

Florida Power

February 27, 1985 3F0285-24

Director of Nuclear Reactor Regulation

Attention: Mr. Hugh L. Thompson, Jr., Director

Division of Licensing

U.S. Nuclear Regulatory Commission

Washington, D.C. 20555

Subject:

Crystal River Unit 3 Docket No. 50-302

Operating License No. DPR-72 NUREG-0737, Item II.K.3.5

Automatic Trip of Reactor Coolant Pumos

Dear Sir:

This submittal is in response to NRC letter dated February 8, 1983, "Resolution of TMI Action Item II.K.3.5, Automatic Trip of Reactor Coolant Pumps" (Generic Letter No. 83-10f). Florida Power Corporation has completed the analysis and has determined the required reactor coolant system pressure and subcooling margin temperature setpoints to trip the reactor coolant pumps during transients.

The B&W Owners Group produced two documents relative to this project. The first provided analytical justification for the new trip setpoint ("Generic Error Analysis and Setpoint Determination", B&W document number 77-1148304-01). The second described a technique for determining the setpoint based on instrument error ("TSAT Error Calculation", B&W document number 51-1153690-00).

The resultant Crystal River Unit 3 calculations are as follows:

RCS Pressure >1500 psig and subcooling < 20°F. RCS Pressure ≤1500 psig and subcooling < 50°F.

The scheduled date for implementation of these procedure change. prior to Mode 4 entry, following Refuel V. These changes will be coordinated with other changes due to modifications installed during the outage. FPC will no longer require that the reactor coolant pumps be tripped when the HPI actuation pressure (1500 psig) is reached.

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This effort completes the requirements of the NRC letter dated February 8, 1983 and NUREG-0737, Item II.K.3.5.

Sincerely,

G. R. Westafer

Manager, Nuclear Operations Licensing and Fuel Management

EMG/feb

cc: Dr. J. Nelson Grace

Regional Administrator, Region II Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission 101 Marietta Street N.W., Suite 2900 Atlanta, Ga. 30323