-1010	8-16-60S
7. NEUTRON MONITORING SYS IED	SHT 1-2	8-16-60S
8. STANDBY LOGIC CONTROL SYS PAID	C41-1010	8-16-60S
9. ISOLATION VALVE PURCH PART DWG	SHT 1-2	8-16-60S
10. HPCI SYSTEM PAID	SHT 1	8-16-60S
11. RWR SYSTEM PAID	SHT 1	8-16-60S
12. RBC SYSTEM PAID	SHT 1	8-16-60S
13. HPCI SYSTEM FCD	SHT 1-3	8-16-60S
14. RWR SYSTEM FCD	SHT 1-3	8-16-60S
15. RBC SYSTEM FCD	SHT 1-3	8-16-60S
16. NUCLEAR BOILER SYSTEMS FCD	SHT 1-3	8-16-60S
17. CORE SPRAY SYSTEM FCD	SHT 1-3	8-16-60S
18. REACTOR PROTECTION SYSTEM IED	SHT 1-3	8-16-60S
19. PROCESS INSTRUMENT PIPING & TUBING INSTALL. SPECIFICATION	481-4870	8-16-60S
20. PLANT REQUIREMENTS	481-4870	8-16-60S
21. REACTOR RECIRCULATION SYSTEM FCD	SHT 1-3	8-16-60S
22. REACTOR WATER CLEANUP SYSTEM P & I	SHT 1	8-16-60S
23. PRESSURE INTEGRITY OF PIPING & EQUIPMENT PRESSURE PARTS	481-4870	8-16-60S
24. NUCLEAR BOILER SYSTEM PROCESS DIAG	821-1020	8-16-60S
25. NUCLEAR BOILER SYSTEM DESIGN SPEC.	821-1020	8-16-60S
26. FEEDWATER CONTROL SYSTEM DESIGN SPEC.	C32-4810	8-16-60S
27. NUCLEAR BOILER LEAK DETECTION DESIGN SPEC.	481-4840	8-16-60S
28. REACTOR SYSTEM OUTLINE	SHT 2	8-16-60S
29. REACTOR ASSEMBLY DRYWELL VALVE & EQUIP DRAINAGE SYS. P&ID	SHT 2	8-16-60S

THIS DWG. DEVELOPED FROM G.E. DWG. NO. 725656BA REV. 5 SHT. 1.

STREAM LINE B SAME AS LINE A UNLESS OTHERWISE SPECIFIED SEE NOTE 2

STREAM LINE C SAME AS LINE A UNLESS OTHERWISE SPECIFIED SEE NOTE 2

STREAM LINE D SAME AS LINE A UNLESS OTHERWISE SPECIFIED SEE NOTE 2

PRESSURE TEMPERATURE INDEX

P-T INDEX	DESIGN	PEAK	MIN		
	PSIG	°F	°F		
1	115	180	137	583	406
2	115	180	137	583	70
3	5	212	-	-	-
4	448	457	530	477	-
5	475	575	475	575	40
6	44	347	345	340	70
7	44	347	345	340	40

NO.	DATE	BY	CHKD.	APP'D.	REVISION
1	10-10-62	J. W. D.	J. W. D.	J. W. D.	REVISED AS NOTED
2	10-10-62	J. W. D.	J. W. D.	J. W. D.	REVISED AS NOTED
3	10-10-62	J. W. D.	J. W. D.	J. W. D.	REVISED AS NOTED
4	10-10-62	J. W. D.	J. W. D.	J. W. D.	REVISED AS NOTED
5	10-10-62	J. W. D.	J. W. D.	J. W. D.	REVISED AS NOTED
6	10-10-62	J. W. D.	J. W. D.	J. W. D.	REVISED AS NOTED
7	10-10-62	J. W. D.	J. W. D.	J. W. D.	REVISED AS NOTED
8	10-10-62	J. W. D.	J. W. D.	J. W. D.	REVISED AS NOTED

(MPL NO. 521-1010)

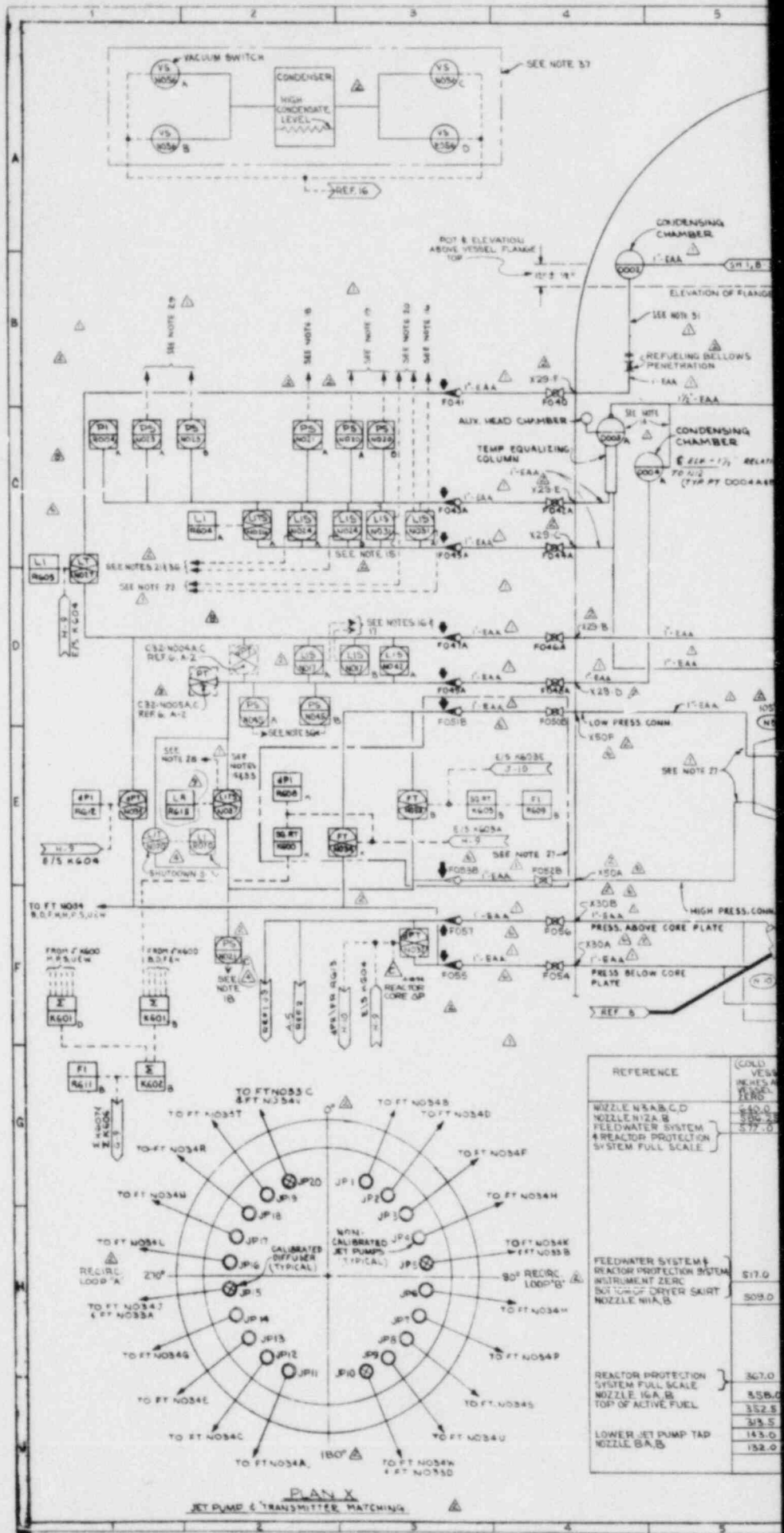
BECHTEL ASSOCIATES  
JOB 6511

SOUTHERN SERVICES INC.  
FOR

GEORGIA POWER CO., ATLANTA, GA.  
GENERAL ENGINEERING DEPARTMENT

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1  
NUCLEAR BOILER SYSTEM P&ID  
SHEET 1

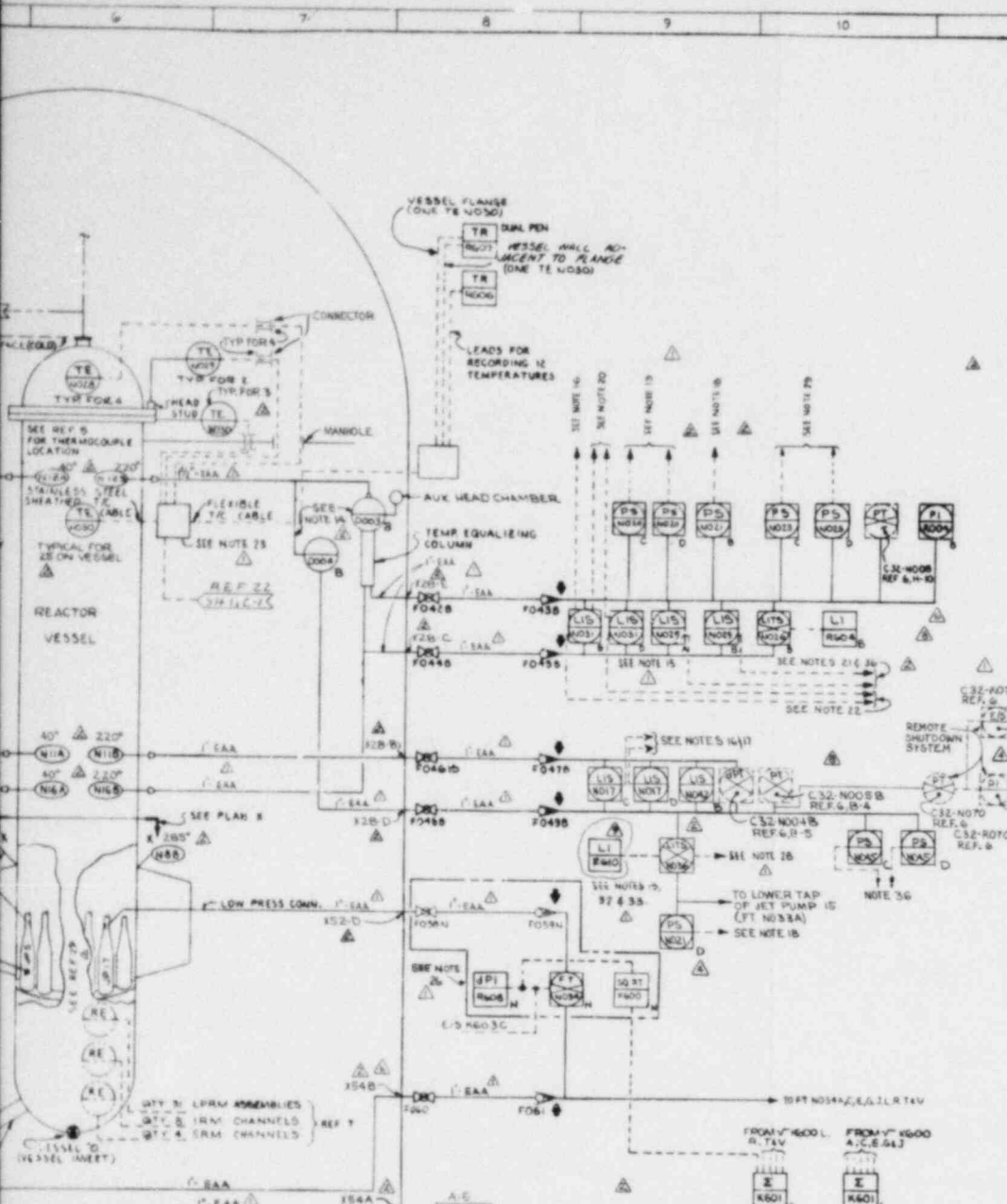
DATE	10-10-62
SCALE	1"=10'
DRAWING NUMBER	10-502
LOCATION	10-502
SHEET NO.	H-16062



REFERENCE	(COLD) VESSEL HEIGHTS ABOVE REF. B
NOZZLE 16A,B,C,D	550.0
NOZZLE 16A,B	552.5
FEEDWATER SYSTEM & REACTOR PROTECTION SYSTEM FULL SCALE	577.0
FEEDWATER SYSTEM & REACTOR PROTECTION SYSTEM INSTRUMENT ZERO	517.0
80° TOP OF DRYER SKIRT	509.0
NOZZLE 16A,B	509.0
REACTOR PROTECTION SYSTEM FULL SCALE	367.0
NOZZLE 16A,B	558.0
TOP OF ACTIVE FUEL	352.5
LOWER JET PUMP TAP	318.5
NOZZLE 16A,B	145.0
NOZZLE 16A,B	152.0

PLAN X  
JET PUMP & TRANSMITTER MATCHING



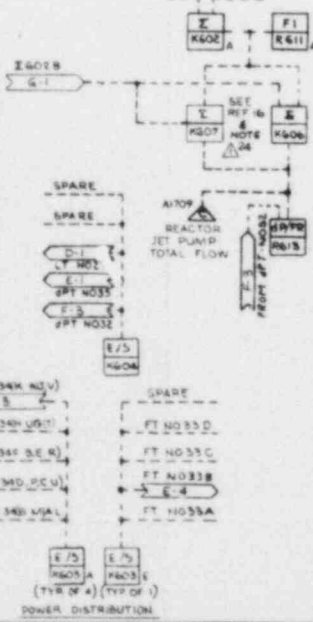


- NOTES CONT'D**
10. ALL RELIEF AND SAFETY VALVE DISCHARGE THERMOCOUPLES SHALL BE CONNECTED TO TEMPERATURE RECORDER RG14.
  11. INSTRUMENTS, INSTRUMENT PIPING AND VALVES MUST COMPLY WITH THE REQUIREMENTS OF REF 10.
  12. ALL MOTOR AND SOLENOID OPERATED VALVES ARE NORMAL AC UNLESS OTHERWISE NOTED.
  13. LINES TO DIFFERENTIAL PRESSURE TRANSMITTERS SHOULD BE AS SHOWN AS PRACTICAL.
  14. INSTALL TEMPERATURE EQUALIZING COLUMN AND LEVEL INSTRUMENT PIPING AS DIRECTED BY VENDORS INSTALLATION DRAWING.
  15. ALARMS ASSOCIATED WITH THE SYSTEMS INITIATED BY THE REACTOR PROTECTION SYSTEM OR SAFEGUARD SYSTEM LEVEL AND PRESSURE SWITCHES ARE SHOWN ON THE P&ID FOR THE PARTICULAR SYSTEM.
  16. TRIP RELIC AND HPCI TURBINES ON HIGH LEVEL (REF 15 & 15).
  17. SCRAM (REF 15) AND INITIATE PRIMARY CONTAINMENT ISOLATION EXCEPT MAIN STEAM LINE.
  18. CORE SPRAY AND RHR SYSTEM VALVE OPENING PERMISSIVE (REF 14 & 15).
  19. MAIN STEAM LINE ISOLATION VALVE CLOSURE INTERLOCK (REF 15) AND CONTRIBUTE TO LOW CONDENSER VACUUM BY-PASS (REF 15).
  20. INITIATE HPCI SYSTEM (REF 15), DCIC SYSTEM (REF 15).
  21. INITIATE CLOSURE OF MAIN STEAM LINE ISOLATION VALVES (REF 14).
  22. CONTRIBUTE TO AUTOBLOWDOWN (REF 14), INITIATE CORE SPRAY (REF 17) RHR SYSTEM (REF 14) AND START STANDBY DIESEL GENERATOR (REF 17).
  23. WATER TIGHT JUNCTION BOX TO BE LOCATED INSIDE DRYWELL.
  24. SUMMER K606 & K607 VALVES SHALL BE INTERLOCKED WITH EXHIBIT PUMP AND VALVES TO ADD "N". WHEN BOTH PUMPS ARE RUNNING AND THEIR DISCHARGE VALVES ARE OPEN OR INTERLOCK LINE INPUT WHEN THE CORRESPONDING PUMP IS STOPPED OR ITS DISCHARGE VALVE IS CLOSED.
  25. NOZZLES GA 4 & 6 MAY BE INTERCHANGED.
  26. TYPICAL FOR ALL (60) NON-CALIBRATED JET PUMP EXCEPT FOR ASSIGNMENT LETTER SUFFIXES. FOR LETTER SUFFIX ASSIGNMENT SEE PLAN "A" (L-5).
  27. TYPICAL FOR ALL (6) CALIBRATED JET PUMPS EXCEPT FOR INSTRUMENT LETTER SUFFIXES. FOR LETTER SUFFIX ASSIGNMENT SEE PLAN "A" (L-5).
  28. CONTAINMENT SPRAY MODE RHR INTERLOCK (REF 14).
  29. REACTOR PROTECTION SYS SCRAM SIGNAL (REF 15).
  30. RHR INTERLOCK (LPCI MODE) (REF 14).
  31. LITS NO 972 & R618 SHALL BE DUAL SCALE DEVICES-ONE SCALE SHALL CORRESPOND WITH THAT FOR LITS NO 53 & R600 (NOT OVER) (THE OTHER SHALL CORRESPOND TO COLD SHUT ON CONDITIONS (195° F & 1400 PSI)).
  32. INSTRUMENTS READ FULL SCALE WHEN JET PUMPS ARE IN OPERATION.
  33. RECIRCULATION LINES TO HOTWELL TO COMPLY WITH REF 20 WATER QUALITY SECTION 7.
  34. SAMPLE PROBE(S) AND FEEDWATER SAMPLE STATION TO COMPLY WITH REF 20, WATER SAMPLING SECTION 5.
  35. ALTERNATE TAP SET ON FEEDWATER FLOW ELEMENT.
  36. TRIP RECIRC. PUMP (REF 21).
  37. LOW CONDENSER VACUUM SWITCHES CONNECTED THROUGH SEPARATE CALIBRATION VALVES TO OPPOSITE SIDES OF THE CONDENSER ABOVE THE HIGH CONDENSATE LEVEL. THE VACUUM SWITCHES MUST BE ACCESSIBLE DURING PLANT OPERATION.
  38. AN ORIFICE (MINIMUM) IS TO BE PROVIDED WITHIN THE PRIMARY CONTAINMENT REACH INSTRUMENT LINE WHICH CONNECTS TO THE REACTOR COOLANT PRESSURE BOUNDARY.
  39. TO BE INSTALLED IN A STRAIGHT RUN OF 12" FEEDWATER PIPE 6" O" FROM % OF ELBOWS ETC AND LOCATED SO THAT 2 PSI FROM TAPS TO THE VESSEL NOZZLES ARE EQUAL TAPS TO MEET ASME ETC 8, 10.4 STEAM TURBINE PARA. 4.74. THESE WIRES ARE TO BE PROVIDED PENETRATING THE DRYWELL FOR READOUT OF TEMPORARY PRESSURE TRANSMITTERS DURING START-UP.

THIS DWG. DEVELOPED FROM G.E. DWG. NO. 729E616BA REV.5 SHT.2

ELEVATION CORRELATION CHART (SEE NOTES)

DESCRIPTION OF TRIPS	INSTRUMENTS PROVIDING TRIP	REACTOR VESSEL LEVELS REF. 3.5	INDICATED LEVEL & TRIP SETTINGS (NOTE D)
TRIP REC & HPCI TURBINE	LIS NO 57A, B, C	6	+58.0
TRIP REACTOR FEED PUMPS	LIS NO 24A, B, C	7	+42.0
CLOSE MAIN TURBINE STOP VALVES	LRS C3-R605	3.6	+32.0
HIGH LEVEL ALARM	LRS C3-R605	4	+32.0
NORMAL WATER	LRS C3-R605	3	+12.5
LOW LEVEL ALARM	LIS NO 7A, B, C	3	+12.5
SCRAM/CLOSE PRIMARY SYSTEMS	LIS NO 7A, B, C	3	+12.5
ISOLATION VALVES EXCEPT MAIN STEAM LINE	LIS NO 7A, B, C	3	+12.5
AUTO DEPRESSURIZATION PERMISSIVE	LIS NO 24A, B, C	7	+42.0
INITIATE RELIC & HPCI & TRIP RECIRCULATION PUMPS	LIS NO 57A, B, C	6	+58.0
INITIATE CORE SPRAY SYSTEMS	LIS NO 24A, B, C	7	+42.0
CONTRIBUTE TO AUTO DEPRESSURIZATION SYSTEM START STANDBY DIESEL	LIS NO 57A, B, C	6	+58.0
CONTAINMENT SPRAY PERMISSIVE	LITS NO 37 LITS NO 37	0	-146.5 -150.0



NO.	DATE	BY	CHKD	APP'D	REVISION
1	10/10/78	J. H. HATCH	J. H. HATCH	J. H. HATCH	INITIAL DESIGN
2	11/10/78	J. H. HATCH	J. H. HATCH	J. H. HATCH	REVISED FOR P&ID
3	12/10/78	J. H. HATCH	J. H. HATCH	J. H. HATCH	REVISED FOR P&ID
4	01/10/79	J. H. HATCH	J. H. HATCH	J. H. HATCH	REVISED FOR P&ID
5	02/10/79	J. H. HATCH	J. H. HATCH	J. H. HATCH	REVISED FOR P&ID
6	03/10/79	J. H. HATCH	J. H. HATCH	J. H. HATCH	REVISED FOR P&ID
7	04/10/79	J. H. HATCH	J. H. HATCH	J. H. HATCH	REVISED FOR P&ID
8	05/10/79	J. H. HATCH	J. H. HATCH	J. H. HATCH	REVISED FOR P&ID
9	06/10/79	J. H. HATCH	J. H. HATCH	J. H. HATCH	REVISED FOR P&ID
10	07/10/79	J. H. HATCH	J. H. HATCH	J. H. HATCH	REVISED FOR P&ID
11	08/10/79	J. H. HATCH	J. H. HATCH	J. H. HATCH	REVISED FOR P&ID
12	09/10/79	J. H. HATCH	J. H. HATCH	J. H. HATCH	REVISED FOR P&ID

(MFL NO. 821-1010)

**RECHTEL ASSOCIATES**  
100 8511

**SOUTHERN SERVICES INC.**  
FOR

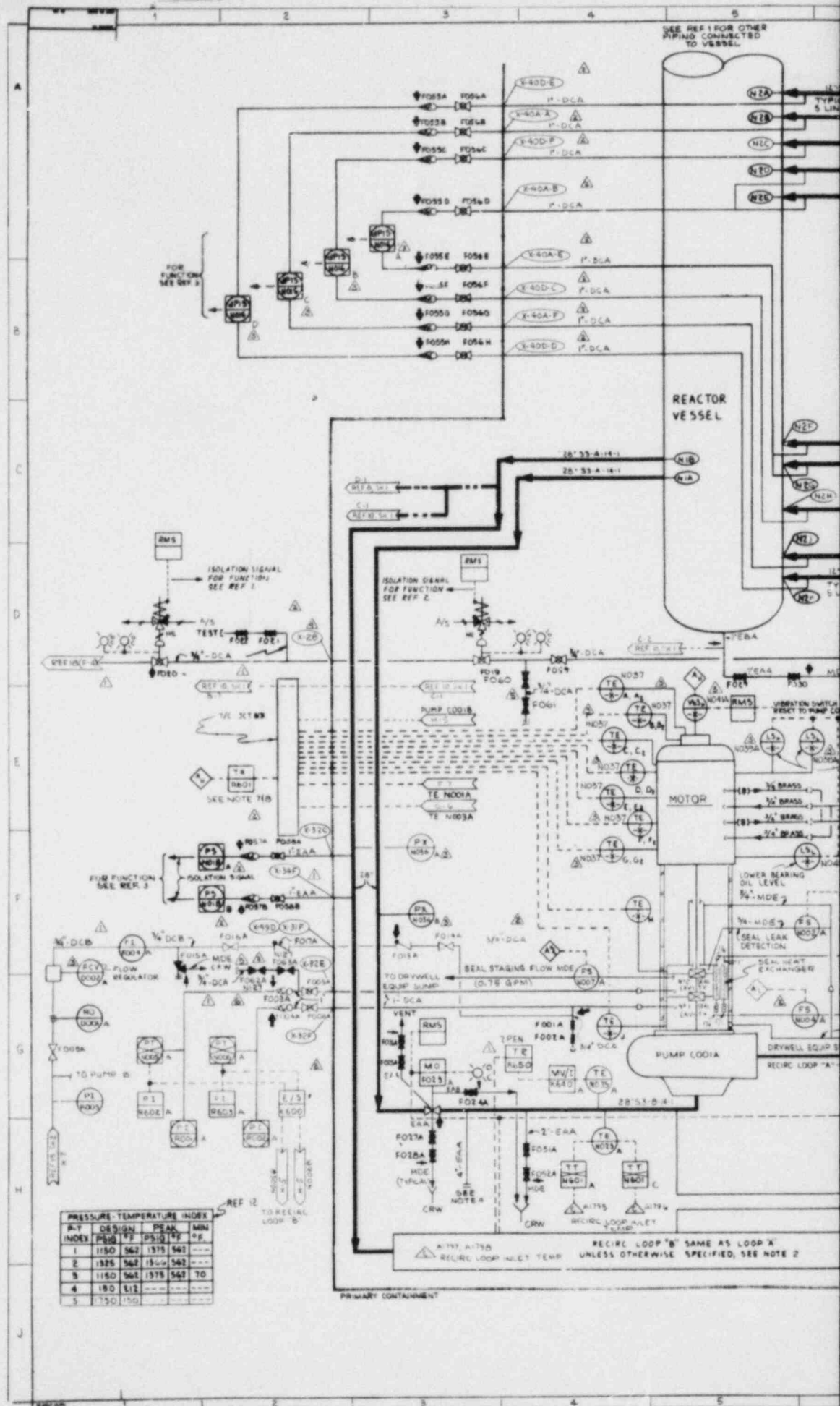
**GEORGIA POWER CO., ATLANTA, GA.**  
GENERAL ENGINEERING DEPARTMENT

**EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1**  
NUCLEAR BOILER SYSTEM P&ID  
SHEET 2

DESIGNED BY	DATE
DRAWN BY	DATE
CHECKED BY	DATE
APPROVED BY	DATE

10-502 H-6063

*Handwritten signature and date: 4-11-77*

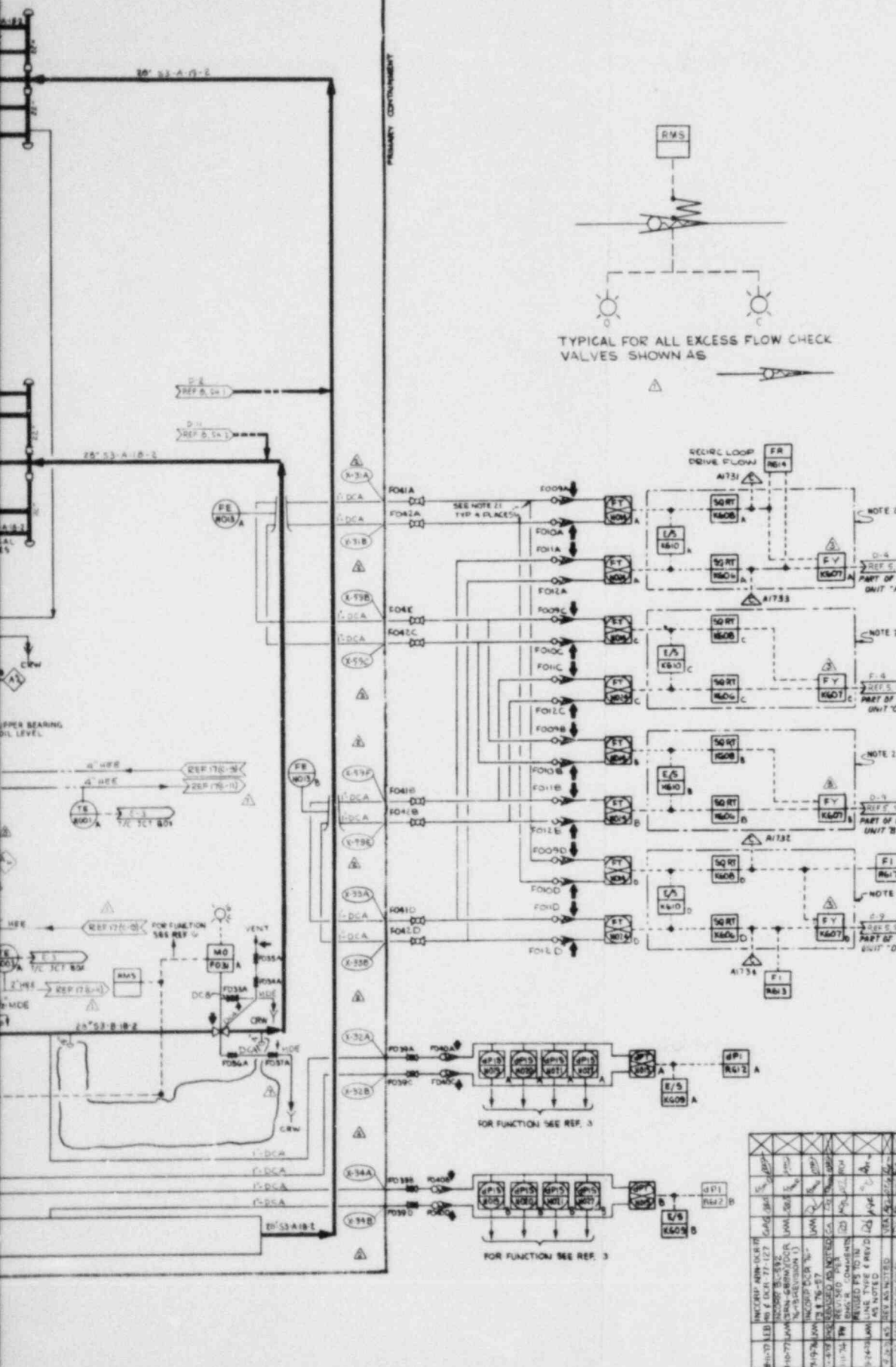


REF 12

PRESSURE-TEMPERATURE INDEX				
P-T INDEX	DESIGN PSIG	PEAK PSIG	MIN OF	
1	1150	562	1375	562
2	1325	562	1510	562
3	1150	562	1375	562 TO
4	180	512		
5	750	750		

RECIRC LOOP "B" SAME AS LOOP "A" UNLESS OTHERWISE SPECIFIED, SEE NOTE 2

PRIMARY CONTAINMENT



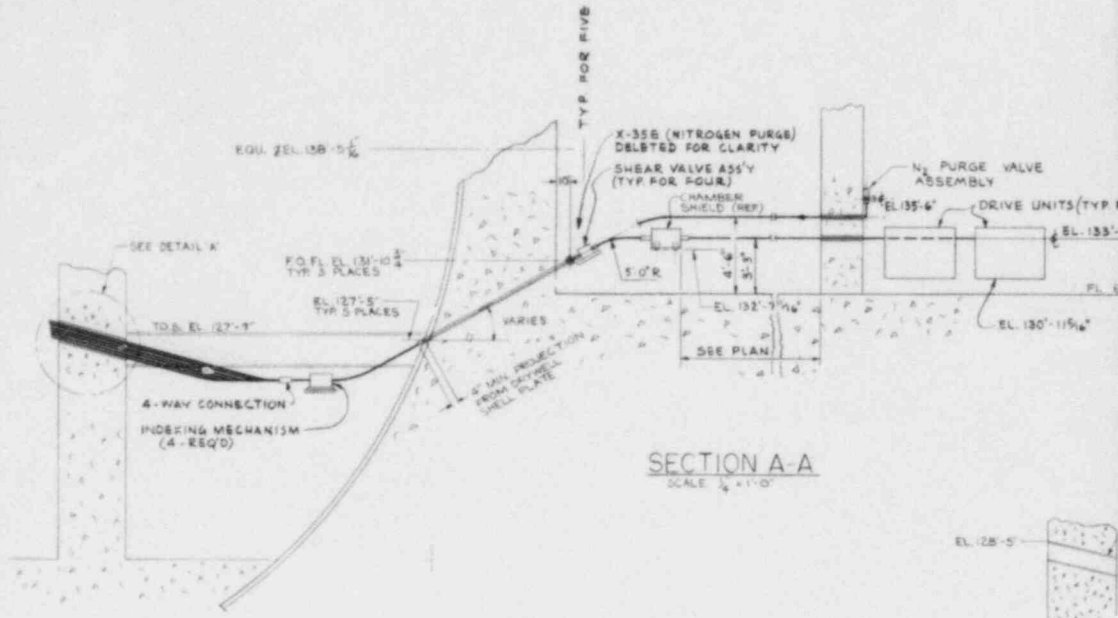
TYPICAL FOR ALL EXCESS FLOW CHECK VALVES SHOWN AS

- REFERENCES
- 1. NUCLEAR SAFETY SYSTEM P&ID SET 1 821-1018 H-10-052
  - 2. REACTOR COOLING SYSTEM P&ID, PART 2 821-1030 H-10-053
  - 3. REACTOR MONITORING SYSTEM P&ID 821-1015 H-10-054
  - 4. REACTOR RECIRCULATION SYSTEM P&ID 821-1016 H-10-055
  - 5. REACTOR RECIRCULATION SYSTEM P&ID 821-1017 H-10-056
  - 6. REACTOR RECIRCULATION SYSTEM P&ID 821-1018 H-10-057
  - 7. REACTOR RECIRCULATION SYSTEM P&ID 821-1019 H-10-058
  - 8. REACTOR RECIRCULATION SYSTEM P&ID 821-1020 H-10-059
  - 9. REACTOR RECIRCULATION SYSTEM P&ID 821-1021 H-10-060
  - 10. REACTOR RECIRCULATION SYSTEM P&ID 821-1022 H-10-061
  - 11. REACTOR RECIRCULATION SYSTEM P&ID 821-1023 H-10-062
  - 12. REACTOR RECIRCULATION SYSTEM P&ID 821-1024 H-10-063
  - 13. REACTOR RECIRCULATION SYSTEM P&ID 821-1025 H-10-064
  - 14. REACTOR RECIRCULATION SYSTEM P&ID 821-1026 H-10-065
  - 15. REACTOR RECIRCULATION SYSTEM P&ID 821-1027 H-10-066
  - 16. REACTOR RECIRCULATION SYSTEM P&ID 821-1028 H-10-067
  - 17. REACTOR RECIRCULATION SYSTEM P&ID 821-1029 H-10-068
  - 18. REACTOR RECIRCULATION SYSTEM P&ID 821-1030 H-10-069

- NOTES
1. ALL EQUIPMENT & INSTRUMENTS ARE SPECIFIED BY DESIGN NO. SEE UNLESS OTHERWISE NOTED.
  2. RECIRCULATION LOOP INCLUDED IN THIS SHALL HAVE FLOW METER CORRECTING FOR ITS RESPECTIVE LINE OR LOOP NUMBER UNLESS OTHERWISE NOTED.
  3. INSTRUMENT LINE VALVES MUST COMPLY WITH INSTRUMENT PIPING STANDARDS REF. 12.
  4. DECONTAMINATION CONNECTION TO BE READILY ACCESSIBLE FOR DECONTAMINATION & RAPID CONNECTION OF TIGHTENING PIPING.
  5. WHERE REF. NO. NUMBERS ARE LISTED, THE VALVES WILL BE TAGGED WITH THESE NUMBERS, WHERE NO NUMBERS ARE NOT LISTED, THE VALVES WILL BE TAGGED WITH THE SPLY. NUMBER.
  6. CLOSED COOLING WATER TO BE FROM THE RECIRCULATION PUMP SHALL BE CAPABLE OF CONTINUOUS OPERATION DURING PERIODS OF NORMAL OPERATION.
  7. MAKE THERMOWELLS FOR PUMP & MOTOR AND DESIGNATION A<sub>1</sub>, A<sub>2</sub>, etc. BY 1/2 X 1/4 STEEL ELEMENT.
  8. LIST OF PUMP & MOTOR SPECIFICATIONS TO BE MADE UP BY 1/2 X 1/4 STEEL ELEMENT SPECIFICATION ON SPT'S, (A)-(D)  
 TOL. A<sub>1</sub> - 1/2" - 1/4" - 1/8" - 1/16" - 1/32" - 1/64"  
 TOL. A<sub>2</sub> - 1/2" - 1/4" - 1/8" - 1/16" - 1/32" - 1/64"  
 TOL. B<sub>1</sub> - MOTOR BEARING "1"  
 TOL. B<sub>2</sub> - MOTOR BEARING "2"  
 TOL. C<sub>1</sub> - MOTOR BEARING "1"  
 TOL. C<sub>2</sub> - MOTOR BEARING "2"  
 TOL. D<sub>1</sub> - 1/2" - 1/4" - 1/8" - 1/16" - 1/32" - 1/64"  
 TOL. D<sub>2</sub> - 1/2" - 1/4" - 1/8" - 1/16" - 1/32" - 1/64"  
 TOL. E<sub>1</sub> - MOTOR BEARING "1"  
 TOL. E<sub>2</sub> - MOTOR BEARING "2"  
 TOL. F<sub>1</sub> - MOTOR BEARING "1"  
 TOL. F<sub>2</sub> - MOTOR BEARING "2"  
 TOL. G<sub>1</sub> - MOTOR BEARING "1"  
 TOL. G<sub>2</sub> - MOTOR BEARING "2"  
 TOL. H<sub>1</sub> - MOTOR BEARING "1"  
 TOL. H<sub>2</sub> - MOTOR BEARING "2"  
 TOL. I<sub>1</sub> - MOTOR BEARING "1"  
 TOL. I<sub>2</sub> - MOTOR BEARING "2"  
 TOL. J<sub>1</sub> - MOTOR BEARING "1"  
 TOL. J<sub>2</sub> - MOTOR BEARING "2"  
 TOL. K<sub>1</sub> - MOTOR BEARING "1"  
 TOL. K<sub>2</sub> - MOTOR BEARING "2"  
 TOL. L<sub>1</sub> - MOTOR BEARING "1"  
 TOL. L<sub>2</sub> - MOTOR BEARING "2"  
 TOL. M<sub>1</sub> - MOTOR BEARING "1"  
 TOL. M<sub>2</sub> - MOTOR BEARING "2"  
 TOL. N<sub>1</sub> - MOTOR BEARING "1"  
 TOL. N<sub>2</sub> - MOTOR BEARING "2"  
 TOL. O<sub>1</sub> - MOTOR BEARING "1"  
 TOL. O<sub>2</sub> - MOTOR BEARING "2"
  9. ALL THERMOWELLS ARE TO BE WELDED INTO THROUGH OPENING TO THE JUNCTION BOX.
  10. ALL MOTOR OPERATED AND AIR OPERATED BLOCK VALVES ARE TO BE CLOSED BY HAND.
  11. THESE SPECIFIED SHALL BE PART OF AND MOUNTED IN PANEL PART NO. P&ID PROTECTION & WELDED THEREON.
  12. ALL 2" DIA. MOTOR OPERATED VALVES & INSTRUMENTS SHALL BE PROVIDED WITH WELDED AND MOUNTED ON THE MOTOR OPERATING CONTROL.
  13. VALVE REGULATOR SHALL BE SUPPLIED AS PART OF PART NO. 1000 BUT SHALL BE INSTALLED, AS PART OF PART NO. P&ID.
  14. THIS TYPE OF BLOCK REPRESENTS A PERMISSIVE CONNECTION WHEN THE CONDITIONS INSIDE THEM ARE SATISFIED.
  15. THIS LIGHTER OFFICE IS FOR P&ID PROTECTION, WITH OPERATING & CLOSING THIS OFFICE SHALL BE TIME DELAYED IS REQUIRED.
  16. THE MOTOR CONTROLLER SWITCHES THIS SIGNAL TO THE MOTOR SET POINT CONTROLLER FOR EACH MOTOR, RESPONDING TO LOAD SIGNALS, IN THE NORMAL RUNNING AND STOPPING MODES WHEN THE MOTOR CONTROLLER IS IN THE AUTOMATIC MODE.
  17. EQUIPMENT RATINGS ARE ESTIMATED AND PRELIMINARY. ACTUAL VALUES ARE TO BE DETERMINED AT TIME OF EQUIPMENT PROCUREMENT.
  18. THESE SPEED LIMITERS REMOVE RECIRCULATION FLOW, IF THE RECIRCULATION PUMP IS NOT RUNNING AND THE LEVEL IN THE REACTOR VESSEL IS BELOW THE LOW LEVEL SLOTTED P&ID.
  19. FOR LOCATION AND IDENTIFICATION OF INSTRUMENTS SEE INSTRUMENT DATA SHEET LISTED IN B&P FOR EACH INSTRUMENT.
  20. THE DESIGN PRESSURE AND TEMPERATURE RATINGS FOR THE REACTOR PIPING AND EQUIP ARE SHOWN IN REF. 12 & 14.
  21. EQUIPMENT IN THESE BLOCKS IS TO BE SUPPLIED FOR LOCATION AND IDENTIFICATION WITHIN THE P&ID AS INDICATED.
  22. CLOSED COOLING WATER TO THE MOTOR BEARING IS TO SERVE BOTH THE UPPER MOTOR BEARING & THE LOWER MOTOR BEARING & THE TURN FLOWS ARE JOINED UPSTREAM OF THE TEMPERATURE ELEMENT.
  23. LINES WITH G.E. LINE CLASS DESIGNATIONS (I.E. 28" S3-A-1B-2) ARE FURNISHED BY G.E. AND ARE TO BE INSTALLED PER REF. 12.

THIS DWG DEVELOPED FROM G.E. DWG  
 720 E-603 B-A REV G  
 (MPL. NO. 831-1010)

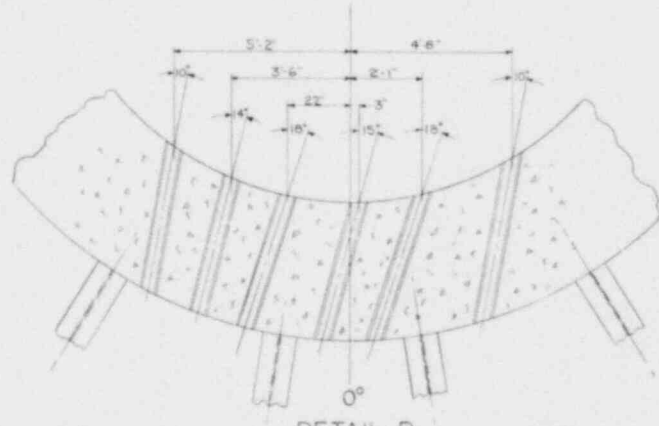
RECHTEL ASSOCIATES JOB 8511				
SOUTHERN SERVICES INC. FOR				
GEORGIA POWER CO., ATLANTA, GA. GENERAL ENGINEERING DEPARTMENT				
EDWIN HATCH NUCLEAR PLANT UNIT NO. 1 REACTOR RECIRCULATION SYSTEM P&ID SHEET NO. 1				
DATE	DRAWN	CHECKED	DATE	SHEET NO.
10-502				H-6056



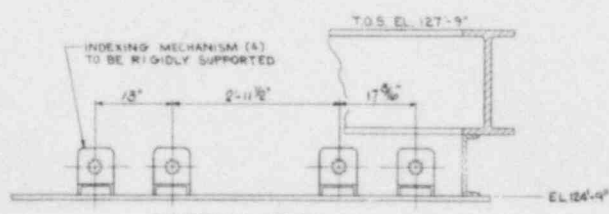
SECTION A-A  
SCALE 1/4" = 1'-0"



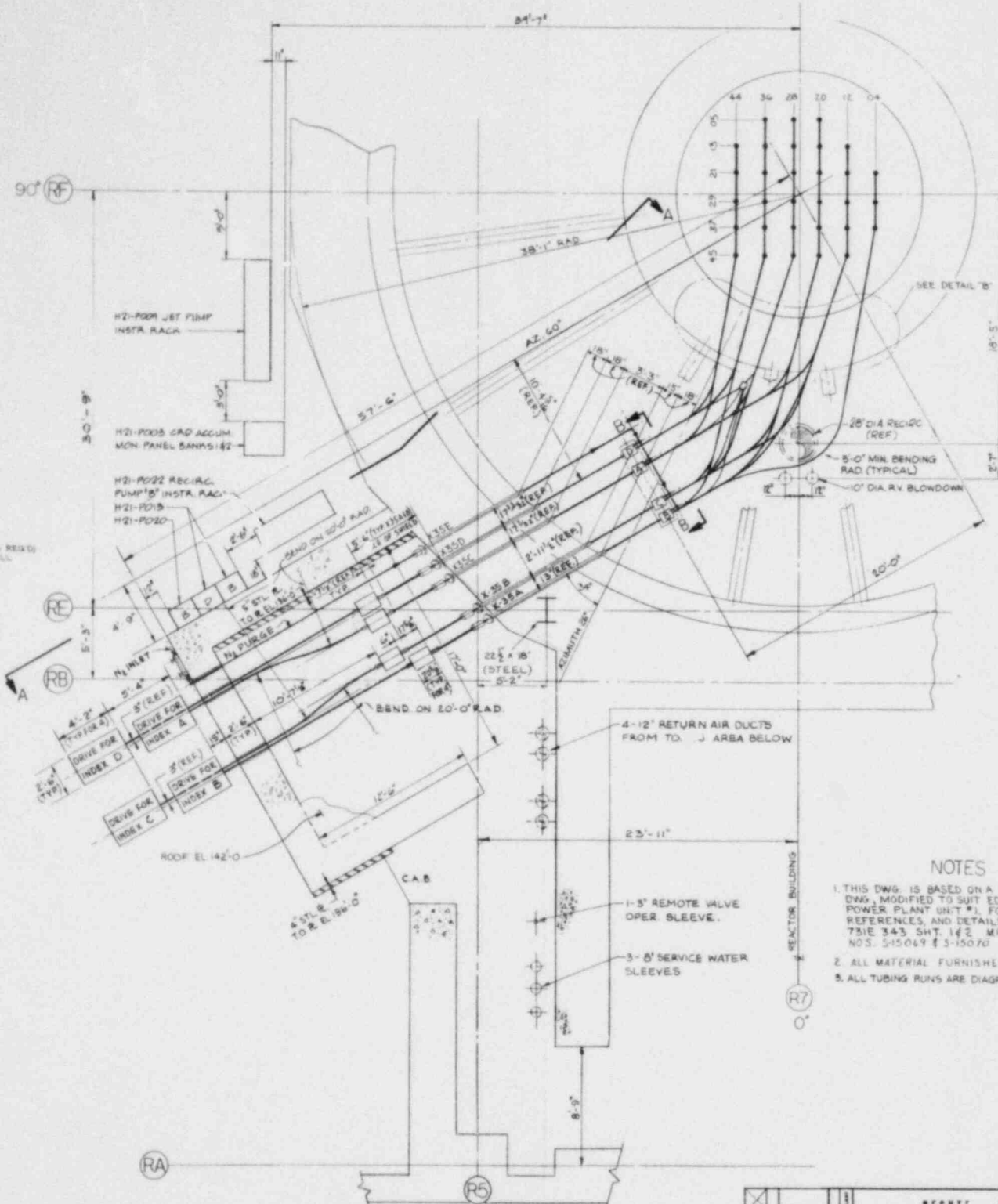
DETAIL  
SCALE 1/4" = 1'-0"



DETAIL B  
SCALE 1/4" = 1'-0"



SECTION B-B  
SCALE 1/4" = 1'-0"



- NOTES**
1. THIS DWG. IS BASED ON A 6\"/>  - 2. ALL MATERIAL FURNISHED BY GE.
  - 3. ALL TUBING RUNS ARE DIAGRAMMATICALLY SHOWN.

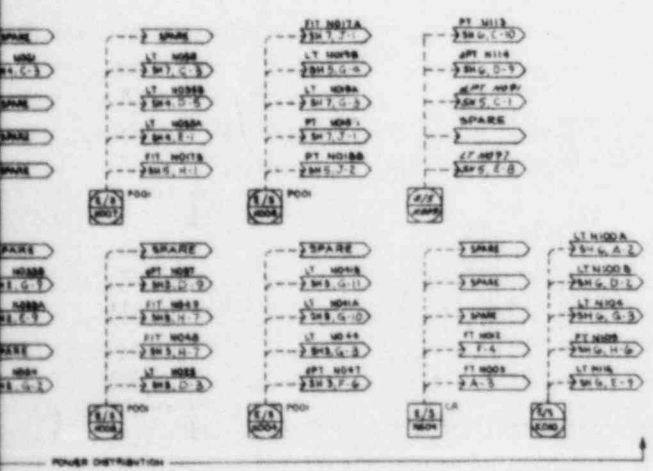
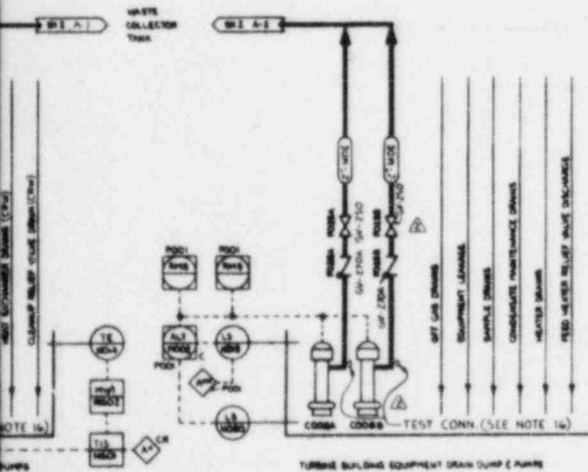
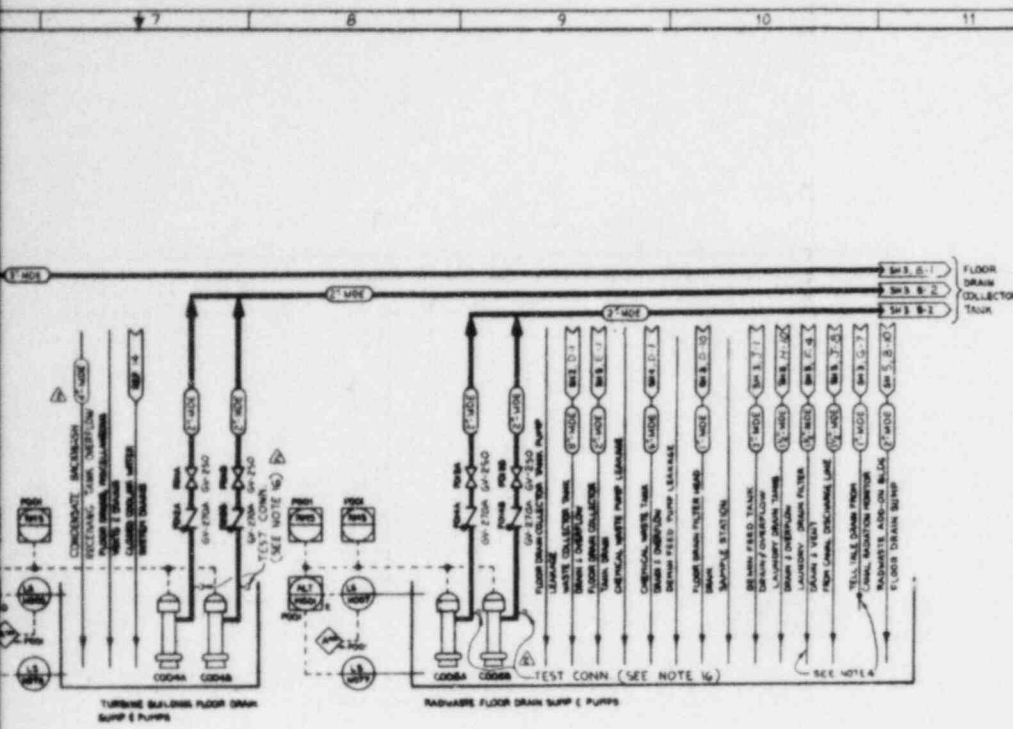
**PARTIAL PLAN AT EL 131'-6"**  
SCALE 1/4"=1'-0"

REVISIONS					
REVISIONS		DATE BY		REVISIONS	
1. HANS SENKOWICZ & ASSOCIATES FOR CONSTRUCTION		11-1-70		1	
2. REVISED TIP LOCUS		11-1-70		2	
3. LOCATION REVISIONS		11-1-70		3	
4. REQUEST RELOCATION		11-1-70		4	
5. ADDITIONAL REVISIONS		11-1-70		5	
6. ADDITIONAL REVISIONS		11-1-70		6	
7. ADDITIONAL REVISIONS		11-1-70		7	
8. ADDITIONAL REVISIONS		11-1-70		8	
9. ADDITIONAL REVISIONS		11-1-70		9	
10. ADDITIONAL REVISIONS		11-1-70		10	
11. ADDITIONAL REVISIONS		11-1-70		11	
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17. ADDITIONAL REVISIONS		11-1-70		17	
18. ADDITIONAL REVISIONS		11-1-70		18	
19. ADDITIONAL REVISIONS		11-1-70		19	
20. ADDITIONAL REVISIONS		11-1-70		20	

<b>BECHTOLD ASSOCIATES</b> JOB 8511	
<b>SOUTHERN SERVICES INC.</b> FOR	
GEORGIA POWER CO., ATLANTA, GA. GENERAL ENGINEERING DEPARTMENT	
EDWIN L HATCH NUCLEAR PLANT UNIT NO. 1 TRaversING IN-CORE PROBE SYSTEM	
SCALE AS NOTED	DATE 1-5-70
DRAWING NUMBER 10-502 H-16070	





- NOTES CONTINUED:
- TEST CONNECTIONS ARE TO BE PLUGGED AND PLUGS WELDED FOLLOWING PRE-OPERATIONAL TESTING OF THIS SYSTEM.
  - FOR DETAILED INFORMATION, SEE REF. 19 THIS DWG.
  - CATION FLOC & ANION FLOC MIXING TANKS, VALVES & EDUCATORS SUPPLIED BY CONDENSATE POLISHING SYSTEM CONTRACTOR.

- NOTES
- ALL EQUIPMENT & INSTRUMENTS ARE PREFIXED BY SYSTEM NO. G11 UNLESS OTHERWISE NOTED.
  - ALL AIR OPERATED VALVES ARE SHOWN IN FAIL AND NORMAL MODE.
  - INCOMING PIPING TO SUMP SHALL TERMINATE BELOW LOW WATER LEVEL TO PREVENT A WATER SEAL UNLESS OTHERWISE SHOWN. 30" DIA. LINE DRAINS SHALL BE SEALED AS ABOVE OR WITH LOOP SEALS SUFFICIENT TO PREVENT GYR LINES FROM ENTERING SUMP.
  - ALL MOTOR AND SOLENOID OPERATED VALVES, CENTRIFUGES AND PUMPS SHALL BE PROVIDED WITH ONE SET OF STATUS INDICATING LIGHTS ADJACENT TO THE MANUAL MANUAL SWITCH. ADDITIONAL LIGHTS ARE NOTED.
  - INTERLOCK TO PREVENT OPENING BOTH VALVES AT THE SAME TIME.
  - USED DURING PRESTARTUP ONLY. TO BE REMOVED AFTER STARTUP.
  - TANK VENTS AND SUMP VENTS SHALL BE PIPED TO BUILDING VENT SYSTEM, 12' FROM ROOM EXHAUST.
  - EQUIPMENT DRAINS & SEALS SHALL BE ROUTED TO EQUIPMENT DRAIN OR FLOOR DRAIN SYSTEM IN ACCORDANCE WITH DESIGN SPEC. FOR RADIOACTIVE DRAIN SYSTEM AND WILL NOT FLOW FREELY ACROSS THE FLOOR.
  - ONE SUMP PUMP WILL START AUTOMATICALLY ON HIGH LEVEL, THE SECOND PUMP WILL START AUTOMATICALLY ON HIGH-HIGH LEVEL AND BOTH WILL STOP AUTOMATICALLY ON LOW LEVEL.
  - FEED SYSTEM ON HIGH TORQUE, FLOOD ON HIGH-HIGH TORQUE & HIGH HOPPER LEVEL.
  - OVERFLOW LINES FROM CLOSED TOP TANKS WILL HAVE A TWO FOOT WATER SEAL FILLED BY A CONDENSATE LINE OR WILL BE SUBMERGED IN THE COLLECTING SUMP TO PREVENT HEATING THROUGH THE OVERFLOW.
  - SINGLE ALARMS AND INDICATING LIGHTS ARE LOCATED IN THE RADWASTE CONTROL ROOM UNLESS OTHERWISE INDICATED.
  - FOR SAMPLING LINES SEE REF. 1.
  - SEE NOTE 7.

REFERENCE

REF. NO.	DESCRIPTION	MPL NO.	REV.
1	CONTROL AND DRIVE HYDRAULIC SYSTEM P & ID	E11-1010	H-10-005
2	PROCESS ADDITION MON. FEED P & ID	SMT 1	E11-1010
3	RESIDUAL HEAT REMOVAL SYSTEM P & ID	SMT 2	E11-1010
4	RADWASTE SYSTEM P&ID	SMT 1	E11-1010
5	REACTOR WATER CLEANSUP JTS P & ID	SMT 1	E11-1010
6	PRECISION INTEGRITY SPECIFICATION	MS-4020	1
7	PLANT REQUIREMENTS	MS-4020	1
8	PIPING AND INSTRUMENT SYMBOLS	AG	1/85/1
9	RADWASTE SYSTEM P & ID	E11-1010	1
10	RADWASTE SYSTEM DES. SPEC.	E11-4010	1
11	CORE SPRAY SYSTEM P & ID	E11-1010	H-10-031
12	NUCLEAR MILLER SYSTEM P&ID	R21-1020	1
13	FUEL POOL COOLING SYS. P & ID	SA-1000	H-10-001
14	REACTOR BLDG. CFW SYS. P & ID	PA0-1000	H-10-009
15	RADWASTE BLDG. ADD. SUPPORT SYS. P&ID	QA-0407000	H-10-017
16	REACTOR & RADWASTE DRAINAGE DIAG.	T45-1020	H-10-018
17	FUEL POOL FILTER/DRAIN. SYS. P&ID	SA-1000	H-10-003
18	PIPING & INSTRUMENTATION DIAGRAM	SA-0000	SA-10-008
19	RADWASTE PACKAGING SYSTEM ELEMENTARY DIAGRAM	G11-P030	H-10-048
20	LEAK DETECTION SYSTEM	T45-1030	H-10-039
21	WASTE GAS TREATMENT BLDG. SUPPORT SYSTEMS P&ID	N6-2-1015	H-10-040

THIS DWG DEVELOPED FROM GE, DWG 729E614BA SHEET 1 REV. 3

MPL NO. G11-1010

**BECHTEL ASSOCIATES**  
JOB 6511

**SOUTHERN SERVICES INC.**  
FOR

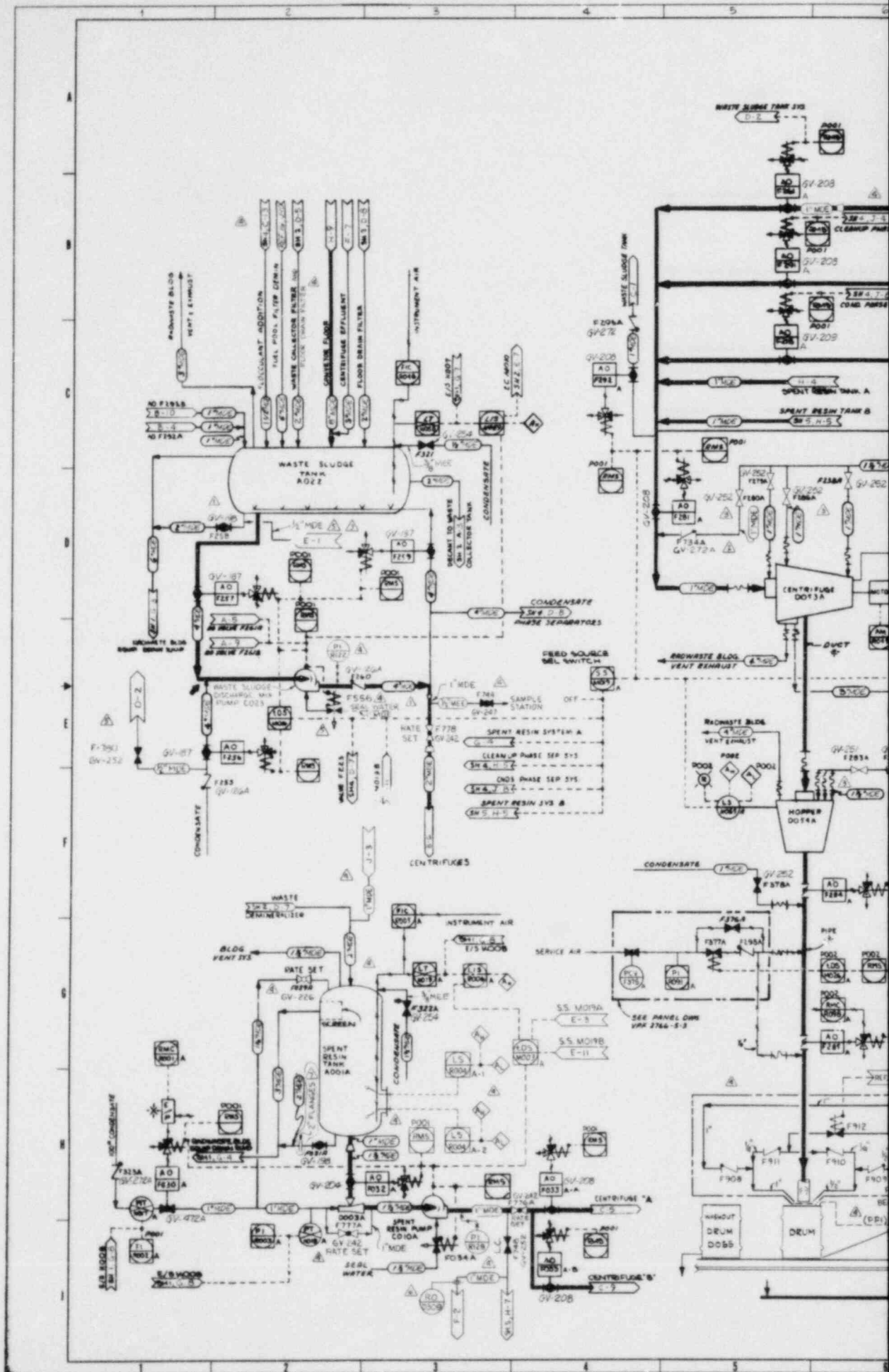
**GEORGIA POWER CO., ATLANTA, GA.**  
GENERAL ENGINEERING DEPARTMENT

**EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1**  
RADWASTE SYSTEM P&ID

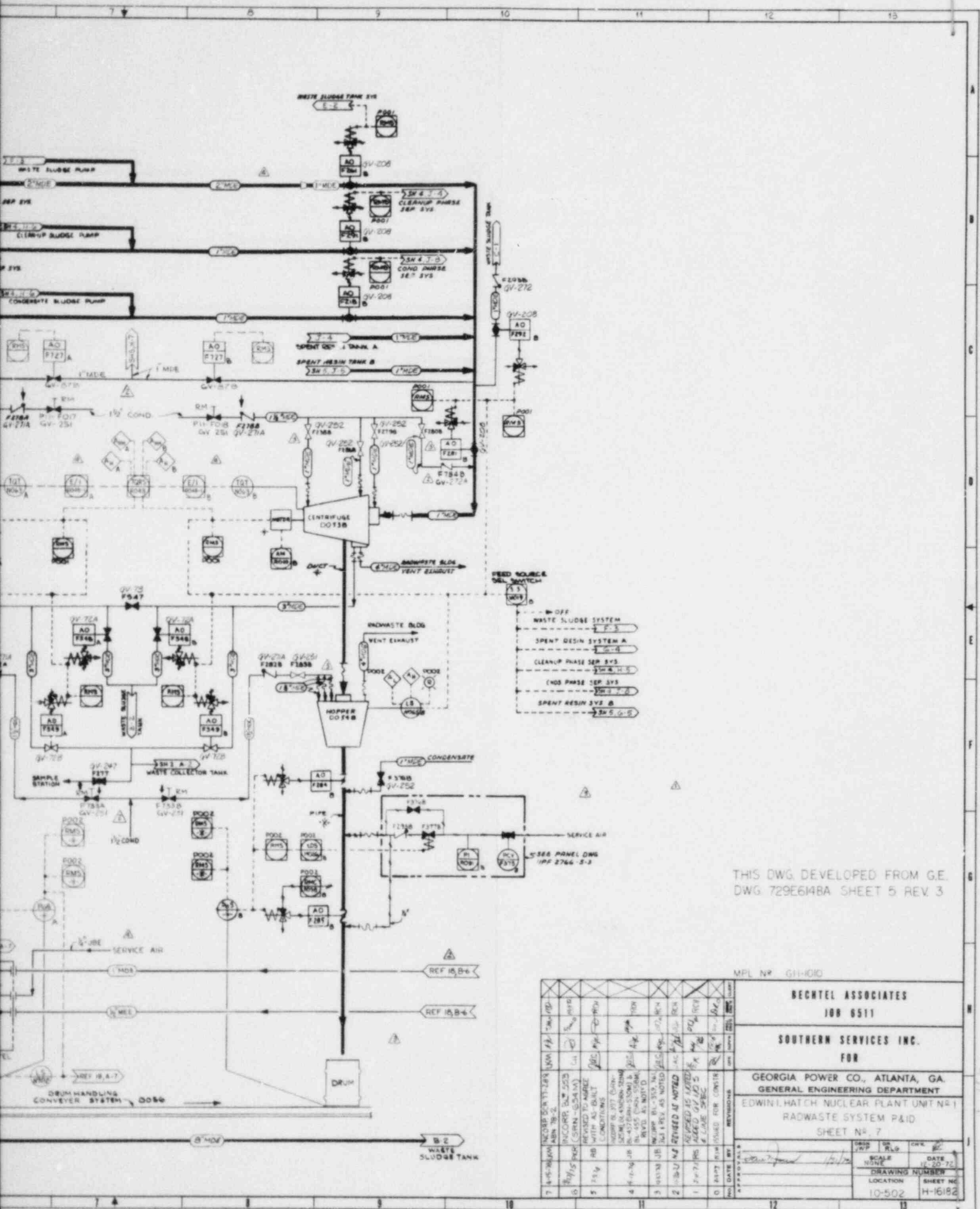
SHEET NO. 1

NO.	DATE	BY	REVISIONS
1	10-5-76	JTB	ISSUED FOR CONSTRUCTION
2	10-5-76	JTB	REV. AS NOTED ABOVE
3	10-5-76	JTB	REV. AS NOTED ABOVE
4	10-5-76	JTB	REV. AS NOTED ABOVE
5	10-5-76	JTB	REV. AS NOTED ABOVE
6	10-5-76	JTB	REV. AS NOTED ABOVE
7	10-5-76	JTB	REV. AS NOTED ABOVE
8	10-5-76	JTB	REV. AS NOTED ABOVE
9	10-5-76	JTB	REV. AS NOTED ABOVE
10	10-5-76	JTB	REV. AS NOTED ABOVE
11	10-5-76	JTB	REV. AS NOTED ABOVE
12	10-5-76	JTB	REV. AS NOTED ABOVE
13	10-5-76	JTB	REV. AS NOTED ABOVE

SCALE	DATE
AS SHOWN	10-5-76
DRAWING NUMBER	10-502
LOCATION	H-10-02
SHEET NO.	H-10-02







- ON OFF
- WASTE SLODGE SYSTEM
- SPENT RESIN SYSTEM A
- CLEANUP PHASE SEP SYS
- COND PHASE SEP SYS
- SPENT RESIN SYS B

THIS DWG DEVELOPED FROM G.E. DWG 729E6M4BA SHEET 5 REV 3

MPL NO G11-010

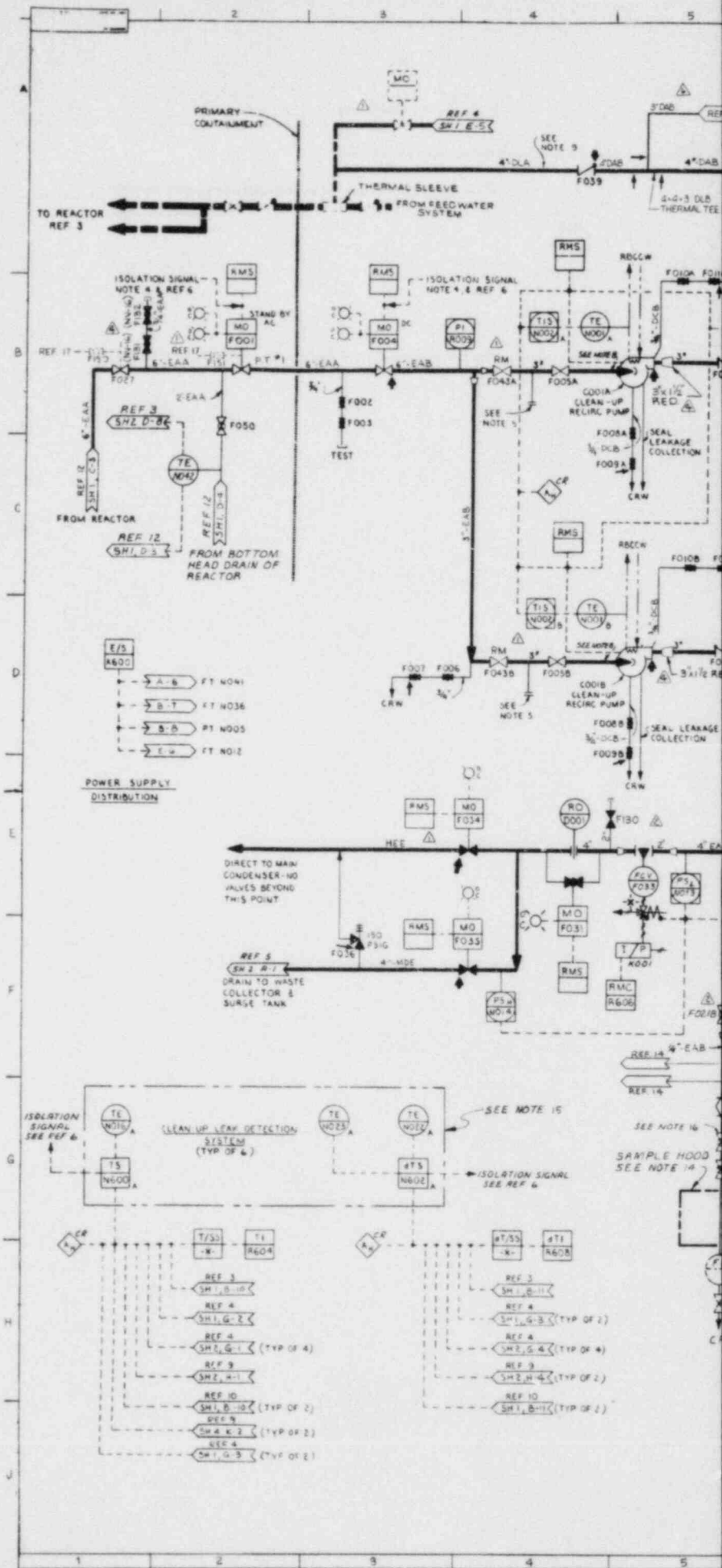
**BECHTEL ASSOCIATES**  
100 6511

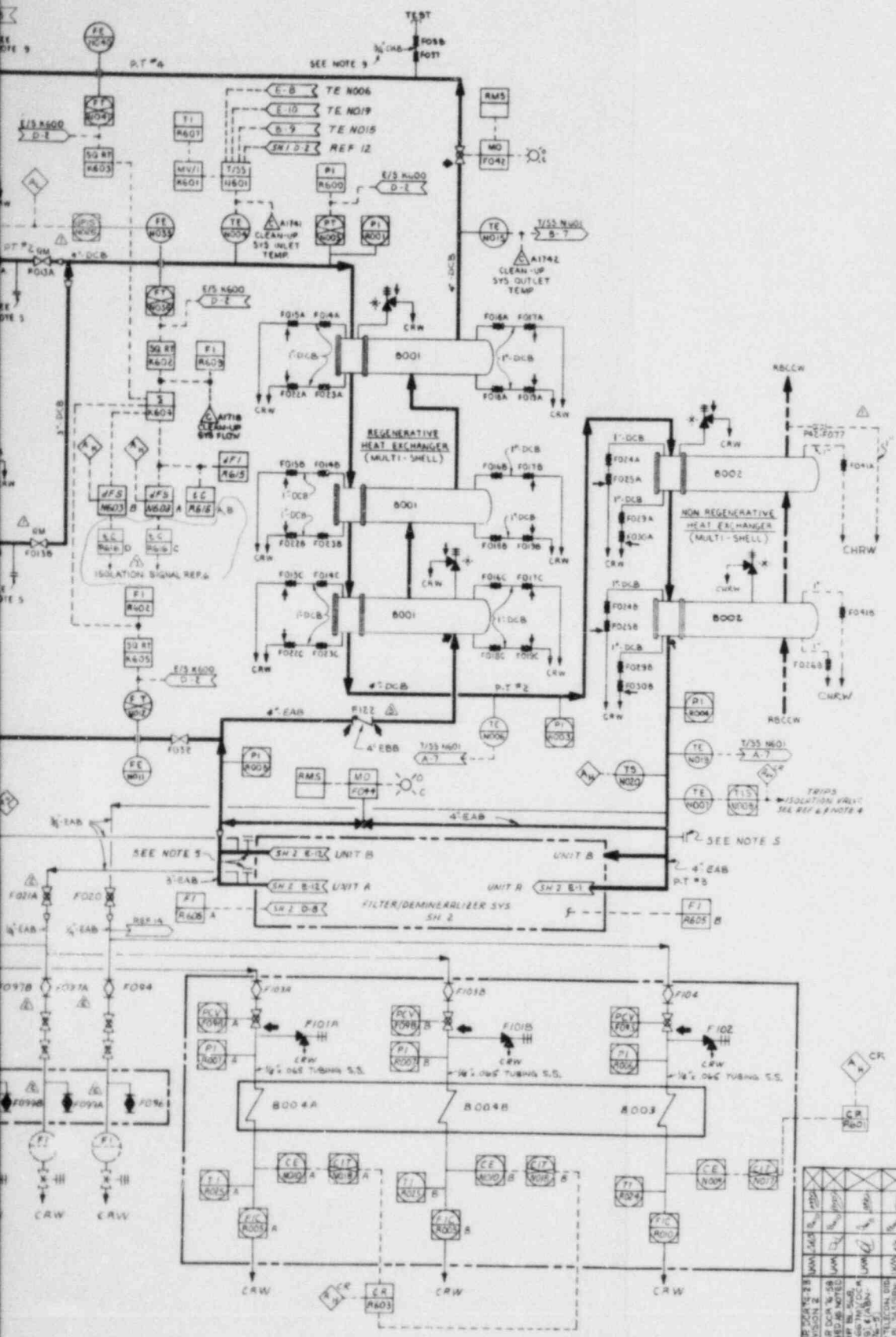
**SOUTHERN SERVICES INC.**  
FOR

**GEORGIA POWER CO., ATLANTA, GA.**  
GENERAL ENGINEERING DEPARTMENT  
EDWIN HATCH NUCLEAR PLANT UNIT NO. 1  
RADWASTE SYSTEM P4ID  
SHEET NO. 7

NO.	DATE	BY	REVISIONS
7	4-15-72	WJL	REVISION 1: 1. 4-15-72
6	4-15-72	WJL	REVISION 2: 1. 4-15-72
5	4-15-72	WJL	REVISION 3: 1. 4-15-72
4	4-15-72	WJL	REVISION 4: 1. 4-15-72
3	4-15-72	WJL	REVISION 5: 1. 4-15-72
2	4-15-72	WJL	REVISION 6: 1. 4-15-72
1	4-15-72	WJL	REVISION 7: 1. 4-15-72
0	4-15-72	WJL	REVISION 8: 1. 4-15-72

SCALE	DATE
10-502	4-20-72
LOCATION	SHEET NO.
	H-16182





- REFERENCES
- | NO. | DESCRIPTION                                               | REV.    | DATE     |
|-----|-----------------------------------------------------------|---------|----------|
| 1.  | REACTOR WATER CLEAN-UP SYS P&ID                           | 001-000 | 02/01/62 |
| 2.  | PLANT DESIGN INDEX                                        | 001-000 |          |
| 3.  | NUCLEAR BOILER SYSTEM P & ID                              | 001-1   | 001-001  |
| 4.  | REC SYSTEM P & ID                                         | 001-1   | 001-001  |
| 5.  | ROBUSTIC P & ID                                           | 001-1   | 001-001  |
| 6.  | REACTOR WATER CLEAN-UP SYSTEM T&I                         | 001-000 | 02/01/62 |
| 7.  | PUMP & INSTRUMENT SYMBOLS                                 | 001-000 | 02/01/62 |
| 8.  | PROCESS INSTRUMENT SYMBOLS & T&I                          | 001-000 | 02/01/62 |
| 9.  | REC SYSTEM P & ID                                         | 001-1   | 001-001  |
| 10. | REC SYSTEM P & ID                                         | 001-1   | 001-001  |
| 11. | PRESSURE INTENSITY OF PIPING AND EQUIPMENT PRESSURE PARTS | 001-000 | 02/01/62 |
| 12. | REACTOR BOILER SYSTEM P & ID                              | 001-1-1 | 001-001  |
| 13. | REACTOR WATER CLEAN-UP SYSTEM SPEC                        | 001-000 | 02/01/62 |
| 14. | LIQUID SAMPLING SYS                                       | P33-200 | H-15281  |
| 15. | CLOUD STORAGE TRANS SYS                                   | PI-1001 | H-15016  |
| 16. | FUEL POLE F&ID/EMERGENCY SYS                              | H-16003 | H-16003  |
| 17. | CONTROL VALVE HEADUP DRAINAGE SYS P&ID                    | H-16019 | H-16019  |
| 18. | CONTROL VALVE CRUISE SYS                                  | CI-1000 | H-16024  |

- NOTES
- ALL EQUIPMENT AND INSTRUMENTS ARE PROVIDED BY MFLING, UNLESS OTHERWISE NOTED.
  - WHEAT OR NUMBERS ARE SHOWN THE VALVES ARE TAGGED WITH THESE NUMBERS. WHERE COUPLERS ARE NOT SHOWN, THE VALVES ARE TAGGED WITH THE MFLING NUMBER.
  - REACTOR WATER CLEAN-UP SYSTEM DRAIN TO CRW, CFW OR CFW WHERE APPLICABLE.
  - REACTOR WATER CLEAN-UP SYSTEM DRAIN TO CRW OR CFW OF THE FOLLOWING DRAINAGE:
    - (a) HIGH TEMP FOLLOWING HIGH TEMP HEAT EXCHANGER (ONLY FOOD)
    - (b) TRIP/STOP VALVE CONTROL VALVE (ACTUATOR ONLY) FOOD
    - (c) LOW REACTOR WATER LEVEL
    - (d) HIGH TEMPERATURE DETECTION
    - (e) HIGH FLOW/LEAK DETECTION
    - (f) HIGH PRESSURE TEMP - LEAK DETECTION
  - GENERAL CLEANING AND INFORMATION CONNECTIONS SHALL BE PROVIDED TO THE SYSTEM OPERATOR. CONNECTIONS SHALL BE ARRANGED TO PROVIDE DECONTAMINATION OF THE FACE OF EQUIPMENT SEPARATELY FROM ALL OTHER EQUIPMENT, I.E. FILTER-DEMINERALIZER SHALL BE PROVIDED SEPARATELY FROM THE FILTER-DEMINERALIZER (HEAT EXCHANGER ETC.).
  - NON-REGENERATIVE HEAT EXCHANGER SHALL HAVE A PIPING DESIGN TO MEET SYSTEM CONDITIONS.
  - HIGH POINT VENTS AND LOW POINT DRAINS ARE TO BE PROVIDED WHERE NECESSARY AS PROVIDED BY PHYSICAL ROUTING OF PIPE.
  - TEMPORARY STAINLESS STEEL SHALL BE PROVIDED ON THE SECTION SIDE OF ALL PIPING IN ACCORDANCE WITH REFERENCE 1, SECTION 5.
  - PIPING TO BE SCHEDULED FOR LOW PRESSURE & TEMP AS MAIN FEED PIPING.
  - FOR LOCATION AND IDENTIFICATION OF INSTRUMENTS, SEE INSTRUMENT DATA SHEET LISTED IN MFL FOR EACH INSTRUMENT.
  - WHERE TWO VALVES ARE INDICATED IN A DOUBLE BLOCK ARRANGEMENT ON VENTS AND DRAINS, BOTH VALVES SHALL BE THE SAME RATING AND COMPATIBLE WITH UPSTREAM SYSTEM SERVICE SPECIFICATIONS.
  - AIR OPERATED VALVES SHOWN IN FILTERING PROCESS STATUS, ARE FAIL CLOSE ON LOSS OF AIR PRESSURE TO VALVE OPERATOR OR LOSS OF ELECTRICAL POWER TO THE VALVE PILOT, EXCEPT HOLDING VALVE F071 WHICH FAILS OPEN.
  - USE NORMAL AC POWER FOR ELECTRICAL DEVICES AND ELECTRIC POWERED VALVES UNLESS OTHERWISE NOTED.
  - THE SAMPLE HOODS SHALL CONTAIN THE TERMINAL SAMPLE HOOD FOR THE SAMPLES. AIR SHALL FLOW INTO THE SAMPLE HOODS AND SHALL EFFECTIVELY PROTECT PERSONNEL FROM RADIATION.
  - TWO SENSOR SYSTEMS PER ROOM.
  - SAMPLE STATIONS MUST COMPLY WITH WATER SAMPLING REQUIREMENTS (SEE REF 2, SECT B). EQUIPMENT SHOWN IN PARANTHESIS IS SUGGESTED DESIGN.

INDEX	DESIGN		PEAK		MIN	
	PSIG	°F	PSIG	°F	PSIG	°F
1	150	542	137	502	70	70
2	150	542	150	542		
3	150	150	150	150		
4	(SEE NOTE 9)		442	442		
5	150	150	150	150		

THIS DWG. DEVELOPED FROM GE DWG. NO 751E995 SH. 1, REV 5  
MPL NR 631-1010

BECHTEL ASSOCIATES  
JOB 6511

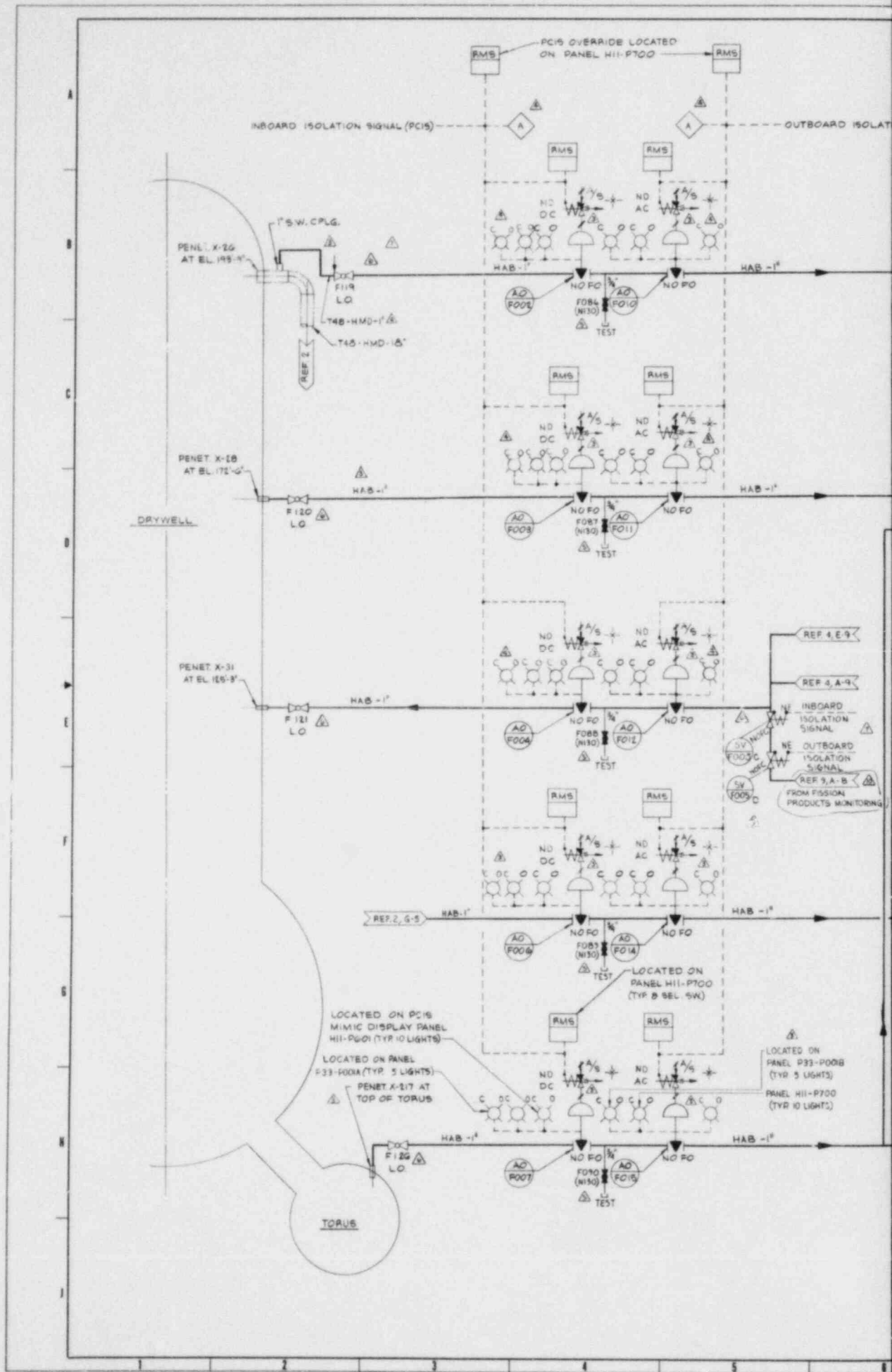
SOUTHERN SERVICES INC.  
FOR

GEORGIA POWER CO., ATLANTA, GA.  
GENERAL ENGINEERING DEPARTMENT

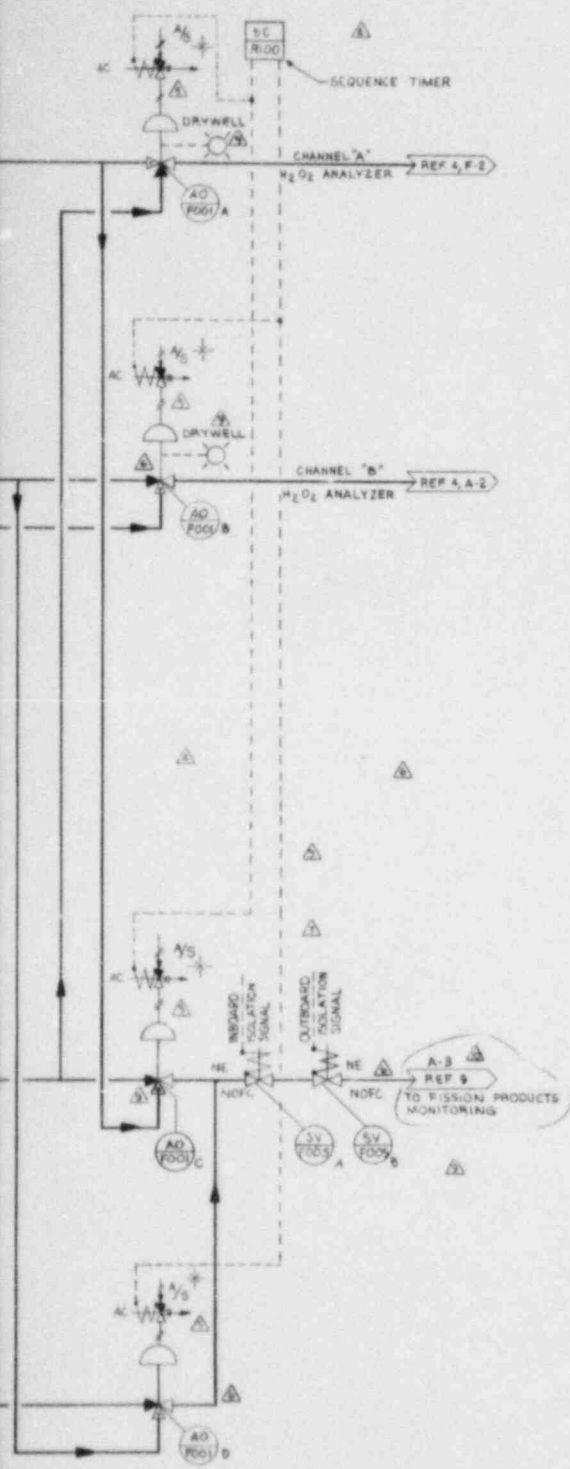
EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1  
REACTOR WATER CLEAN-UP SYSTEM  
P&ID SHEET 1

DATE	BY	SCALE	OR	DATE
10/25/62	JMM		OR	10/25/62
DRAWING NUMBER				
LOCATION				
SHEET NO.				

NO.	DATE	BY	REVISION
1	10/25/62	JMM	ISSUED FOR CONSTRUCTION
2	11/15/62	JMM	REVISIONS AS NOTED
3	12/10/62	JMM	REVISIONS AS NOTED
4	1/15/63	JMM	REVISIONS AS NOTED
5	2/15/63	JMM	REVISIONS AS NOTED
6	3/15/63	JMM	REVISIONS AS NOTED
7	4/15/63	JMM	REVISIONS AS NOTED
8	5/15/63	JMM	REVISIONS AS NOTED
9	6/15/63	JMM	REVISIONS AS NOTED
10	7/15/63	JMM	REVISIONS AS NOTED
11	8/15/63	JMM	REVISIONS AS NOTED
12	9/15/63	JMM	REVISIONS AS NOTED



IN SIGNAL (PC/4)



**NOTES**

1. ALL EQUIPMENT & INSTRUMENT NUMBERS ARE TO BE PRECEDED BY MPL P33 UNLESS OTHERWISE NOTED. EXAMPLE: P33-A001.
2. REDUNDANT MODES:
  - A) ENERGIZE FOO1 A,C; DE-ENERGIZE FOO1 B,D
  - B) ENERGIZE FOO1 B; DE-ENERGIZE FOO1 A,C
3. OPERATION: MODES WILL ALTERNATE AT 24HR INTERVALS
4. SOLENOID VALVES FOO1 A,B,C,D SHOULD BE LOCATED AS CLOSE AS POSSIBLE TO THE H<sub>2</sub>O<sub>2</sub> ANALYZER AND FISSION PRODUCTS MONITORING PANELS.
5. RETURN LINES SHOULD "TEE" TOGETHER AS CLOSE AS POSSIBLE TO THE H<sub>2</sub>O<sub>2</sub> ANALYZER AND FISSION PRODUCTS MONITORING PANELS.
6. ROUTING OF THESE GAS SAMPLE LINES IS AS SHOWN ON REFERENCES 5,6,7,8.

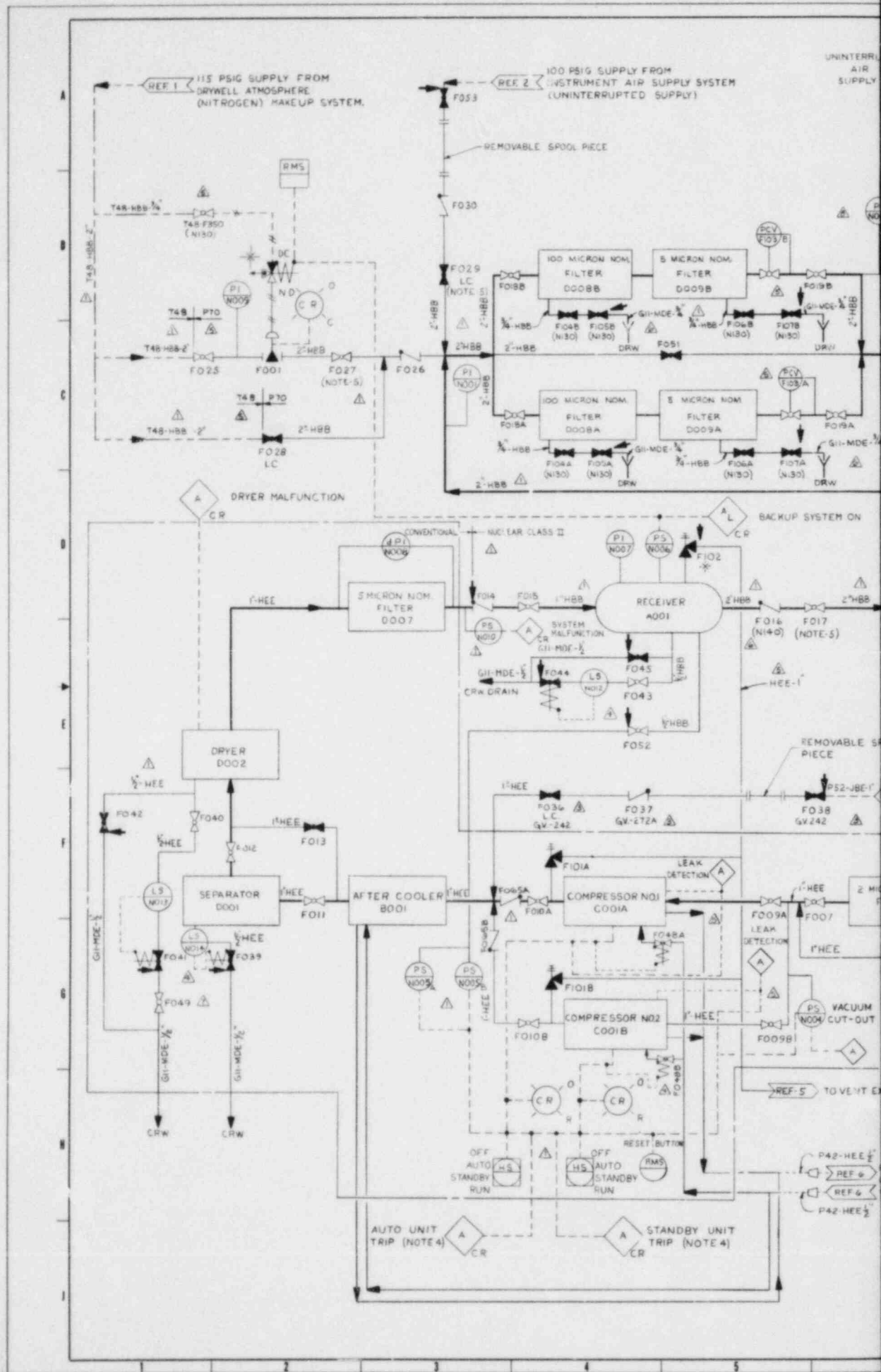
**REFERENCES**

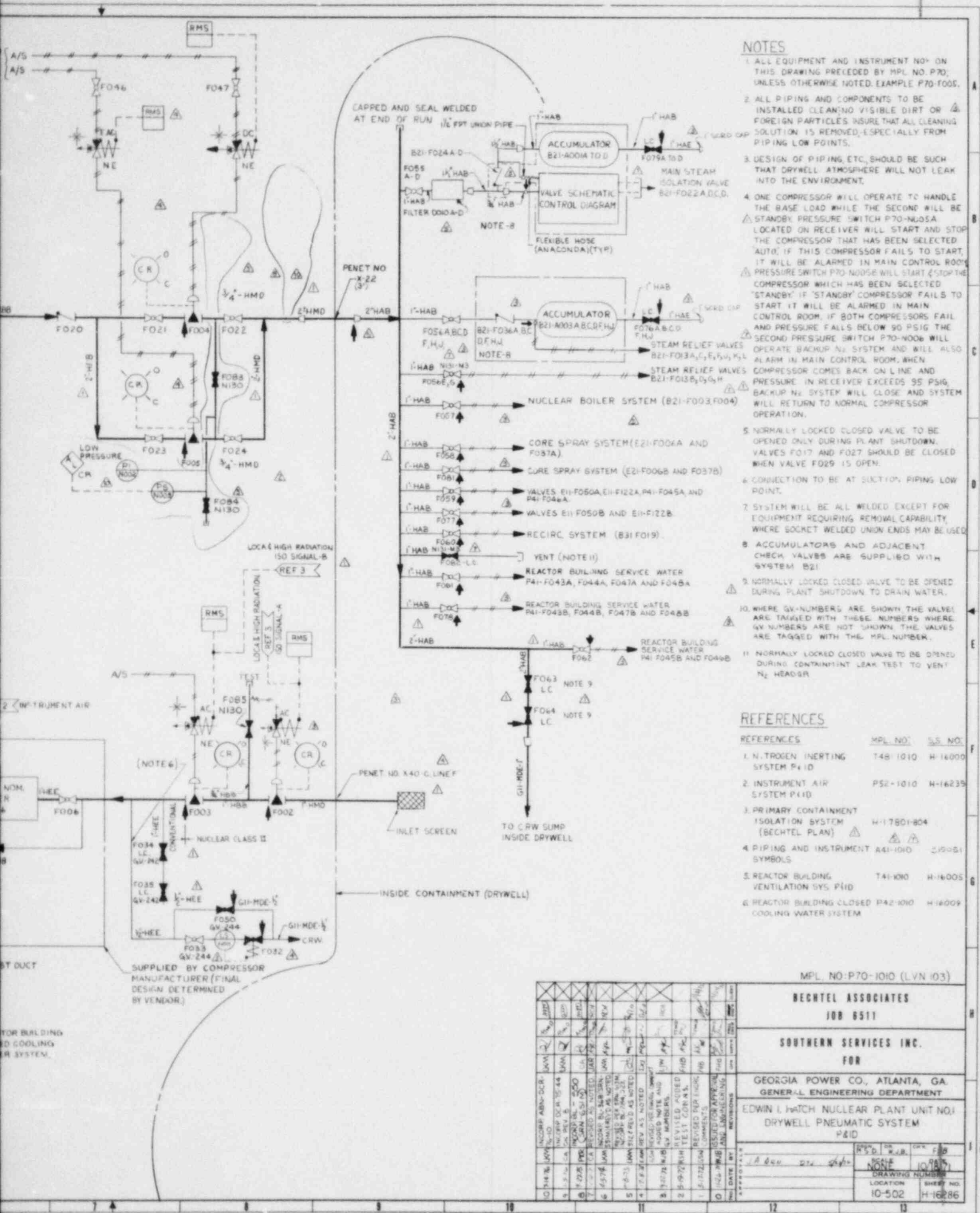
REFERENCE	MPL NO	S.S.N#
1. PIPING & INSTRUMENT SYMBOLS	A44-1010	S-16091
2. PRIMARY CONT. PURGE T4B-1020 INERTING SYSTEM	T4B-1020	H-16024
3. PRIMARY CONT. FISSION PRODUCTS MONITORING SYS	D11-1010, -1011	SX-16927
4. H <sub>2</sub> O <sub>2</sub> ANALYZER SYSTEM P33-1010 P410 FOR PANEL NOS. P33-P001A & B	P33-1010	H-16280
5. SAMPLE LINE ROUTINGS REACTOR & RADW. BLDGS. BELOW EL. 130'-0"		H-16553
6. SAMPLE LINE ROUTINGS REACTOR & RADW. BLDGS. EL. 130'-0"		H-16554
7. SAMPLE LINE ROUTINGS REACTOR & RADW. BLDGS. EL. 158'-0"		H-16556
8. SAMPLE LINE ROUTINGS REACTOR & RADW. BLDGS. EL. 165'-0"		H-16557
9. FISSION PRODUCTS MONITORING SYS. P41.D		H-16274

MPL NO P33-1010

BECHTEL ASSOCIATES 100 6511	
SOUTHERN SERVICES INC. FOR	
GEORGIA POWER CO., ATLANTA, GA. GENERAL ENGINEERING DEPARTMENT	
EDWIN I HATCH NUCLEAR PLANT UNIT NO. 1 PRIMARY CONTAINMENT ATMOSPHERE H <sub>2</sub> O <sub>2</sub> ANALYZER SYSTEM P & I.D. SHEET NO. 1	
NO. DATE BY	REV. NO. DATE BY
10 10 76 JMM INCORP DCR NTER	1 10 76 JMM
9 10 76 JMM	2 10 76 JMM
8 10 76 JMM	3 10 76 JMM
7 10 76 JMM	4 10 76 JMM
6 10 76 JMM	5 10 76 JMM
5 10 76 JMM	6 10 76 JMM
4 10 76 JMM	7 10 76 JMM
3 10 76 JMM	8 10 76 JMM
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0 10 76 JMM	11 10 76 JMM
12 10 76 JMM	13 10 76 JMM
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18 10 76 JMM	19 10 76 JMM
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52 10 76 JMM	53 10 76 JMM
54 10 76 JMM	55 10 76 JMM
56 10 76 JMM	57 10 76 JMM
58 10 76 JMM	59 10 76 JMM
60 10 76 JMM	61 10 76 JMM
62 10 76 JMM	63 10 76 JMM
64 10 76 JMM	65 10 76 JMM
66 10 76 JMM	67 10 76 JMM
68 10 76 JMM	69 10 76 JMM
70 10 76 JMM	71 10 76 JMM
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74 10 76 JMM	75 10 76 JMM
76 10 76 JMM	77 10 76 JMM
78 10 76 JMM	79 10 76 JMM
80 10 76 JMM	81 10 76 JMM
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84 10 76 JMM	85 10 76 JMM
86 10 76 JMM	87 10 76 JMM
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92 10 76 JMM	93 10 76 JMM
94 10 76 JMM	95 10 76 JMM
96 10 76 JMM	97 10 76 JMM
98 10 76 JMM	99 10 76 JMM
100 10 76 JMM	101 10 76 JMM

DRAWING NUMBER  
 10-502  
 SHEET NO.  
 H16276





**NOTES**

1. ALL EQUIPMENT AND INSTRUMENT NO. ON THIS DRAWING PRECEDED BY MPL NO. P70, UNLESS OTHERWISE NOTED. EXAMPLE P70-FO05.
2. ALL PIPING AND COMPONENTS TO BE INSTALLED CLEAN, NO VISIBLE DIRT OR FOREIGN PARTICLES. INSURE THAT ALL CLEANING SOLUTION IS REMOVED, ESPECIALLY FROM PIPING LOW POINTS.
3. DESIGN OF PIPING, ETC., SHOULD BE SUCH THAT DRYWELL ATMOSPHERE WILL NOT LEAK INTO THE ENVIRONMENT.
4. ONE COMPRESSOR WILL OPERATE TO HANDLE THE BASE LOAD WHILE THE SECOND WILL BE STANDBY. PRESSURE SWITCH P70-NU05A LOCATED ON RECEIVER WILL START AND STOP THE COMPRESSOR THAT HAS BEEN SELECTED. AUTO. IF THIS COMPRESSOR FAILS TO START, IT WILL BE ALARMED IN MAIN CONTROL ROOM. PRESSURE SWITCH P70-NU05B WILL START THE COMPRESSOR WHICH HAS BEEN SELECTED. 'STANDBY' IF 'STANDBY' COMPRESSOR FAILS TO START IT WILL BE ALARMED IN MAIN CONTROL ROOM. IF BOTH COMPRESSORS FAIL AND PRESSURE FALLS BELOW 90 PSIG THE SECOND BACKUP SWITCH P70-NU06 WILL OPERATE. BACKUP N<sub>2</sub> SYSTEM AND WILL ALSO ALARM IN MAIN CONTROL ROOM. WHEN COMPRESSOR COMES BACK ON LINE AND PRESSURE IN RECEIVER EXCEEDS 95 PSIG, BACKUP N<sub>2</sub> SYSTEM WILL CLOSE AND SYSTEM WILL RETURN TO NORMAL COMPRESSOR OPERATION.
5. NORMALLY LOCKED CLOSED VALVE TO BE OPENED ONLY DURING PLANT SHUTDOWN. VALVES FO17 AND FO27 SHOULD BE CLOSED WHEN VALVE FO29 IS OPEN.
6. CONNECTION TO BE AT SUCTION, PIPING LOW POINT.
7. SYSTEM WILL BE ALL WELDED EXCEPT FOR EQUIPMENT REQUIRING REMOVAL CAPABILITY, WHERE SOCKET WELDED UNION ENDS MAY BE USED.
8. ACCUMULATORS AND ADJACENT CHECK VALVES ARE SUPPLIED WITH SYSTEM B21.
9. NORMALLY LOCKED CLOSED VALVE TO BE OPENED DURING PLANT SHUTDOWN TO DRAIN WATER.
10. WHERE GV-NUMBERS ARE SHOWN THE VALVE ARE TAGGED WITH THESE NUMBERS. WHERE GV NUMBERS ARE NOT SHOWN THE VALVES ARE TAGGED WITH THE MPL NUMBER.
11. NORMALLY LOCKED CLOSED VALVE TO BE OPENED DURING CONTAINMENT LEAK TEST TO VENT N<sub>2</sub> HEADS.

**REFERENCES**

REFERENCES	MPL NO.	U.S. NO.
1. NITROGEN INERTING SYSTEM P410	T48-1010	H-16000
2. INSTRUMENT AIR SYSTEM P410	P52-1010	H-16235
3. PRIMARY CONTAINMENT ISOLATION SYSTEM (BECHTEL PLAN)	H-17801-804	
4. PIPING AND INSTRUMENT SYMBOLS	P41-1010	S19-051
5. REACTOR BUILDING VENTILATION SYS. P410	T41-1010	H-16005
6. REACTOR BUILDING CLOSED PA2-1010 COOLING WATER SYSTEM	H16009	H-16009

MPL NO. P70-1010 (LVN 103)

**BECHTEL ASSOCIATES**  
JOB 8511

**SOUTHERN SERVICES INC.**  
FOR

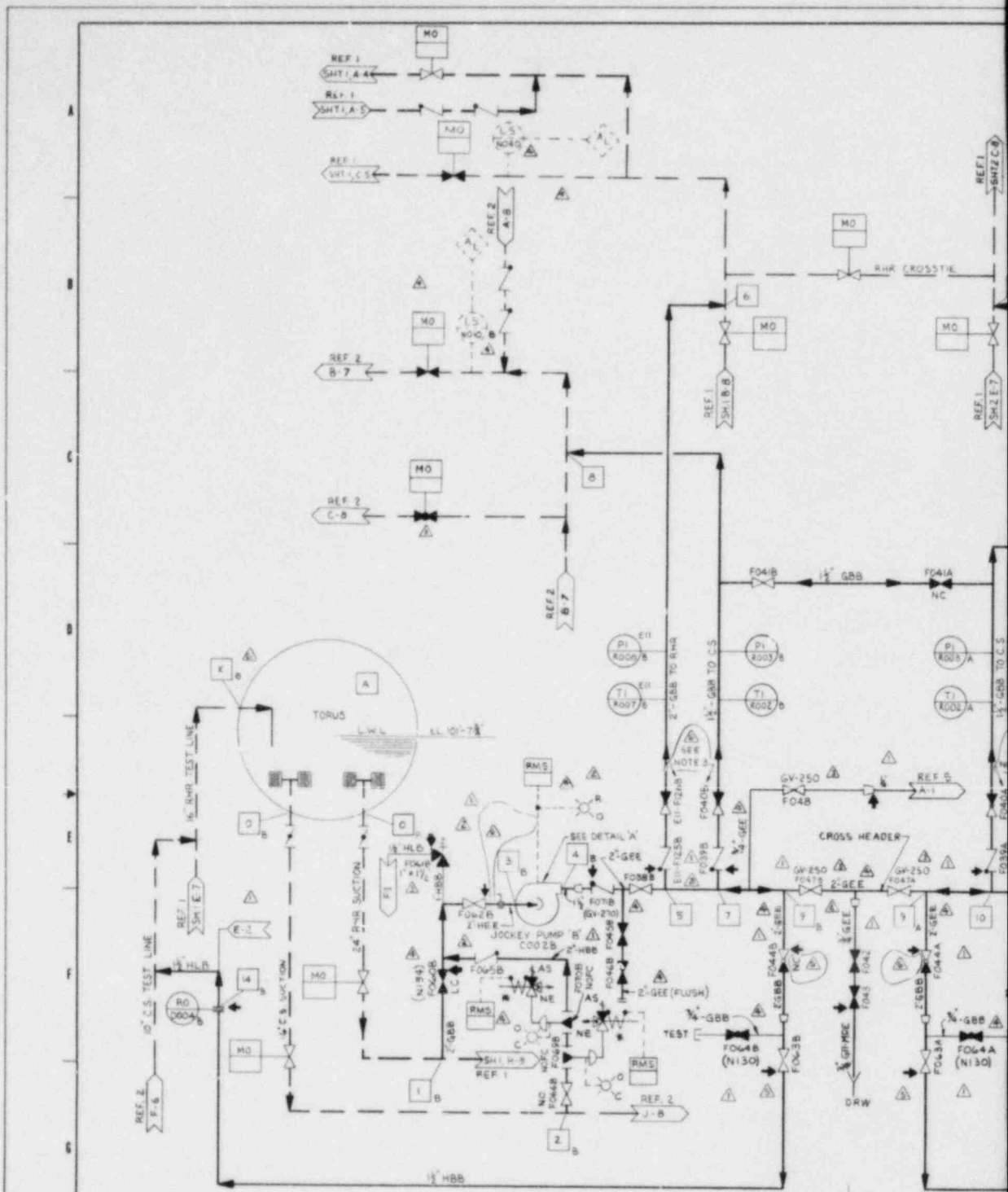
**GEORGIA POWER CO., ATLANTA, GA.**  
GENERAL ENGINEERING DEPARTMENT

**EDWIN L. WATCH NUCLEAR PLANT UNIT NO. 1**  
DRYWELL PNEUMATIC SYSTEM  
P410

NO.	DATE	BY	REVISIONS
1	JAN 21 1970	WMB	ISSUED FOR APPROVAL
2	FEB 10 1970	WMB	REVISED FOR APPROVAL
3	MAR 10 1970	WMB	REVISED FOR APPROVAL
4	APR 10 1970	WMB	REVISED FOR APPROVAL
5	MAY 10 1970	WMB	REVISED FOR APPROVAL
6	JUN 10 1970	WMB	REVISED FOR APPROVAL
7	JUL 10 1970	WMB	REVISED FOR APPROVAL
8	AUG 10 1970	WMB	REVISED FOR APPROVAL
9	SEP 10 1970	WMB	REVISED FOR APPROVAL
10	OCT 10 1970	WMB	REVISED FOR APPROVAL
11	NOV 10 1970	WMB	REVISED FOR APPROVAL
12	DEC 10 1970	WMB	REVISED FOR APPROVAL

APPROVED BY	DATE	BY
WMB	12/10/70	WMB
WMB	12/10/70	WMB
WMB	12/10/70	WMB
WMB	12/10/70	WMB

DRAWING NUMBER	10-502
SHEET NO.	H-16286



REACTOR BLDG. N.E. CORNER EQUIP. RM.      REACTOR BLDG. S.E. CORNER EQUIP. RM.

MODE A - FULL FLOW TO RHR PUMPS DISCHARGE LEG (FAR SIDE)

POSITION	A	0 <sub>A</sub>	1 <sub>A</sub>	0 <sub>A</sub>	2 <sub>A</sub>	3 <sub>A</sub>	4 <sub>A</sub>	5	6	7	9 <sub>A</sub>	10	12	X <sub>A</sub>
FLOW - GPM	-	0	40					30		40				10
PRESSURE - PSIA	14.7													
TEMP. - °F (MAX)	200													200
MAX. PRESSURE DROP - FEET														

REQD TDH = 121.5 FT

MODE B - FULL FLOW TO C.S. PUMP DISCHARGE LEG

POSITION	A	0 <sub>A</sub>	1 <sub>A</sub>	0 <sub>A</sub>	2 <sub>A</sub>	3 <sub>A</sub>	4 <sub>A</sub>	5
FLOW - GPM	-	0	40					
PRESSURE - PSIA	14.7							
TEMP. - °F (MAX)	200							
MAX. PRESSURE DROP - FEET								

REQD TDH = 126 FT



**MODES**

MODE A - FULL FLOW FOR EXCESS AMOUNT OF LEAKAGE AT CHECK VALVE SEATS E11-F031A OR C (REF. 1, SH.1&2).  
 MODE B - FULL FLOW FOR EXCESS AMOUNT OF LEAKAGE AT CHECK VALVE SEAT E21-F003A (REF. 2).  
 MODE C - NORMAL OPERATION, SYSTEM PRESSURIZED, MIN. FLOW RECIRCULATION.

**GENERAL NOTES**

- ALL EQUIPMENT AND INSTRUMENT NUMBERS ARE TO BE PRECEDED BY MPL-E21 UNLESS OTHERWISE NOTED. EXAMPLE E21-C002A.
- FOR WEIGHT & MATERIAL OF PIPE, VALVES & FITTINGS SEE MATERIAL SPEC 65-6903-1.
- ALL CHECK & GLOBE STOP CHECK VALVES MUST BE LOCATED CLOSE TO THE CROSS HEADER.
- JOCKEY PUMP DRAINS & VENTS TO DRW.
- PUMP C002A IS RUNNING & C002B IS ON STANDBY.
- ALL HIGH POINT VENTS ARE X AND ALL LOW POINT DRAINS ARE I UNLESS NOTED OTHERWISE.

**PROCESS FLOW NOTES**

- FOR PUMP NPSH<sub>AVAIL</sub> CALCULATIONS:  
 MAX. TORUS WATER TEMP. = 200°F  
 TORUS PRESS. = 14.7 PSIA
- MIN NPSH<sub>AVAIL</sub> CALCULATED @ 7 FT.
- VALVES WILL BE THROTTLED TO MAINTAIN DESIGN FLOWS.
- WHEN THE CORE SPRAY SYSTEM IS AT TEST MODE THEN SUCTION TO JOCKEY PUMP WILL BE THROUGH [1]a, [1]b, [1]c.
- THE JOCKEY PUMPS ARE TO PROVIDE A MINIMUM OF 10 PSIG PRESSURE IN THE MAIN PUMP DISCH. LEGS IN ADDITION TO HEAD PRESSURE FROM WATER COLUMN.
- THE RESTRICTING ORIFICES ARE DESIGNED TO DROP THE DISCH HEAD TO A LOW TORUS PRESSURE (15.7 PSIA).
- RELIEF VALVES F01A & B SET PRESSURE SHALL BE 100 PSIG.
- CAP SUCTION LINE FROM RHR SUCTION AND SPOOL THROUGH SUCTION LINE FROM CORE SPRAY SUCTION UNTIL VALVES F06D A&B, F06E A&B, AND F06G A & B ARE DELIVERED.
- PUMPS C002A & B SHALL OPERATE ON EMERGENCY A-C PUMPS.

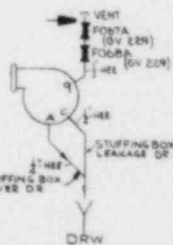
**REFERENCES**

REFERENCE	MPL NO.	S.S.I. NO.
1. RHR SYS PSD	E11-1010 SH.1 SH.2	H-16329 H-16330
2. CORE SPRAY SYS PSD	E21-1010	H-16331
3. RHR SYS PROCESS DIA.	E11-1020	S-15324-15328
4. CORE SPRAY PROCESS DIA.	E21-1020	S-15117
5. SAMPLING SYSTEM PSD (P/D)	F93-1010	H-16281
6. PUMP SEAL DWS.	E21-C002A&B	SX-15040
7. JOCKEY PUMP NSTR. MANUAL	E21-C002A&B	SX-15110
8. JOCKEY PUMP OUTLINE	E21-C002A&B	SX-15109
9. PIPING & INST SYMBOLS	441-1010	S-15051

**LEGEND**

- EXISTING PIPING FOR E11-1010 AND/OR E21-1010.
- PIPING FOR JOCKEY PUMP SYSTEM.

**DETAIL "A"**  
 VENT & DRAIN CONNECTIONS (TYP 2 PLACES)



AR SIDE

T	B	9A	10	12	XA
			40		10
					200°

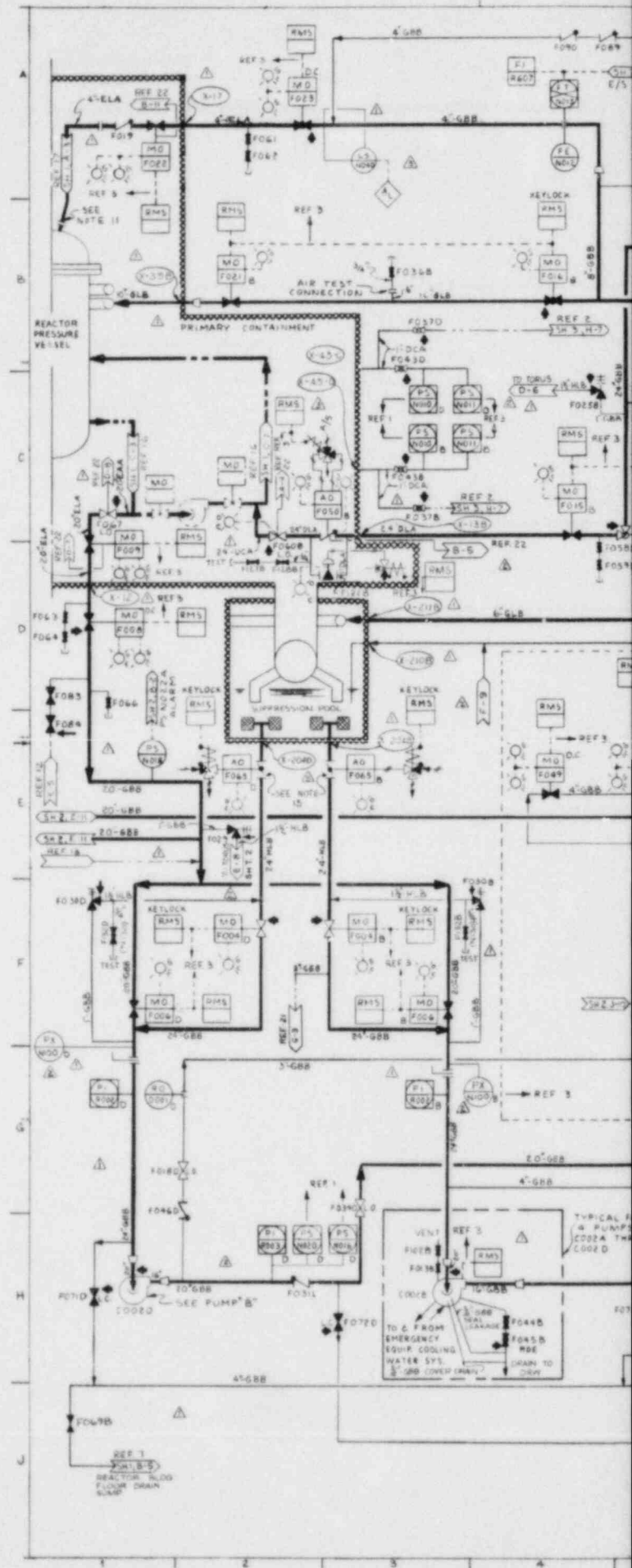
MODE C - RELING TO TORUS (NO FLOW TO PUMP DISCH. LEGS) NEAR SIDE

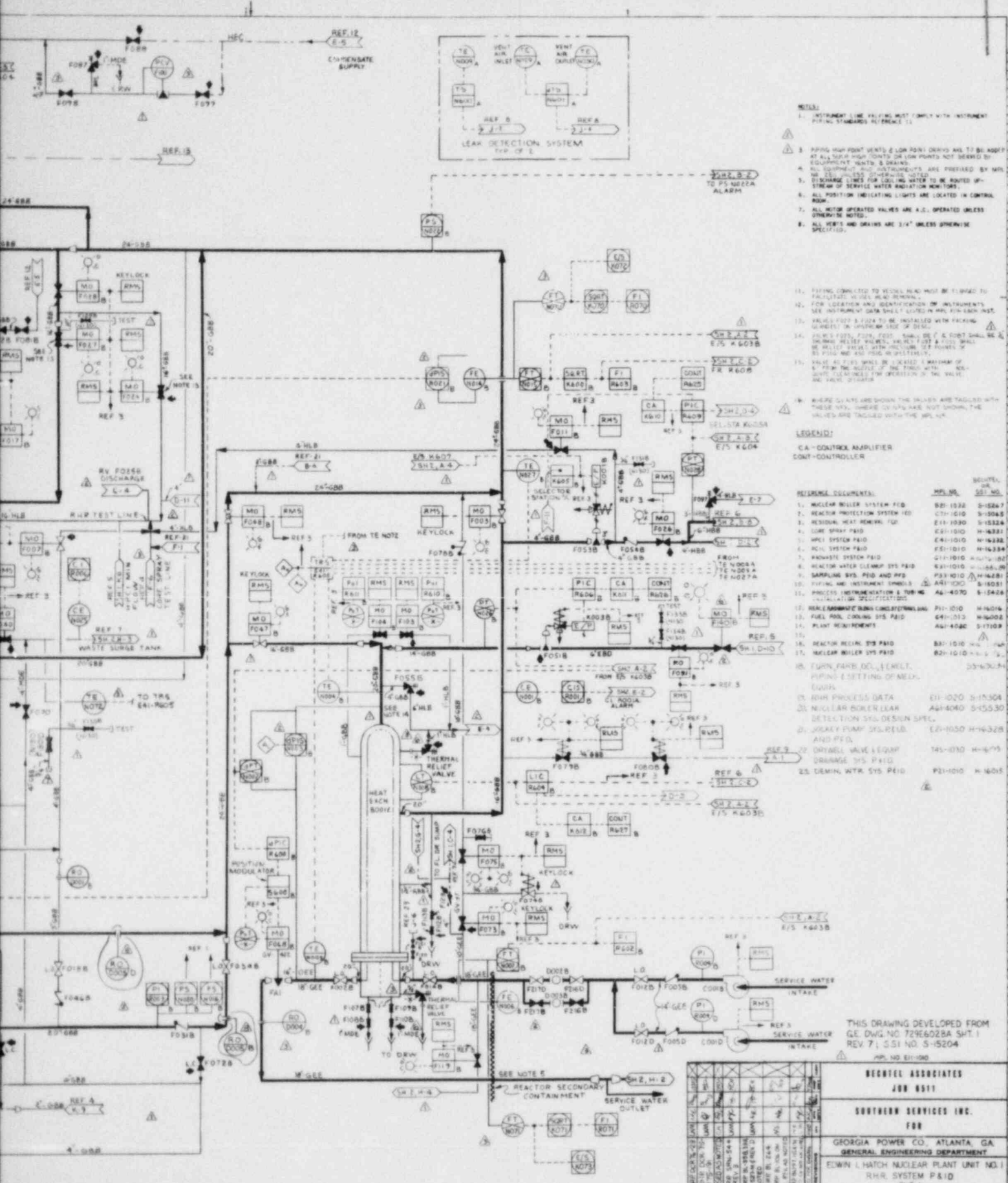
POSITION	A	O <sub>A</sub>	I <sub>A</sub>	O <sub>A</sub>	2A	3A	4A	9A	10	12	14A	X <sub>A</sub>
FLOW - GPM.			0		10							
PRESSURE - PSIA	14.7											15.7
TEMP. °F (MAX)	200°											200°
MAX. PRESSURE DROP - FEET												TCH = 150 FT

NO.	DATE	BY	REVISION
1	12/27/73	...	...
2	...	...	...
3	...	...	...
4	...	...	...
5	...	...	...
6	...	...	...

MPL NO. E21-1050	LVN-F126
<b>BECHTEL ASSOCIATES</b> JOB 8511	
<b>SOUTHERN SERVICES INC.</b> FOR	
<b>GEORGIA POWER CO., ATLANTA, GA.</b> GENERAL ENGINEERING DEPARTMENT	
<b>EDWIN I HATCH NUCLEAR PLANT UNIT NO. 1</b> JOCKEY PUMP SYSTEM P&ID & PROCESS FLOW DIAGRAM	
DESIGN BY: <i>[Signature]</i>	DATE: 11/1/73
SCALE: AS SHOWN	DATE: 11/1/73
DRAWING NUMBER: 10-5C2	SHEET NO. H-16328

Microfilmed 9-9-7





- NOTES:**
1. INSTRUMENT LINE VALVES MUST COMPLY WITH INSTRUMENT PIPING STANDARDS REFERENCE 11.
  2. PIPING HIGH POINT VENTS & LOW POINT DRAINS ARE TO BE ADDED AT ALL HIGH POINTS AND LOW POINTS NOT SHOWN BY EQUIPMENT VENTS & DRAINS.
  3. ALL EQUIPMENT AND INSTRUMENTS ARE PROVIDED BY MFR. NO. IS TO BE SHOWN OTHERWISE NOTED.
  4. EXCHANGE LINES FOR LOADING WATER TO BE ROUTED UPSTREAM OF SERVICE WATER RADIATION MONITORS.
  5. ALL POSITION INDICATING LIGHTS ARE LOCATED IN CONTROL ROOM.
  6. ALL MOTOR OPERATED VALVES ARE A.C. OPERATED UNLESS OTHERWISE NOTED.
  7. ALL VENTS AND DRAINS ARE 3/4" UNLESS OTHERWISE SPECIFIED.

11. PIPING CONNECTED TO REEL HEAD MUST BE TURNED TO FACILITATE REEL HEAD REMOVAL.
12. FOR LOCATION AND IDENTIFICATION OF INSTRUMENTS SEE INSTRUMENT DATA SHEET LOCATED IN RHR FOR EACH INST.
13. VALVES F007 & F024 TO BE INSTALLED WITH PACKING GUNBELT ON UPSTREAM SIDE OF DISK.
14. VALVE F027, F028, F029, F030, F031 & F032 SHALL BE 4" OPENING PRESSURE VALVES. VALVE F031 & F032 SHALL BE WELDED VALVES WITH PULLERS OFF POINTS OF 83.5 INCH AND 105.0 INCH RESPECTIVELY.
15. VALVE F010 SHALL BE LOCATED 1/2 WAY FROM 5' FROM THE MIDDLE OF THE TROUGH WITH 100% GRADE CLEARANCE FROM OBSTRUCTION OF THE VALVE AND VALVE STUBS.

WHERE GYANTS ARE SHOWN THE VALVES ARE TAGGED WITH THESE TAGS. WHERE GYANTS ARE NOT SHOWN THE VALVES ARE TAGGED WITH THE MPL 4/4.

**LEGEND:**  
 CA - CONTROL AMPLIFIER  
 CONT - CONTROLLER

**REFERENCE DOCUMENTS:**

REF. NO.	DESCRIPTION	REV. NO.	DATE
1.	NUCLEAR BOILER SYSTEM P&ID	82-1032	5-15-67
2.	REACTOR PROTECTION SYSTEM P&ID	87-1010	5-15-68
3.	RESIDUAL HEAT REMOVAL P&ID	87-1030	5-15-68
4.	LOW DRAIN P&ID	82-1010	5-15-67
5.	WPT SYSTEM P&ID	87-1010	5-15-68
6.	ACU SYSTEM P&ID	82-1010	5-15-67
7.	NUCLEAR SYSTEM P&ID	81-1010	5-15-67
8.	REACTOR WATER CLAMP SYS P&ID	81-1010	5-15-67
9.	SAMPLING SYS. P&ID AND P&ID	83-1010	5-15-68
10.	PIPING AND INSTRUMENT SYMBOLS	84-1010	5-15-68
11.	PROCESS INSTRUMENTATION & TUBING INSTRUMENTATION SPECIFICATION	84-1010	5-15-68
12.	NUCLEAR BOILER (BURNS) (CONSTRUCTION)	81-1010	5-15-67
13.	FUEL POOL COOLING SYS P&ID	84-1010	5-15-68
14.	PLANT REQUIREMENTS	84-1010	5-15-68
15.	REACTOR HEAT EXCH. P&ID	82-1010	5-15-67
16.	NUCLEAR BOILER SYS P&ID	82-1010	5-15-67

18. TURN P&ID, DCL, ERECT, PIPING, ERECTING OF REEL.
19. REEL PROCESS DATA 87-1020 5-15-68
20. NUCLEAR BOILER CLEAR DETECTION SKD DESIGN SPEC. 84-1010 5-15-68
21. JOCKEY PUMP SKD P&ID AND P&ID 82-1010 5-15-67
22. DRIVEL VALVE EQUIP DRAINAGE SYS P&ID 74-1010 5-15-67
23. DEMIN. WTR. SYS P&ID 82-1010 5-15-67

THIS DRAWING DEVELOPED FROM GE DWG NO. 7296028A SHT. 1 REV. 7; SSI NO. 5-15204  
 RPL. NO. 81-1010

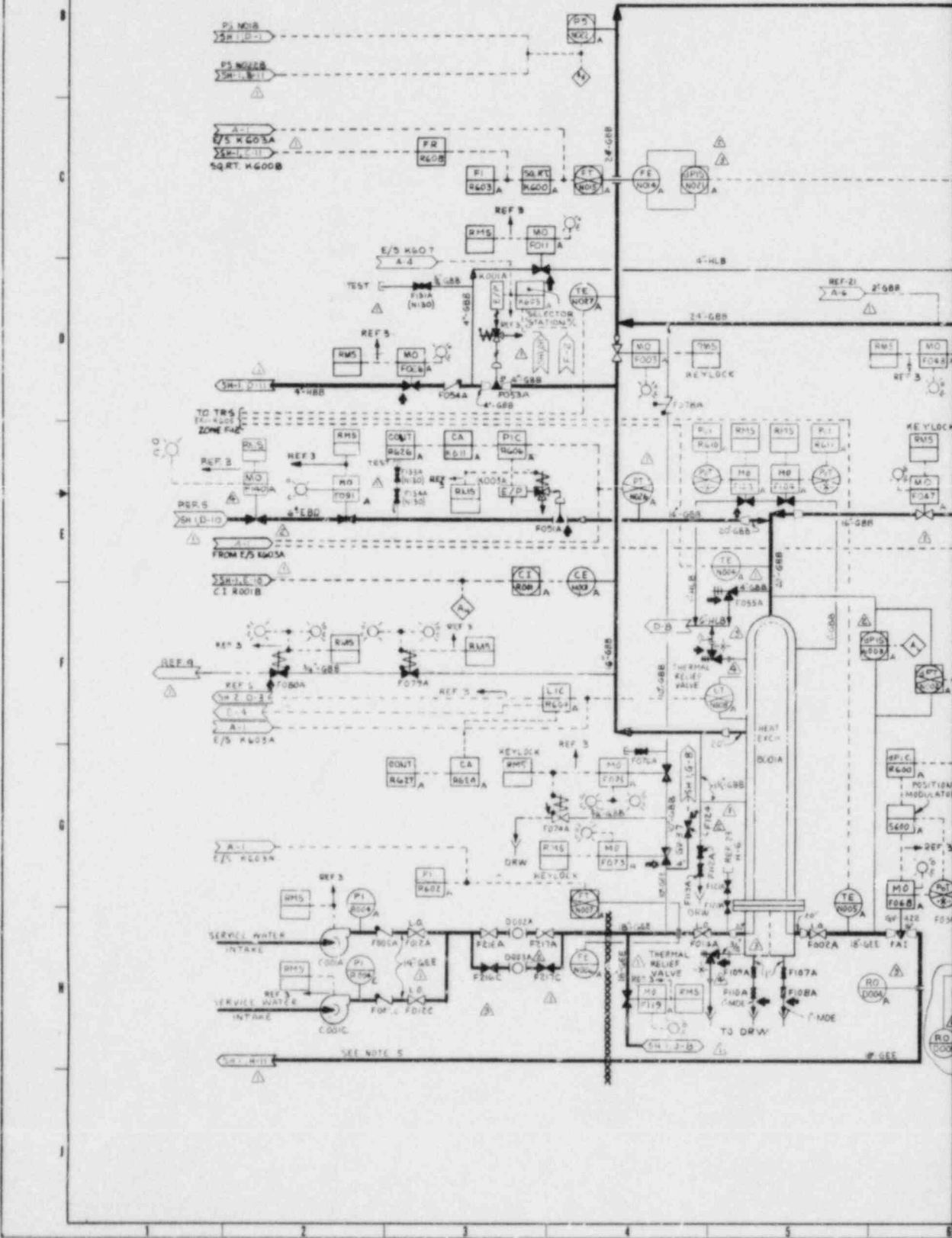
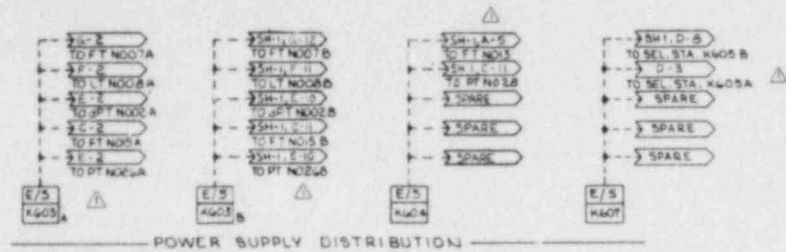
**RECORD ASSOCIATED**  
 JOB 6511

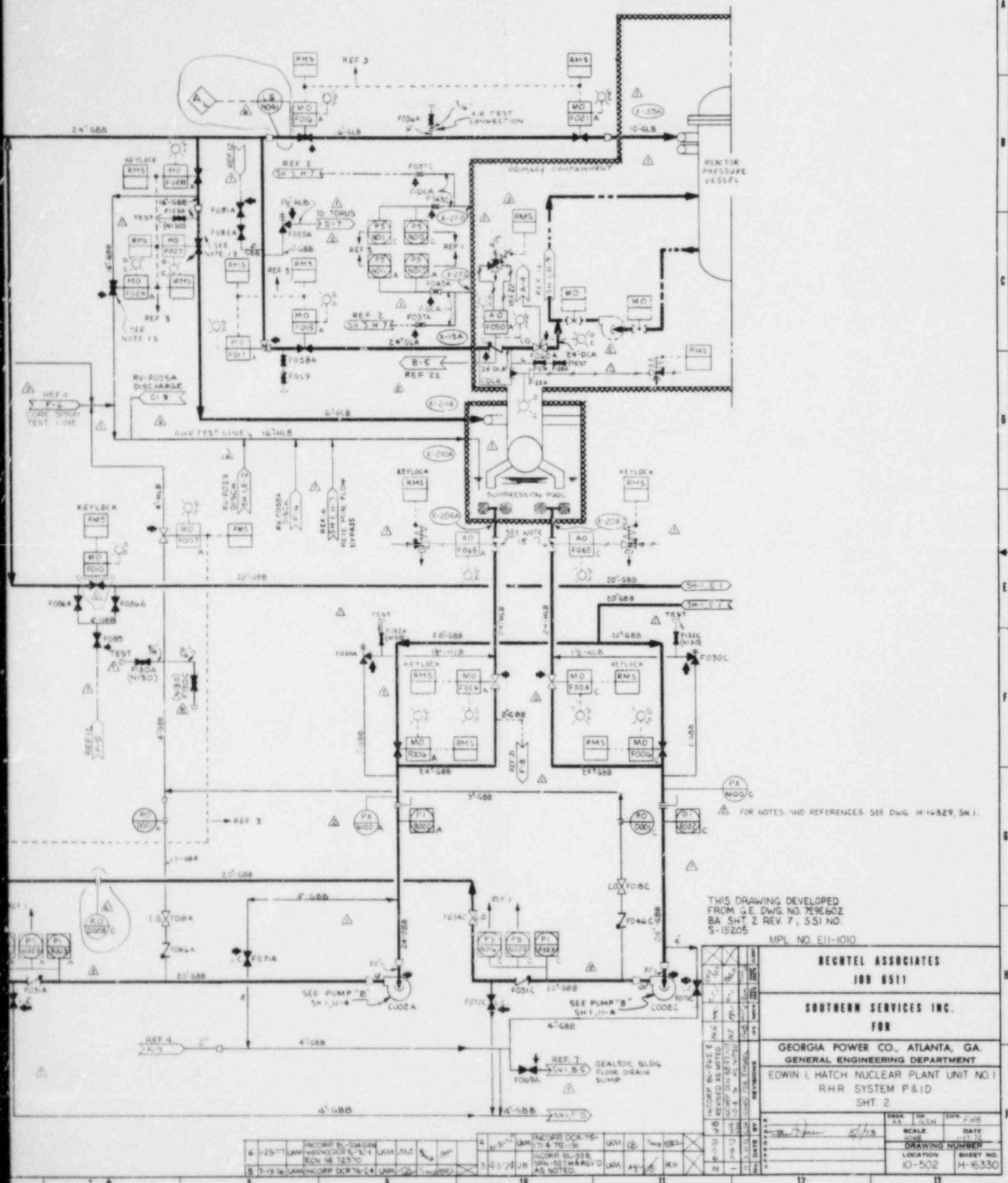
**SOUTHERN SERVICES INC.**  
 FOR

GEORGIA POWER CO., ATLANTA, GA.  
 GENERAL ENGINEERING DEPARTMENT

EDWIN L. HATCH NUCLEAR PLANT UNIT NO. 1  
 RHR SYSTEM P&ID  
 SHT. 7

REVISED PER	DATE	BY	REVISIONS
1	10-1-67	JMB	ISSUED FOR CONSTRUCTION
2	10-1-67	JMB	ISSUED FOR CONSTRUCTION
3	10-1-67	JMB	ISSUED FOR CONSTRUCTION
4	10-1-67	JMB	ISSUED FOR CONSTRUCTION
5	10-1-67	JMB	ISSUED FOR CONSTRUCTION
6	10-1-67	JMB	ISSUED FOR CONSTRUCTION
7	10-1-67	JMB	ISSUED FOR CONSTRUCTION
8	10-1-67	JMB	ISSUED FOR CONSTRUCTION
9	10-1-67	JMB	ISSUED FOR CONSTRUCTION
10	10-1-67	JMB	ISSUED FOR CONSTRUCTION

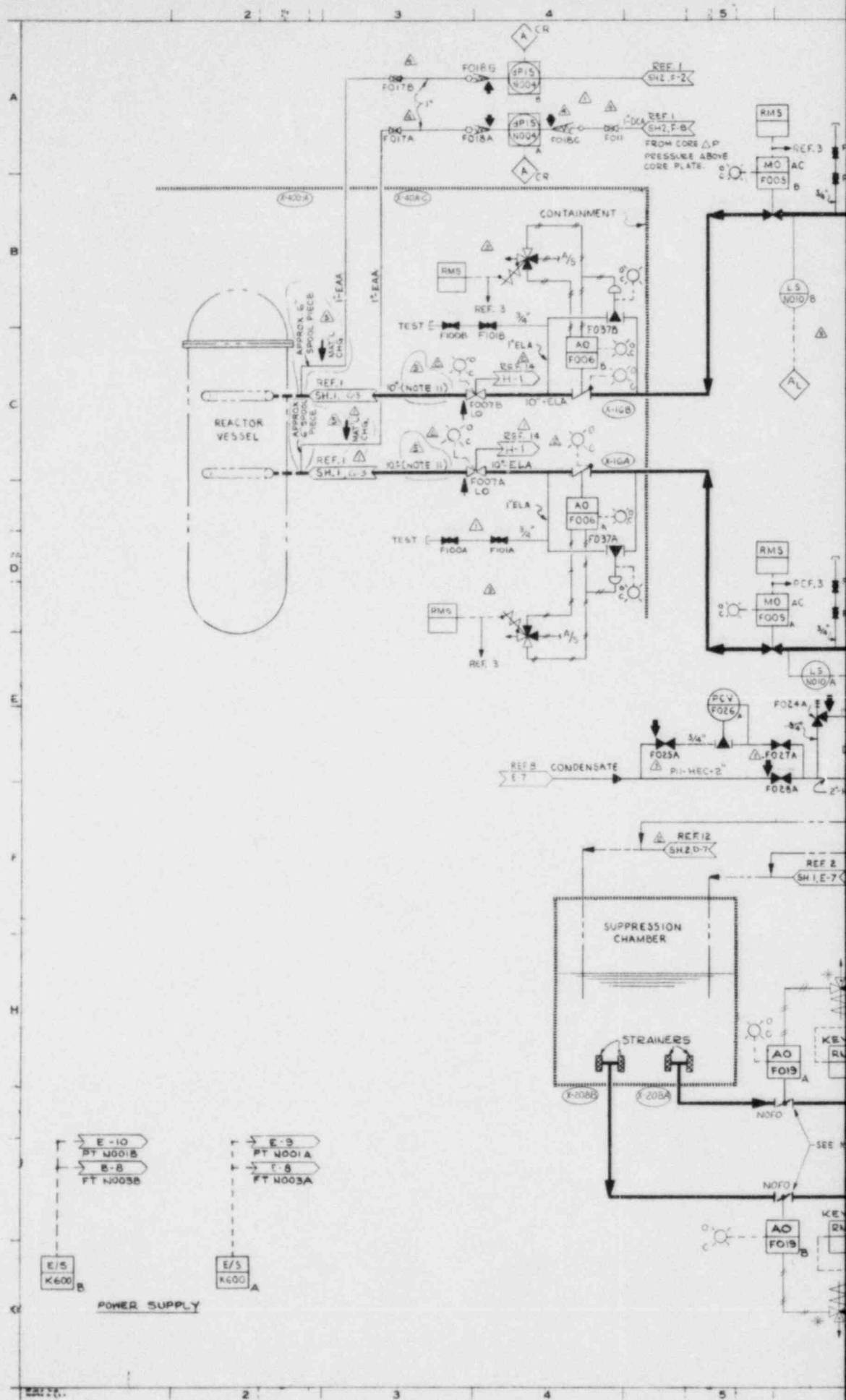




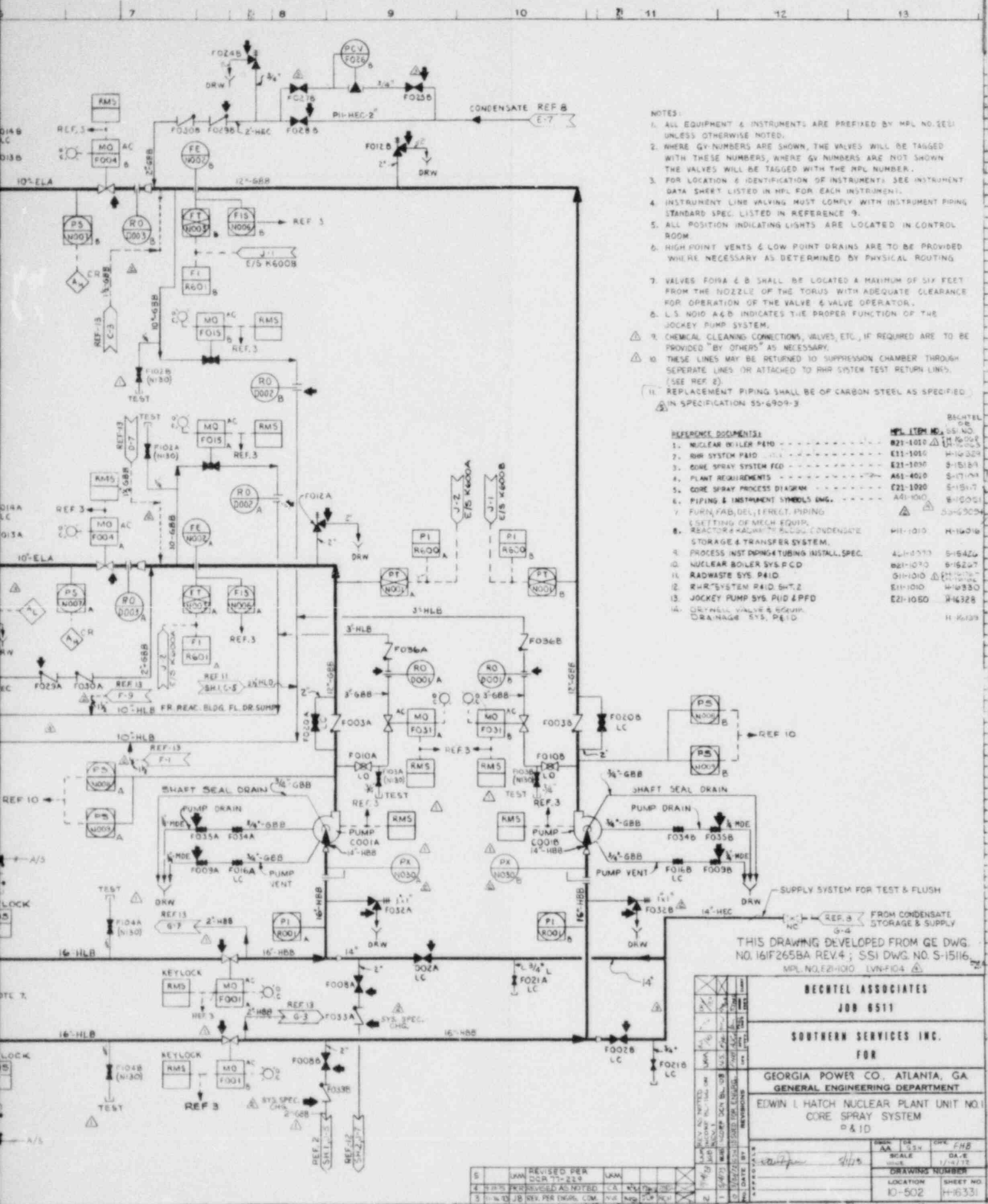
THIS DRAWING DEVELOPED FROM G.E. DWG. NO. 729E602 BA SHT 2 REV. 7, SSI NO. S-15205 MPL NO. E11-1010

<p>RECNET ASSOCIATES JOB 8511</p>			
<p>SOUTHERN SERVICES INC. FOR</p>			
<p>GEORGIA POWER CO., ATLANTA, GA. GENERAL ENGINEERING DEPARTMENT</p>			
<p>EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1 RHR SYSTEM P&amp;ID SHT. 2</p>			
<p>NO. 1</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 2</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 3</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 4</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 5</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 6</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 7</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 8</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 9</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 10</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 11</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 12</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 13</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
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<p>NO. 22</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 23</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
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<p>NO. 28</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 29</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
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<p>NO. 31</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
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<p>NO. 35</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 36</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 37</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 38</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 39</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 40</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 41</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 42</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 43</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
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<p>NO. 46</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 47</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 48</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 49</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>
<p>NO. 50</p>	<p>DATE</p>	<p>SCALE</p>	<p>DATE</p>

10-502 H-6330



POWER SUPPLY



- NOTES:
1. ALL EQUIPMENT & INSTRUMENTS ARE PREFIXED BY MPL NO. 2001 UNLESS OTHERWISE NOTED.
  2. WHERE GV NUMBERS ARE SHOWN, THE VALVES WILL BE TAGGED WITH THESE NUMBERS, WHERE GV NUMBERS ARE NOT SHOWN THE VALVES WILL BE TAGGED WITH THE MPL NUMBER.
  3. FOR LOCATION & IDENTIFICATION OF INSTRUMENTS SEE INSTRUMENT DATA SHEET LISTED IN MPL FOR EACH INSTRUMENT.
  4. INSTRUMENT LINE VALVING MUST COMPLY WITH INSTRUMENT PIPING STANDARD SPEC LISTED IN REFERENCE 9.
  5. ALL POSITION INDICATING LIGHTS ARE LOCATED IN CONTROL ROOM.
  6. HIGH POINT VENTS & LOW POINT DRAINS ARE TO BE PROVIDED WHERE NECESSARY AS DETERMINED BY PHYSICAL ROUTING.
  7. VALVES FO19A & B SHALL BE LOCATED A MAXIMUM OF SIX FEET FROM THE NOZZLE OF THE TORUS WITH ADEQUATE CLEARANCE FOR OPERATION OF THE VALVE & VALVE OPERATOR.
  8. L.S. NO10 A&B INDICATES THE PROPER FUNCTION OF THE JOCKEY PUMP SYSTEM.
  9. CHEMICAL CLEANING CONNECTIONS, VALVES, ETC., IF REQUIRED ARE TO BE PROVIDED "BY OTHERS" AS NECESSARY.
  10. THESE LINES MAY BE RETURNED TO SUPPRESSION CHAMBER THROUGH SEPARATE LINES OR ATTACHED TO RHR SYSTEM TEST RETURN LINES. (SEE REF 2)
  11. REPLACEMENT PIPING SHALL BE OF CARBON STEEL AS SPECIFIED IN SPECIFICATION 55-6909-3.

REFERENCE DOCUMENTS:

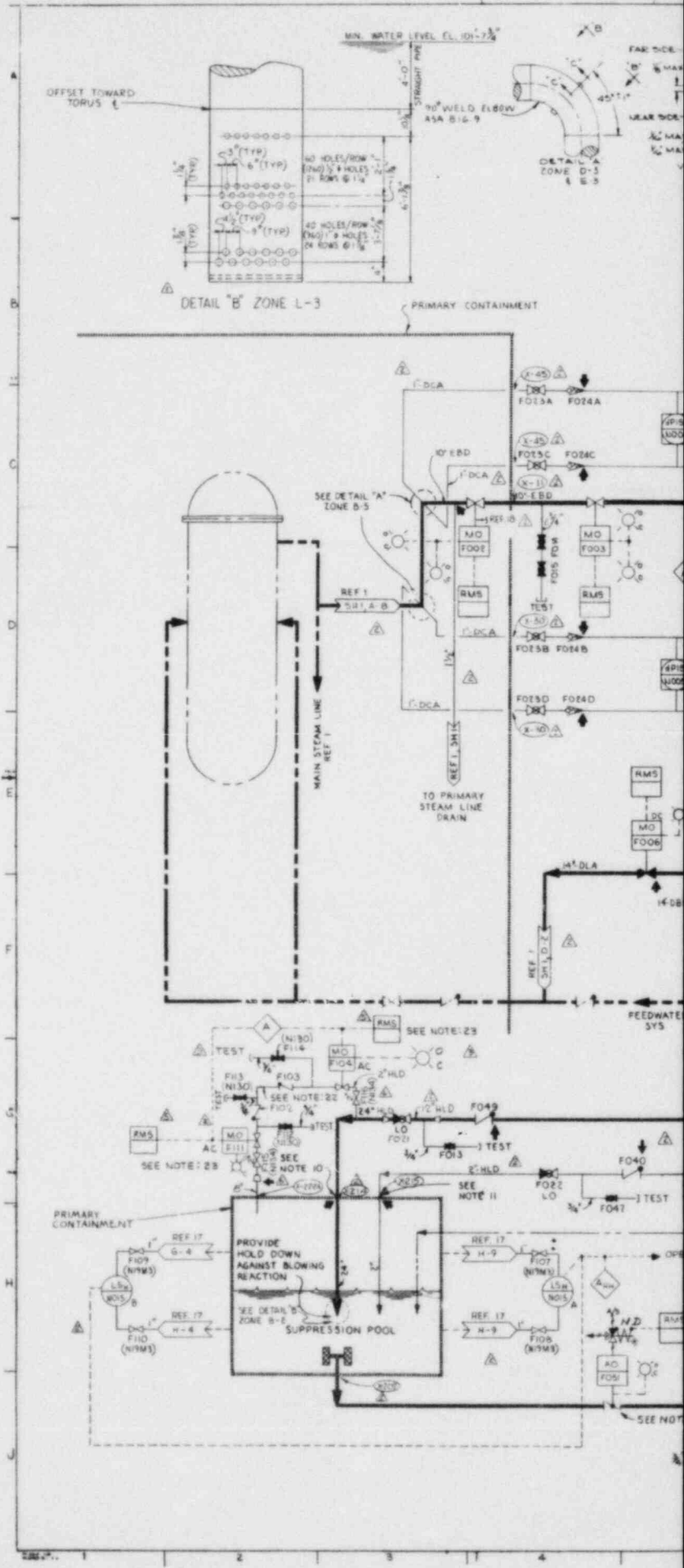
REF.	DESCRIPTION	MPL ITEM NO.	REV. NO.	DATE
1.	NUCLEAR BOILER PAID	E21-1010	1	11-10-70
2.	RHR SYSTEM PAID	E11-1010	1	11-10-70
3.	CORE SPRAY SYSTEM FCO	E21-1010	1	11-10-70
4.	PLANT REQUIREMENTS	A61-4010	1	11-10-70
5.	CORE SPRAY PROCESS DIAGRAM	E21-1010	1	11-10-70
6.	PIPING & INSTRUMENT SYMBOLS ENG.	A41-1010	1	11-10-70
7.	FURN. FAB. DEL. I RECT. PIPING	53-6909	1	11-10-70
8.	SETTING OF MECH EQUIP. REACTOR KALADRY BLDG. CONDENSATE STORAGE & TRANSFER SYSTEM.	111-1010	1	11-10-70
9.	PROCESS INST DRNG & TUBING INSTAL. SPEC.	41-1010	1	11-10-70
10.	NUCLEAR BOILER SYS PCD	601-1010	1	11-10-70
11.	RADWASTE SYS PAID	511-1010	1	11-10-70
12.	RHR SYSTEM PAID SH-2	511-1010	1	11-10-70
13.	JOCKEY PUMP SYS PAID & PFD	E21-1010	1	11-10-70
14.	DRY-CELL VALVE & GROUP DRAINAGE SYS. PAID	H-1010	1	11-10-70

THIS DRAWING DEVELOPED FROM GE DWG. NO. 161F265BA REV.4; SSI DWG. NO. S-15116. MPL NO. E21-1010 LVN-F104

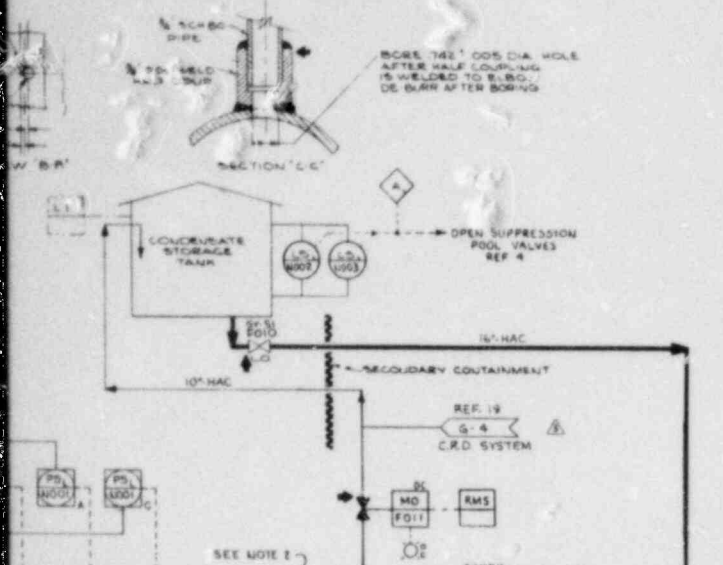
<b>BECHTEL ASSOCIATES</b> JOB 8511	
<b>SOUTHERN SERVICES INC.</b> FOR	
<b>GEORGIA POWER CO., ATLANTA, GA.</b> GENERAL ENGINEERING DEPARTMENT	
<b>EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1</b> CORE SPRAY SYSTEM 0 & 10	
DRAWN BY: <i>[Signature]</i> SCALE: 1/4" = 1'-0" DATE: 11/17/70	CHECKED BY: <i>[Signature]</i> DATE: 11/17/70
DRAWING NUMBER 10-502	SHEET NO. H-6331

NO.	DATE	BY	REVISION
1	JAN 71	REVISED PER DCR 77-229	
2	4-15-71	FORWARDED AS NOTED	
3	11-10-70	REV. PER ENGR. COM.	

LITHO NEG.







**REFERENCE DOCUMENTS:**

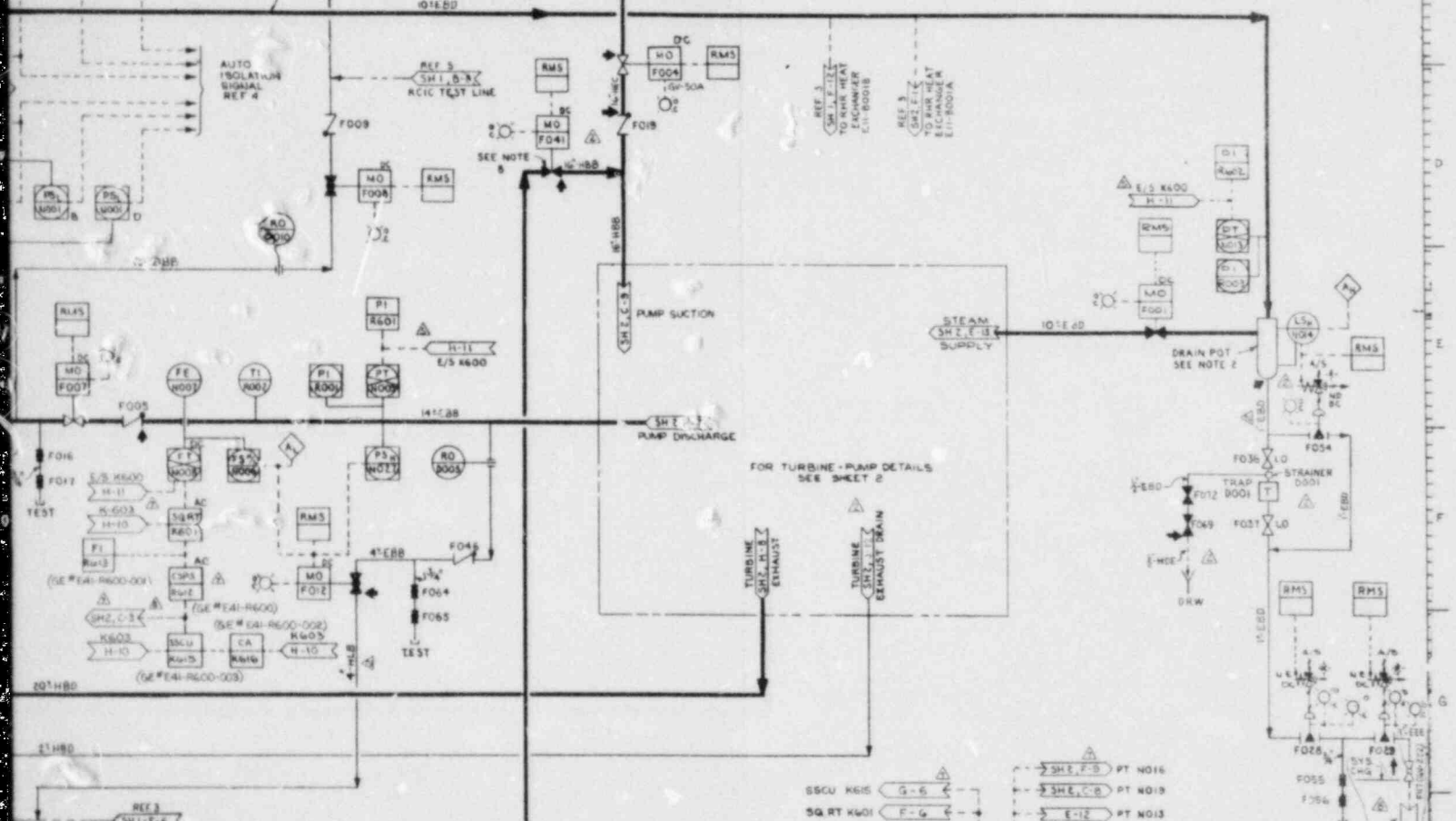
1. NUCLEAR BOILER SYS PAID SHT 1&2
2. NUCLEAR BOILER SYS PCD
3. RWI SYS PAID, SHT 1, 2, 3, 4, 5, 6, 7, 8, 9, 10, 11, 12, 13, 14, 15, 16, 17, 18, 19, 20, 21, 22, 23, 24, 25, 26, 27, 28, 29, 30, 31, 32, 33, 34, 35, 36, 37, 38, 39, 40, 41, 42, 43, 44, 45, 46, 47, 48, 49, 50, 51, 52, 53, 54, 55, 56, 57, 58, 59, 60, 61, 62, 63, 64, 65, 66, 67, 68, 69, 70, 71, 72, 73, 74, 75, 76, 77, 78, 79, 80, 81, 82, 83, 84, 85, 86, 87, 88, 89, 90, 91, 92, 93, 94, 95, 96, 97, 98, 99, 100
4. RWI SYS PCD
5. RWI SYS PAID, SHT 1
6. NUCLEAR BOILER LEAK DETECTION SYS PAID SHT 1&2
7. REACTOR WATER CLEANUP SYS PAID SHT 1&2
8. TURBINE CONTROLS DIAGRAM
9. TURBINE SHT LINE DRAWING
10. PLANT REQUIREMENTS
11. PROCESS INSTRUMENT PIPING AND TUBING INSTALLATION SPEC.
12. PIPING & INSTRUMENT SYMBOLS
13. PUMP, PIP, DEL & ERECT TYPING & SETTINGS OF MECH EQUIPMENT
14. RWI SYS DESIGN SPEC
15. RWI SYS PD
16. REACTOR BUILDING VENTILATION
17. PRIMARY CONTAINMENT PURGE & INERTING P&ID
18. DRIVE/REL VALVE & EQUIP DRAINAGE SYS P&ID
19. C.R.D. SYSTEM P&ID SHT 2

**LEGEND:**

- CA - CONTROL AMPLIFIER
- CSPS - CONTROLLER SET POINT STATION
- SSCU - SELF SYNCHRONIZING CONTROL UNIT

**NOTES:**

1. CHEMICAL CLEANING CONNECTIONS, VALVES, ETC., IF REQUIRED ARE TO BE PROVIDED AS NECESSARY.
2. LINE TO SUPPLY DRAIN FROM POINT OF CONNECTION TO MAIN STEAM LINE TO THE DRAIN POT AREA OF THE TURBINE WITH NO POINTS (DRAIN). LOCATE DRAIN POT AND VALVE FOO AS CLOSE AS POSSIBLE TO TURBINE SHEET (F-13).
3. INSTRUMENT LINE WELDING MUST COMPLY WITH REF 11.
4. DESIGN PRESSURE & TEMP. TO BE ESTABLISHED AND/OR RWI PUMP TEST-SET PRESSURE & FEEDWATER TEMP. HIGH POINT TESTS & DRAWING TO BE ACCORD AS NECESSARY.
5. EQUIPMENT & INSTRUMENTS ARE PRETYPED BY SYSTEM NO. (E/S) UNLESS OTHERWISE NOTED.
6. LOCATE VALVE FOO (2) (E/S) AS CLOSE AS POSSIBLE TO PUMP SUCTON LINE FROM CONDENSATE STORAGE.
7. FOR INTERLOCKING REQUIREMENTS SEE FUNCTIONAL CONTROL DIAGRAM REF. 4.
8. PENETRATION LOCATION TO BE ABOVE MAXIMUM EXPECTED WATER LEVEL, WITH PIPING EXTENDED 3 FT. BELOW LOW WATER LEVEL (E/S).
9. PENETRATION LOCATION TO BE MAXIMUM 3 FT. ABOVE NORMAL WATER LEVEL, WITH PIPING EXTENDED 3 FT. BELOW LOW WATER LEVEL (E/S).
10. THE BAROMETRIC CONDENSER AND SUCTON TANK SHALL BE LOCATED TO TEST ITS WATER LEVEL TO BELOW THE BOTTOM OF THE TURBINE EXHAUST.
11. T&C PIPING FOR RWI INSTRUMENTS SHALL BE DERIVED FROM THE PLANT T&C PIPING SYSTEM, UNLESS OTHERWISE NOTED. T&C DRAWING IS TO BE SEPARATE FROM THIS WHICH SUPPLES THE RWI SYSTEM.

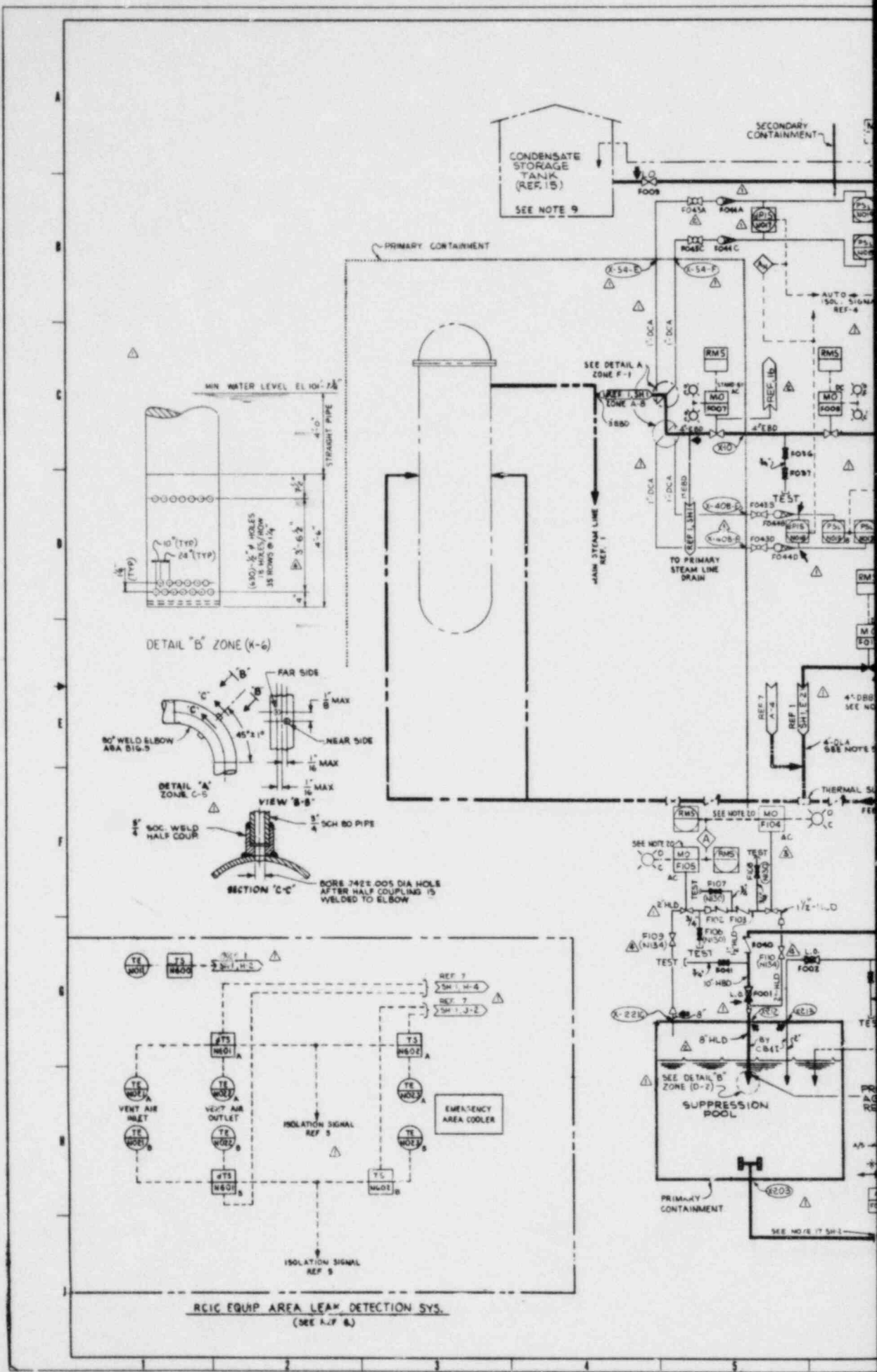


THIS DRAWING DEVELOPED FROM S.E. DWG. NO. 723E-0008A SHT 1, REV. 4, S.S.I. DWG. NO. S-1G150.

DC TO AC INVERTER	K808	E/S K600	POWER SUPPLY
SSCU KW6	G-6	SHT E-5	PT NO16
SQRT KW01	F-6	SHT C-8	PT NO19
CA KW6	G-6	E-12	PT NO13
		F-8	PT NO09
		F-8	FT NO08

RECD TEL ASSOCIATES  
JOB 8511  
SOUTHERN SERVICES INC.  
FBB  
GEORGIA POWER CO., ATLANTA, GA  
GENERAL ENGINEERING DEPARTMENT  
EDWIN L. HATCH NUCLEAR PLANT UNIT NO. 1  
HPCI SYSTEM PAID  
SHEET NO. 1

DATE: 10-5-77  
DRAWING NUMBER: 10-502  
SHEET NO: H-6332



**NOTES**

1. CHEMICAL CLEANING CONNECTIONS VALVES ETC IF REQUIRED, ARE TO PROVIDED AS NECESSARY.
  2. SLOPE STEAM LINE DOWN ALL THE WAY FROM MAIN STEAM LINE TO DRAIN POT JUST AHEAD OF TURBINE.
  3. INSTRUMENT LINE VALVING MUST COMPLY WITH INSTRUMENT PIPING STANDARDS.
  4. DESIGN PRESSURE & TEMP TO BE ESTABLISHED BASED ON MAIN FEED PUMP SHUT OFF PRESSURE AND/OR RCIC PUMP SHUT-OFF PRESSURE & FEEDWATER TEMP.
  5. EQUIPMENT VENT & DRAIN QUANTITIES SHOWN ARE TO BE MODIFIED TO AGREE WITH VPF DATA FOR ACTUAL EQUIPMENT PURCHASED. PIPING HIGH POINT VENTS & LOW POINT DRAINS TO BE ADDED AS NECESSARY.
  6. EQUIPMENT AND INSTRUMENTS ARE PREFIXED BY MDL NO E51 UNLESS OTHERWISE NOTED.
  7. LOCATE VALVE FO29 (ZONE D-B) AS CLOSE AS POSSIBLE TO PUMP SUCTION LINE FROM CONDENSATE STORAGE.
  8. REQUIRED TOTAL RESERVE STORAGE FOR RCIC SVS AND HPCI SVS, 100,000 GAL. THIS AMOUNT OF STORAGE SHALL BE CAPABLE OF BEING ISOLATED FROM SERVING OTHER SYSTEMS.
  9. "A-C" POWER FOR RCIC INSTRUMENTS SHALL BE DERIVED FROM A "D-C" SOURCE SEPARATE FROM THAT WHICH SUPPLIES THE HPCI SYSTEM.
  10. FOR INTERLOCKING REQUIREMENTS AND AUTO VALVE ACTUATION SEE FUNCTIONAL CONTROL DIAGRAM REF. 2.
- NOTES CONT'D ON SH. 2.

**REFERENCES**

REF DOCUMENTS	MDL ITEM NO.	RECHTEL OR S.S.I. NO.
1. NUCLEAR BOILER SVS P&ID SHTS 142	B21-1000	H-10062 (4) 93
2. NUCLEAR BOILER SVS FCD	B21-1030	S-15267
3. RWR SVS P&ID	E11-010	H-10325
4. HPCI SVS P&ID	E41-010	H-16332
5. RCIC SVS FCD	E51-030	S-1558
6. NUCLEAR BOILER LEAK DETECTION SVS DESIGN SPEC.	AG-4040	S-19530
7. RWCU SVS P&ID SHTS 142	G31-100	H-10688-89
8. PIPING & INSTRUMENT SYMBOLS	AA-1000	S-15051
9. PROCESS INSTRUMENT PIPING AND TUBING INSTALLATION SPEC.	AG-4070	S-15424
10. PLANT REQUIREMENTS	AG-4020	S-17109
11. TURBINE CONTROL DWG.	VFP 2757-014	
12. PURN, FAB & DEL OF PIPING & SETTINGS OF MECH EQUIP		SS-4797-1
13. TURBINE OUTLINE	E51-002	S-15496
14. HPCI SVS FCD	E41-030	S-16155
15. REACTOR & RADWASTE BUILDINGS CONDENSATE STORAGE & TRANSFER SVS DIAGRAM	PH-1010	H-16616
16. DRYWELL VALVE & EQUIP DRAINAGE SVS P&ID		H-16199
17. PRIMARY CONTAINMENT PURGE & INERTING SVS P&ID	TAE-1020	H-16024

**LEGEND**

- CA - CONTROL AMPLIFIER
- BSCU - SELF SYNCHRONIZING CONTROL UNIT
- CSPS - CONTROL SET POINT STATION

THIS DWG DEVELOPED FROM GE DWG. NO 727604ARA SHT 1, REV. 4, SSI DWG. NO 514-57.

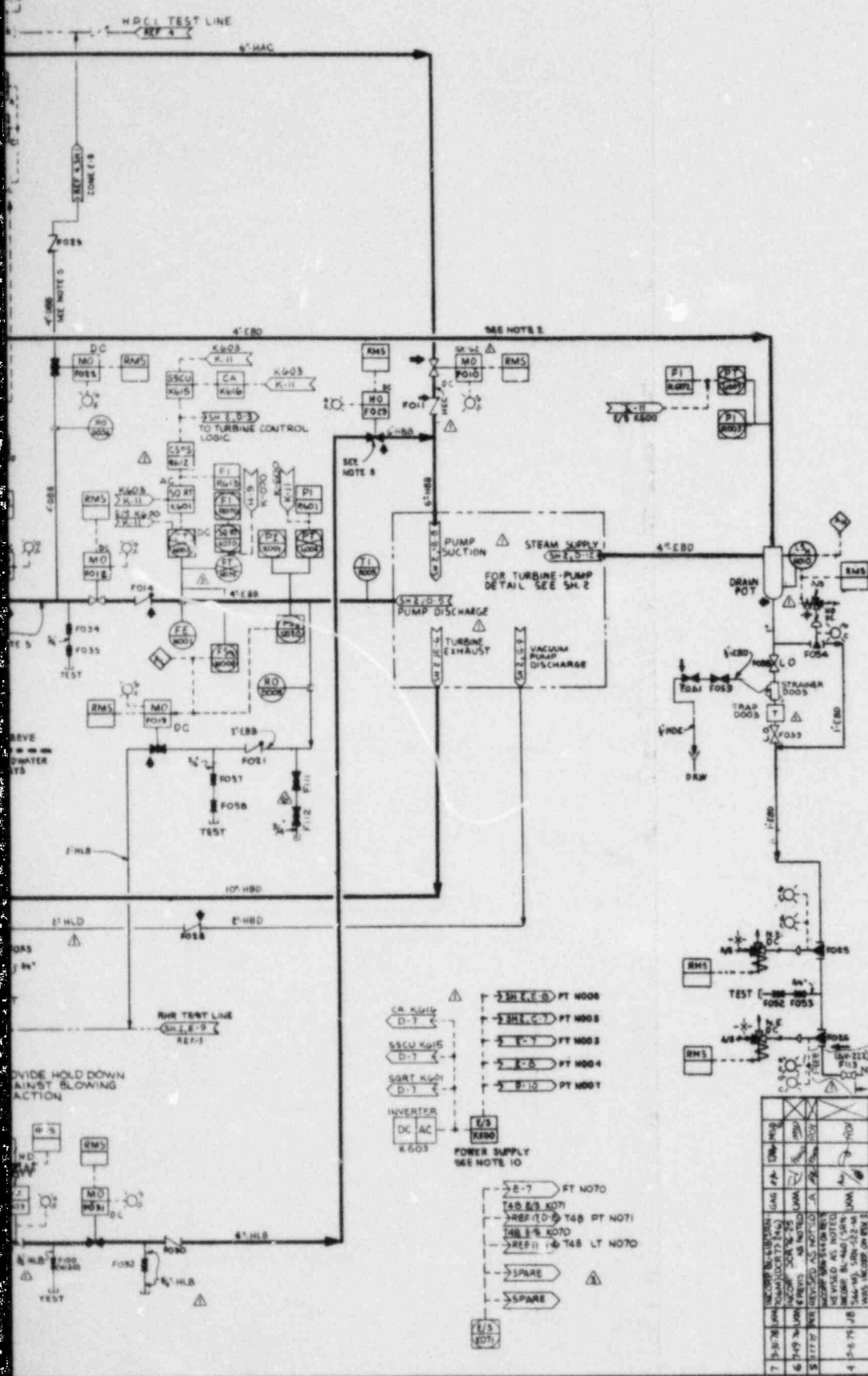
MDL NO E51-100

**RECHTEL ASSOCIATES**  
100 8511

**SOUTHERN SERVICES INC.**  
FOR

**GEORGIA POWER CO., ATLANTA, GA.**  
GENERAL ENGINEERING DEPARTMENT  
EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1  
RCIC SYSTEM P&ID  
SHEET NO. 1

DATE	BY	SCALE	DATE
10-5-67	...	1:10	...
DRAWING NUMBER		SHEET NO.	
10-502		1-16334	



- CA K616 → SHLE-9 PT N008
- D-7 → SHLE-9 PT N008
- BSCU K615 → C-7 PT N003
- D-7 → C-5 PT N004
- SGRT K601 → E-10 PT N001

INVERTER  
DC AC  
K 603  
POWER SUPPLY  
SEE NOTE 10

- E-7 → FT N070
- T48 & S N071
- REF ID → T48 PT N071
- T48 & S N070
- REF ID → T48 LT N070
- SPARE
- SPARE

E/S  
ED1

PROVIDE HOLD DOWN AGAINST BLOWING ACTION