JAN 24 1985

Docket No.: 50-382

Mr. R. S. Leddick Senior Vice President - Nuclear Operations Louisiana Power and Light Company 142 Delaronde Street Post Office Box 6008 New Orleans, Louisiana 70174

Dear Mr. Leddick:

DISTRIBUTION Docket File 50-382

NRC PDR ACRS (16) Local PDR HRDenton DGEisenhut NSIC PRC System ECase

LB#3 Reading

JLee JWilson. EJordan NGrace STurk, OELD

Subject: Compliance of the Waterford Steam Electric Station, Unit 3, with

Appendix G of 10 CFR 50

Appendix G of 10 CFR 50 permits the Director of the Office of Nuclear Reactor Regulation to approve alternative fracture toughness test methods other than that required by 10 CFR § 50.55a(c)(3). However, as provided by paragraph III.A. of Appendix G, for reactor vessels constructed in accordance with an ASME Code earlier than the Summer 1972 Addenda of the 1971 Edition, prior to granting such approval the fracture toughness data and data analysis must be supplemented in an approved manner to demonstrate equivalence with the fracture toughness requirements of Appendix G. The reactor vessel for the Waterford Steam Electric Station, Unit 3 was constructed to an ASME Code which is earlier than the Summer 1972 Addenda of the 1971 Edition.

Accordingly, for the reasons discussed in the Waterford Safety Evaluation Report, Supplement 8 (NUREG-0787, December 1984) I have determined that the alternative fracture toughness data presented by the applicant for Waterford Unit 3 are sufficient to demonstrate equivalence with the fracture toughness requirements of Appendix G of 10 CFR 50, and are, therefore, acceptable.

Sincerely.

Original Signed by H. R. Denten

Harold R. Denton, Director Office of Nuclear Reactor Regulation

cc: See next page

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