



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

February 22, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 84-047-00
ER 100450 FILE 841-23
PLAS-042

Docket No. 50-387
License No. NPF-14

Attached is a copy of Licensee Event Report 84-047-00. This was originally issued November 30, 1984. Due to apparent distribution problems, this is being reissued.

A.P. Piemontese
Compliance

SJS/pjg

Attachment

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Mr. R.H. Jacobs
Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 52
Shickshinny, PA 18655

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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7	PAGE(S) 1 OF 0 1
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TITLE (4)
RWCU Isolation on High Flow Following 'A' Demin. precoat and backwash.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																
1	1	0	2	8	4	8	4	0	4	7	0	0	1	1	3	0	8	4			0	5	0	0	0		

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10) 1 1 0 1 0	20.402(b)	<input checked="" type="checkbox"/>	80.73(a)(2)(iv)	<input type="checkbox"/>	73.71(b)					
	20.406(a)(1)(i)	<input type="checkbox"/>	80.73(a)(2)(v)	<input type="checkbox"/>	73.71(e)					
	20.406(a)(1)(ii)	<input type="checkbox"/>	80.73(a)(2)(vi)	<input type="checkbox"/>	OTHER (Specify in Abstract below and in Text, NRC Form 306A)					
	20.406(a)(1)(iii)	<input type="checkbox"/>	80.73(a)(2)(vii)(A)	<input type="checkbox"/>						
	20.406(a)(1)(iv)	<input type="checkbox"/>	80.73(a)(2)(vii)(B)	<input type="checkbox"/>						
	20.406(a)(1)(v)	<input type="checkbox"/>	80.73(a)(2)(ix)	<input type="checkbox"/>						

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME Benjamin L. Wilks		AREA CODE 7 1 7	5 4 2 - 3 9 1 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
A	CIE	IPIDIS	B101810	N					

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)		
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Shortly before 1300 hours on November 2, 1984, Operations personnel were in the process of placing the 'A' Reactor Water Cleanup (RWCU) Filter Demineralizer in service in accordance with OP-161-001 "Reactor Water Cleanup" following backwash and precoat operations. During deisolation of the 'A' Filter Demineralizer, total RWCU system flow exceeded the system's high flow trip settings (approximately 415 gpm), causing system containment isolation valves 1F001 and 1F004 to shut.

The cause of the system isolation was determined to be the result of a surge in system flow created by excessive air in the 'A' Filter Demineralizer as the result of an extended period of precoat operation. The System isolation valves were opened at 1305 hours.

As a result of this occurrence, RWCU system operating procedures are being revised to ensure the demineralizers are filled and vented prior to deisolation; precoat operations will be limited in time. This event will be reviewed with Operations personnel.

Isolation of the RWCU system is an Engineered Safety Feature (ESF) actuation due to the closure of the system's containment isolation valves. The occurrence of this event had no adverse effects on the health and safety of the public.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Susquehanna Steam Electric Station UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7 8 4	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	- 0 4 7	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Shortly before 1300 hours on November 2, 1984, Operations personnel were in the process of placing the 'A' RWC Filter Demineralizer in service in accordance with OP-161-001 "Reactor Water Clean Up" following backwash and precoat operations. The Filter Bypass valve was closed and RWC System flow was reduced to approximately 210 gpm. During deisolation of the 'A' Filter Demineralizer, total RWC system flow quickly increased to greater than 400 gpm, exceeding the system high flow trip settings causing valves 1F001 and 1F004 to go shut, isolating the RWC system from the plants reactor coolant system. The high flow trip settings for valves 1F001 and 1F004 are 415 gpm and 417 gpm respectively.

A Limiting Condition for Operation was entered at 1300 hours in accordance with T.S. 3.4.4 to sample Reactor Coolant once every four hours, but was cleared at 1305 hours when it was learned that the cause for the occurrence was a surge in system flow created by excessive air in the 'A' Filter Demineralizer as the result of an extended period of precoat operation; the air was drawn into the Demineralizer during precoat operations from the precoat tank. The Demineralizer precoat cycle normally last eighteen minutes, but can be extended if Chemistry determines it needs to be.

As a result of this occurrence, OP-161-001 (Unit 1) and OP-261-001 (Unit 2) are being revised to ensure RWC Filter Demineralizers are filled and vented prior to deisolation and opening of the demineralizer inlet valve. In addition, a precaution will be added to limit demineralizers precoat operating time. This event has been placed on the Operations Supervisor's Training Agenda and will be reviewed with Operations personnel.

Isolation of the RWC System is an Engineered Safety Feature (ESF) actuation due to the closure of the systems Containment Isolation Valve. The occurrence of this event had no adverse effects on the health and safety of the public.

PP&L

SUSQUEHANNA STEAM ELECTRIC STATION
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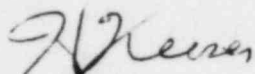
November 30, 1984

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Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 84-047-00
ER 100450 FILE 841-23
PLAS- 015

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report 84-047-00. This event was determined reportable per 10CFR50.73(a)(2)(iv), in that an unplanned Engineered Safety Feature (ESF) actuation occurred.



H.W. Keiser
Superintendent of Plant-Susquehanna

BLW/pjg

cc: Dr. Thomas E. Murley
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