

**GULF STATES UTILITIES COMPANY**

RIVER BEND STATION    POST OFFICE BOX 220    ST. FRANCISVILLE, LOUISIANA 70778  
AREA CODE 504    635-7034    342-8651

July 20, 1992  
RBG- 37218  
File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Gentlemen:

River Bend Station - Unit 1  
Docket No. 50-458

Please find enclosed License Event Report No. 92-009 for River Bend Station - Unit 1. This report is submitted pursuant 10CFR50.73.

Sincerely,

*for Sale & Anderson!*  
W.H. Odell  
Manager - Oversight  
River Bend Nuclear Group

*205 PDR*  
LAE/PDG/FRC/GAB/IRH/DNL/kvm  
*wjls*

9207280152 920720  
PDR ADOCK 05000458  
S PDR

*JEZ*

cc: U.S. Nuclear Regulatory Commission  
611 Ryan Plaza Drive, Suite 400  
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NRC Resident Inspector  
P.O. Box 1051  
St. Francisville, LA 70775

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Mr. C.R. Oberg  
Public Utility Commission of Texas  
7800 Shoal Creek Blvd., Suite 400 North  
Austin, TX 78757

Louisiana Department of Environmental Quality  
Nuclear Energy Division  
P.O. Box 82135  
Baton Rouge, LA 70884-2135  
ATTN: Administrator

**LICENSEE EVENT REPORT (LER)**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F530), U.S. NUCLEAR REGULATORY COMMISSION WASHINGTON, DC 20555 AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1) **RIVER BEND STATION** DOCKET NUMBER (2) **0 5 0 0 0 4 5 8 1** PAGE 3 OF 4

TITLE (4) **DISCREPANCY IN TECHNICAL SPECIFICATION LED TO FAILURE TO PROPERLY PERFORM SURVEILLANCES TESTS**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
06	18	92	92	009	00	03	20	92			05000
											05000

OPERATING MODE (9) **5** THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(a)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(a)
<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(a)(1)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(a)
<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.36(a)(2)	<input type="checkbox"/> 50.73(a)(2)(iv)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input checked="" type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(iv)(B)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME **L.A. ENGLAND - DIRECTOR, NUCLEAR LICENSING** TELEPHONE NUMBER **5 0 4 3 8 1 - 4 1 4 5**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)  YES //  NO

EXPECTED SUBMISSION DATE (15) MONTH **09** DAY **30** YEAR **92**

ABSTRACT (Limit to 1400 space - i.e. approximately fifteen single space typewritten lines) (16)

On June 18, 1992, GSU determined that a discrepancy in Technical Specification (TS) 3.6.1.3, Table 3.6.1.3-1 resulted in the non-performance of required surveillance tests for valves 1SSR\*SOV133, SOV134, SOV140, and 1SSR\*V706. These valves are located in the post-accident sampling system (PASS) panel supply and return lines. The TS table included the penetrations for the containment atmosphere and monitoring system (CMS) 'A' hydrogen analyzer supply and return lines. However, the penetrations for the CMS 'B' hydrogen analyzer and PASS lines should have been included instead of the CMS 'A' lines because of the presence of non-safety-related piping in the PASS lines which branch off the CMS 'B' lines outside containment.

Corrective actions include revisions to the technical specifications and surveillance test procedures and the performance of leak tests on the subject valves. A root cause evaluation and a safety assessment will be performed and provided in a revision to this LER.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

APPROVED OMB NO 3150-0104  
EXPIRES 4/30/92

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530) U.S. NUCLEAR REGULATORY COMMISSION WASHINGTON, DC 20555 AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104) OFFICE OF MANAGEMENT AND BUDGET WASHINGTON, DC 20503

FACILITY NAME (1)  RIVER BEND STATION	DOCKET NUMBER (2)  0 5 0 0 0 4 5 8 9 2	LER NUMBER (3)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 2	0 0 9	0 0	0 2	OF 0 4

TEXT (if more space is required, use additional NRC Form 305A's) (17)

REPORTED CONDITION

On June 18, 1992, GSU determined that a discrepancy in Technical Specification (TS) 3.6.1.3, Table 3.6.1.3-1 resulted in the non-performance of required surveillance tests for valves (\*ISV\*) 1SSR\*SOV133, SOV134, SOV140, and 1SSR\*V706. These valves are located in the post-accident sampling system (\*IP\*) (PASS) panel supply and return lines. The TS table included the penetrations for the containment atmosphere and monitoring system (\*BB\*) (CMS) 'A' hydrogen analyzer supply and return lines. However, the penetrations for the CMS 'B' hydrogen analyzer and PASS lines should have been included instead of the CMS 'A' lines because of the presence of non-safety-related piping in the PASS lines which branch off the CMS 'B' lines outside containment. This condition created an annulus bypass leakage path. As a result of this discrepancy, the valves referenced above have not received the surveillance tests required by TS 4.6.1.3. Therefore, this event is reportable pursuant to 10CFR50.73(a)(2)(i)(B) as operation prohibited by the TS.

INVESTIGATION

Annulus bypass leakage is limited by TS 3.6.1.3.d, to the following:

A combined leakage rate of less than or equal to 13,500 cc/hr for all penetrations shown in Table 3.6.1.3-1 as annulus bypass leakage paths when pressurized to Pa (accident pressure), 7.6 psig.

Condition report (CR) 92-0280 was issued to document a discrepancy with respect to containment penetrations serving CMS lines. The penetrations (1KJB\*Z605E and F) for the CMS 'A' hydrogen analyzer supply and return lines (valves (\*ISV\*) 1CMS\*SOV31A and 35C for 1KJB\*Z605E and 1CMS\*SOV31C and 35A for 1KJB\*Z605F) are included in TS Table 3.6.1.3-1. However, penetrations 1KJB\*Z601E and F are not included in TS Table 3.6.1.3-1. These penetrations serve the CMS 'B' hydrogen analyzer and post-accident sampling system (PASS) panel supply and return lines. Valves 1CMS\*SOV35D, 31B, 1SSR\*SOV133 and 134 isolate penetration 1KJB\*Z601E and 1CMS\*SOV35B, 35D, 1SSR\*V706, and 1SSR\*SOV140 isolate penetration 1KJB\*Z601F. Figure 1 shows the penetrations with the valves indicated. The discrepancy identified in the CR was that the CMS 'A' lines, which were listed in the TS, did not meet the criteria for inclusion in the TS due to the fact that all of the piping was ASME class 2 (safety-related). However, the PASS panel supply and return lines include class 4 piping. This piping is non-safety-related and thus it should have been included in the TS.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATIONAPPROVED OMB NO. 3150-0104  
EXPIRES 4/30/92ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS  
INFORMATION COLLECTION REQUEST 500 HRS. FORWARD  
COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS  
AND REPORTS MANAGEMENT BRANCH (PS-30), U.S. NUCLEAR  
REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO  
THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE  
OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  RIVER BEND STATION	DOCKET NUMBER (2)  0500045892	LER NUMBER (3)		PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT IF more space is required, use additional NRC Form 206A (1/77)

Initially the investigation focused on the containment isolation valves (\*ISV\*) (the CMS valves). GSU evaluated the as-left leak rate test histories of valves 1CMS\*SOV35D, 31B, 35B, and 31D, and found the as-left leakage data to be within the allowables of TS 3.6.1.3. This led to the preliminary conclusion that this condition was not reportable. However, further investigation revealed that an annulus bypass leakage path was created due to the presence of the non-safety-related piping in the PASS panel piping supply and return lines. It was then found that valves 1SSR\*SOV133, SOV134, SOV140 and V706 have been stroke tested but not leak tested. Thus, GSU determined that a reportable condition (ie., failure to perform surveillances on the valves) existed.

ROOT CAUSE

A root cause evaluation is being conducted to determine the extent of this condition. The results of this evaluation will be provided in a revision to this LER by September 30, 1992.

CORRECTIVE ACTION

The following corrective actions have been identified through the initial evaluation of the subject condition. These actions will be re-evaluated based on the final root cause determination.

1. Surveillance Test Procedure (STP) 610-3827 will be revised to include testing of 1SSR\*SOV133, SOV134, SOV140, V706 in their existing condition to obtain baseline data for safety assessment. This STP will also be revised to delete 1CMS\*SOV31A, C and 1CMS\*SOV35A, C when the T/S change discussed below is approved. STPs 057-3880 and 057-3900 will be revised to include the SSR valve data.
2. Leak rate tests will be performed on the subject SSR valves.
3. A safety assessment will be performed on the unidentified annulus bypass pathway using the data collected from the leak tests.
4. A T/S change request will be prepared to correct Section 3.6.1.3. The USAR will be revised, as necessary, to insure agreement with the revised T/S.

SAFETY ASSESSMENT

A safety assessment will be performed to determine the impact of the as found leak rates on offsite doses. The results of this assessment will be provided in a revision to this LER by September 30, 1992.

NOTE: (\*\*) indicates system or component codes.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F-50) U.S. NUCLEAR REGULATORY COMMISSION WASHINGTON, DC 20555 AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (8)

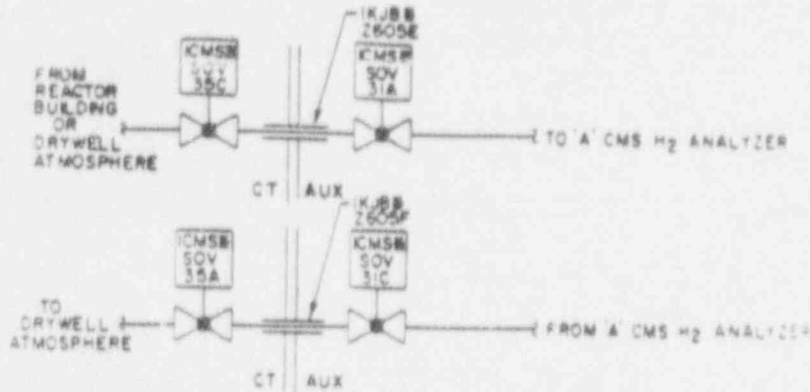
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RIVER BEND STATION

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TEXT (If more space is required, use additional NRC Form 305A's) (17)

DIV I CMS HYDROGEN ANALYZER PENETRATIONS  
1KJB\*Z605E & F



DIV II CMS HYDROGEN ANALYZER PENETRATIONS  
1KJB\*Z601E & F

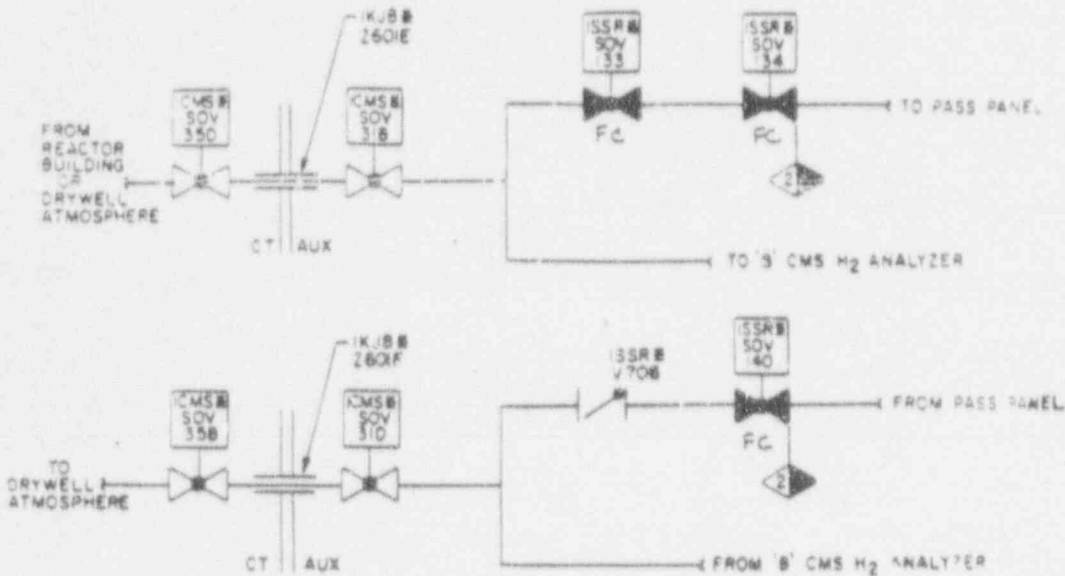


FIGURE 1