



Northern States Power Company

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February 21, 1985

U S Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT  
Docket No. 50-282 License No. DPR-42

One Containment Isolation Valve Failed Its  
Local Leakage Rate Test

The License Event Report for this occurrence is attached.

This event was reported via Emergency Notification System per 10 CFR Part 72  
on January 23, 1985.

David Musolf  
Manager - Nuclear Support Services

DMM/EFE/dab

c: Regional Administrator-III, NRC  
NRR Project Manager, NRC  
Resident Inspector, NRC  
MPCA  
Attn: J W Ferman

Attachment

8503040494 850221  
PDR ADOCK 05000282  
S PDR

IE22  
11

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Prairie Island Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 8 2	PAGE (3) 1 OF 0 2
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TITLE (4)  
One Containment Isolation Valve Failed Its Local Leakage Rate Test

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)									
0	1	2	3	8	5	8	5	0	0	5	0	0	0	2	1	8	5			0 5 0 0 0
																			0 5 0 0 0	

OPERATING MODE (9)  N

POWER LEVEL (10)

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

20.402(b)	<input type="checkbox"/>	20.406(c)	<input type="checkbox"/>	50.73(a)(2)(iv)	<input type="checkbox"/>	73.71(b)	<input type="checkbox"/>
20.406(a)(1)(i)	<input type="checkbox"/>	50.36(e)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	<input type="checkbox"/>	73.71(e)	<input type="checkbox"/>
20.406(a)(1)(ii)	<input type="checkbox"/>	50.36(c)(2)	<input type="checkbox"/>	50.73(a)(2)(vi)	<input type="checkbox"/>	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	<input type="checkbox"/>
20.406(a)(1)(iii)	<input type="checkbox"/>	50.73(a)(2)(i)	<input checked="" type="checkbox"/>	50.73(a)(2)(vii)(A)	<input type="checkbox"/>		<input type="checkbox"/>
20.406(a)(1)(iv)	<input type="checkbox"/>	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(vii)(B)	<input type="checkbox"/>		<input type="checkbox"/>
20.406(a)(1)(v)	<input type="checkbox"/>	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(x)	<input type="checkbox"/>		<input type="checkbox"/>

LICENSEE CONTACT FOR THIS LER (12)

NAME Arne A Hunstad, Staff Engineer	TELEPHONE NUMBER
	AREA CODE 6 1 2 3 8 8 - 1 1 2 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)       NO

EXPECTED SUBMISSION DATE (15)      MONTH    DAY    YEAR  
0 3    2 9    8 5

ABSTRACT (Limit to 1400 spaces i.e. approximately fifteen single-space typewritten lines) (16)

During local leakage rate testing at refueling, one containment isolation check valve exhibited leakage in excess of Tech Spec limits. The valve was replaced with a spare. A supplemental report will be submitted when the cause of the failure is determined.

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### LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  Prairie Island Unit 1	DOCKET NUMBER (2)  0 5 0 0 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	- 0 0 5	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During local leakage rate testing at refueling, one containment isolation check valve exhibited leakage in excess of Tech Spec limits. The valve was replaced with a spare. A supplemental report will be submitted when the cause of the failure is determined.