NRC Form (9-83)	LICENSEE EVENT REPORT (LER)														U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 31500104 EXPIRES. 8/31/85								
FACILITY	ACILITY NAME (1)															Tpo	CKET NUMBER	(2)	-		PAGI	(3)	
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POWE	POWER LEVEL (10)			-	20.406(a)(1)(i) 50					50.36	20.406(e) 60.36(e)(1) 60.36(e)(2)			50.73(a) (2)(iv) 50.73(a) (2)(vii) 50.73(a) (2)(viii) (A) 50.73(a) (2)(viii) (B) 50.73(a) (2)(viii) (B)			-	73.71(e) 73.71(e) OTHER (Specify in below and in Text. I					
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Terry M. McGhee, Acting Technical																	AREA CODE	Г		NE NUM		_	
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

The B Pressurizer Code Safety Relief Valve was sent out for testing during the Fourth Refueling Outage due to abnormal temperature indications received during startup following the Third Refueling Outage. The valve was tested for set pressure and leakage. The valve successfully met the leakage test requirements on both the pilot and main discs. The "as received" set pressure of the pilot cartridge was not within the acceptable limits of 2485 \pm 1%. The set pressure of the cartridge was adjusted and the pilot successfully retested.

9502070402 850111 PDR ADDCK 05000334 PDR LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES. 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

YEAR

SEQUENTIAL
REVISION
NUMBER

NUMBER

REVISION
NUMBER

DEST (If more space is required, use edditional NRC Form 3884's) (17)

The B Pressurizer Code Safety Relief Valve was sent out for testing during the Fourth Refueling Outage due to abnormal temperature indications received during startup following the Third Refueling Outage. On 12/08/84 after evaluation of the test data, received from Wyle Laboratories, the Maintenance Department determined that the "as found" lift point check was not within the limits of $2485 \pm 1\%$ required by Plant Technical Specifications. The valve successfully met the leakage test requirements on both the pilot and main discs. The pilot cartridge was adjusted to meet the lift point requirements and successfully retested for leak rate and lift point.

The failure of the valve to meet the $\pm 1\%$ tolerance has been attributed to setpoint drift as no other problems were found and the "as found" lift point was within 2.1% of the required value. There were no safety implications resulting from this incident as the failure was discovered during testing and even though the Tech. Spec. maximum allowable limit of 2485 \pm 1% could have been exceeded, the maximum RCS pressure would still have remained far below the safety limit pressure of 2735 PSIG. In addition, both redundant pressurizer code safety valves were fully operational.



Nuclear Division P. O. Box 4 Shippingport, PA 15077-0004

> January 11, 1985 NDISS1:2322

Beaver Valley Power Station, Unit No. 1 Docket No. 50-334, License No. DPR-66 LER 84-019-01

Dr. Thomas E. Murley Regional Administrator United States Nuclear Regulatory Commission Region I Park Avenue King of Prussia, PA 19046

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 84-019-01, 10 CFR 50.73.a.2.i, "Pressurizer Code Safety Valve Lift Setting Greater Than Technical Specification Limit".

Very truly yours,

Plant Manager

md

Attachment

1/1 IE 32

T. E. Murley January 11, 1985 ND1SS1:2322 Page two

cc: Director of Management & Program Analysis
United States Nuclear Regulatory Commission
Washington, D.C. 20555

C. A. Roteck, Ohio Edison

Director, Office of Inspection and Enforcement Headquarters United States Nuclear Regulatory Commission Washington, D.C. 20555

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