

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) BYRON, UNIT 1	DOCKET NUMBER (2) 050004541	PAGE (3) 1 OF 02
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TITLE (4)
LOSS OF COMPONENT COOLING (CC) SURGE TANK LEVEL

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
01	02	85	85	004	00	01	30	85			05000
											05000

OPERATING MODE (9) 5

POWER LEVEL (10) 000

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(a)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(a)(1)	<input checked="" type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(a)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(a)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(vii)(A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(vii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 50.73(a)(2)(v)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: John Bartleman, System Test Engineer, Ext. 606

TELEPHONE NUMBER: 8115 21341-15441

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
A	C	C	SPPW	120	N				

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces - i.e., approximately fifteen single-space typewritten lines) (16)

The Unit 1 Component Cooling (CC) Surge Tank level decreased due to construction personnel unbolting an improper flange which began leaking. The Unit 1 CC Surge Tank level was restored by initiating make-up flow to the tank. Construction management notified their personnel of the importance of working under proper clearances and authorization so that incidents such as this will not recur.

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PDR ADOCK 05000454
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) BYRON, UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 4 5 4	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	- 0 0 4	- 0 0 0	2	OF

TEXT (If more space is required, use additional NRC Form 386A's) (17)

On 1/2/85 at 1920, during Mode 5, the Unit 1 operator noticed a rapidly decreasing Unit 1 Component Cooling System (CC) Surge Tank level. The level loss was due to construction personnel unbolting a flange on the 2A Residual Heat Removal System (RH) Heat Exchanger return line which was crosstied to the CC Pumps Suction Header. The flange was unbolted to reinstall an orifice plate which had been removed during flushing. Construction had misunderstood a request to restore certain portions of the Unit 2 CC System which were covered under Out of Service #84-2-8115. Construction unbolted the flange thinking it was covered by this Out of Service when in fact it was not, therefore when the bolts from the flange were removed water from the suction side of the CC Pumps began running out of the pipe and the Unit 1 CC Surge Tank level started decreasing.

This incident was caused by improper review of the Out of Service to see what was physically being isolated. The Out of Service only covered cleaning and inspection of the Unit 2 CC Surge Tank and did not isolate the piping being worked on. Work on the installation of the orifice plate was not covered by any other Out of Service.

The Unit 1 CC Surge Tank level dropped to 30% before make-up flow was established. The CC Pumps did not trip due to the restoration of the Surge Tank level. Since the CC pumps remained operational there was no affect on plant and public safety. The requirement to work under proper clearances was discussed and stressed during a construction management meeting. Construction management will review this concern with their personnel to prevent recurrence.

No previous occurrences have been identified.



Commonwealth Edison
Byron Nuclear Station
4450 North German Church Road
Byron, Illinois 61010

January 30, 1985

LTR: BYRON 85-0162

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR 50.73 (a) (2) (v) which requires a 30 day written report.

This report is number 85-004-00, Docket No. 50-454.

Very truly yours,

R. E. Querio
Station Superintendent
Byron Nuclear Power Station

Enclosure: Licensee Event Report No. 85-004-00

cc: J. G. Keppler, NRC Region III Administrator
J. Hinds, NRC Resident Inspector
INPO Record Center
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