

JAN 24 1985

Docket No.: 50-412

Mr. Earl J. Woolever
Vice President, Nuclear Construction
Duquesne Light Company
Robinson Plaza Building, No. 2, Suite 210
PA Route 60
Pittsburgh, Pennsylvania 15205

Dear Mr. Woolever:

Subject: Beaver Valley Unit 2 Safety Review Requests for Additional
Information (RAI)

Enclosed please find the RAI relating to Quality Assurance. To maintain a workable review schedule, your response to these requests should be submitted to the NRC by February 4, 1985.

Should you have any questions concerning the enclosed requests please contact the Project Manager, B. K. Singh at 301-492-8423.

Sincerely,

Original signed by:
George W. Knighton

George W. Knighton, Chief
Licensing Branch No. 3
Division of Licensing

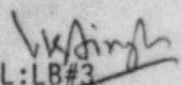
Enclosure:
As stated

cc: See next page

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Beaver Valley 2
Request for Additional Information

260.0 Quality Assurance Branch

260.64 Section 17.1.2.2 of the standard format (Regulatory Guide 1.70) requires the identification of safety-related structures, systems, and components controlled by the QA program. You are requested to supplement and clarify the Beaver Valley 2 FSAR in accordance with the following:

- a. The following items do not appear on Table 3.2-1. Add the appropriate items to the table and commit to apply the pertinent QA program requirements to the remaining items during the operations phase or justify not doing so.
 1. Biological shielding within the reactor containment, safeguards building, auxiliary building, control building, and other areas as appropriate.
 2. Missile barriers within the reactor containment, service building, auxiliary building, fuel and decontamination building, emergency diesel generator building, safeguards building, control building, main steam valve area, and other areas as appropriate.
 3. Fabricated supports such as Unistrut or Superstrut that are used to support systems and components identified in Regulatory Guide 1.29.
 4. Items that are within the scope of Regulatory Position C.2 and C.3 of Regulatory Guide 1.29.
 5. Spent fuel pool and liner.
 6. Fuel assemblies.
 7. Core support structure.
 8. Fuel transfer system.
 9. Steam generator steam flow restrictors.
 10. Station stack.
 11. Portions of the reactor plant sampling system that are Safety Class 2 or Safety Class 3.
 12. Quench spray system chemical injection pumps.
 13. Reactor coolant pump seals.

14. Service building.
15. Auxiliary building.
16. Cable vault.
17. Pipe tunnels.
18. Foundations for:
 - a) Electrical conduit ducts (including manholes).
 - b) Service water pipes.
 - c) Sheetpile retaining walls adjacent to primary intake structure.
 - d) Cooling water discharge pipes.
19. Modifications to site grading (paving, roads, railroads, etc.)
20. Containment sump.
21. Containment penetrations, air locks, and access hatches.
22. Control building radiation monitor (outside air intake).
23. Control building chlorine monitor (outside air intake).
24. Fuel building radiation monitor.
25. Expendable and consumable items necessary for the functional performance of safety-related structures, systems, and components (i.e., fuel oil, weld rod, boric acid, snubber oil, etc.)
26. Accident-related meteorological data collection equipment.
27. Standby AC Auxiliary Power Systems (Class 1E)
 - a) Diesel generator packages including auxiliaries (e.g., lube system, jacket cooling, air start system, governor, voltage regulator, and excitation system).
 - b) Instrumentation, control and power cables (including underground cable system, cable splices, connectors, and terminal blocks).
 - c) Conduit, cable trays, and their supports*
 - d) Valve operators.

* Raceway installations containing Class 1E cables and other raceway installations required to meet seismic Category 1 requirements (those whose failure during a seismic event may result in damage to any Class 1E or other safety related system or components).

- e) Electrical penetration for containment - vital and non-vital including primary and backup fault current protective devices.
 - f) Emergency lighting battery packs.
28. DC Power Systems (Class 1E)
- a) Conduit, cable trays, and their supports*
 - b) Battery racks
- b. Provide a commitment that all safety-related instrumentation and controls (I&C) described in Sections 7.1 through 7.6 of the FSAR and other safety-related I&C for safety-related fluid systems will be subject to the pertinent requirements of the FSAR Appendix B QA program. This can be done by a footnote to FSAR Table 3.2-1.
- c. Enclosure 2 of NUREG-0737, "Clarification of TMI Action Plan Requirements" (November 1980) identified numerous items that are safety-related and appropriate for OL application and therefore should be on Table 3.2-1. These items are listed below. Add the appropriate items to Table 3.2-1 and provide a commitment that the remaining items are subject to the pertinent requirements of the FSAR operational QA program or justify not doing so.

- | | |
|---|-------------------|
| 1. Reactor coolant system vents. | II.B.1 |
| 2. Plant shielding. | II.B.2 |
| 3. Valve position indication. | II.D.3. |
| 4. Auxiliary feedwater system. | II.E.1.1 |
| 5. Auxiliary feedwater system initiation and flow. | II.E.1.2 |
| 6. Emergency power for pressurizer heaters. | II.E.3.1 |
| 7. Dedicated hydrogen penetrations. | II.E.4.1 |
| 8. Containment isolation dependability. | II.E.4.2 |
| 9. Accident monitoring instrumentation. | II.F.1 |
| 10. Instrumentation for detection of inadequate core cooling. | II.F.2 |
| 11. Power supplies for pressurizer relief valves, block valves, and level indicators. | II.G.1 |
| 12. Automatic PORV isolation. | II.K.3(1) |
| 13. Automatic trip of reactor coolant pumps. | II.K.3(5) |
| 14. PID controller. | II.K.3(9) |
| 15. Anticipatory reactor trip on turbine trip. | II.K.3(12) |
| 16. Power on pump seals. | II.K.3(25) |
| 17. Emergency plans (and related equipment). | III.A.1.1/III.A.2 |
| 18. Equipment and other items associated with the emergency support facilities. | III.A.1.2 |
| 19. Inplant I ₂ radiation monitoring. | III.D.3.3 |
| 20. Control room habitability. | III.D.3.4 |